



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

REGION IV  
611 RYAN PLAZA DRIVE, SUITE 1000  
ARLINGTON, TEXAS 76011

MAY 22 1992

Docket Nos. 50-445  
50-446  
License No. NPF-87  
Construction Permit No. CPPR-127

TU Electric  
ATTN: W. J. Cahill, Jr., Group Vice President  
Nuclear Engineering and Operations  
Skyway Tower  
400 North Olive Street, Lock Box 81  
Dallas, Texas 75201

Gentlast  
emen:

SUBJECT: NRC INSPECTION REPORT NOS. 50-445/92-13; 50-446/92-13

This refers to the special inspection conducted by Messrs. R. M. Latta, A. C. Cerne, V. G. Gaddy, and M. F. Runyan during the period April 13-17, 1992, of activities authorized for the Comanche Peak Steam Electric Station, Units 1 and 2. Following the conclusion of this inspection, the findings were discussed on April 20, 1992, with you and those members of your staff identified in the enclosed report.

Areas which were examined during this Design Attribute Verification inspection included the evaluation of the translation of Unit 1 reverification requirements to Unit 2 as committed to in the Comanche Peak Steam Electric Station Corrective Action Program.

Within the scope of the inspection, no violations or deviations were identified. Inspection results indicated that the process for the translation of Unit 1 Postconstruction Hardware Validation Program (PCHVP) results to Unit 2 construction completion activities were well established and properly implemented and that they were commensurate with the methodologies utilized for Unit 1.

Notable strengths were identified with respect to strong management involvement and support for the implementation aspects of the Unit 2 PCHVP process and for the superior engineering efforts which were manifested in the conservative engineering justifications and detailed evaluations associated with the Unit 2 design validation process. Additionally, the enhanced Unit 2 installation procedures and specifications which incorporated validated design criteria and the additional assurance which was gained from the implementation of Corrective Action Requests were identified as strengths.

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Three observations were identified as a result of these inspection activities. The first observation involved the reliance on Unit 1 PCHVP results for rad. we... shrinkage versus the performance of a sample evaluation of Unit 2 piping welds which were subject to this phenomenon (paragraph 2.1.2). The second observation concerned the lack of specific procedural guidance for the establishment of the minimum sample size necessary to develop statistical conclusions (paragraph 2.2.1). The third observation pertained to the relatively large number of outstanding design changes associated with the specification for control room board design (paragraph 2.4.1).

In accordance with 10 CFR Part 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be place in the NRC Public Document Room.

Should you have any questions concerning this inspection, we will be pleased to discuss them with you.

Sincerely,



A. Bill Beach, Director  
Division of Reactor Projects

Encl'sure:  
Appendix - NRC Inspection Report  
50-445/92-13; 50-446/92-13  
w/Attachments A - E

cc w/enclosure:  
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MAY 22 1992

TU Electric  
NRC Inspection Report 50-445/92-13; 50-446/92-13

bcc to DMB (TE01)

bcc distrib. by RIV:

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5/19/92 5/19/92 5/22/92 5/19/92 5/19/92 5/19/92 5/22/92

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pe. Sullivan