



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20665

COMMONWEALTH EDISON COMPANY  
DOCKET NO. 50-373  
LASALLE COUNTY STATION, UNIT 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83  
License No. NPF-11

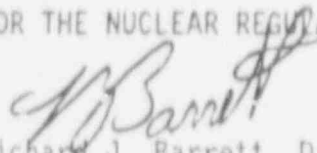
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated January 21, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 83 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director  
Project Directorate III-2  
Division of Reactor Projects - III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance. May 19, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 83

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. Pages identified by an asterisk are provided for convenience.

REMOVE

\* 3/4 9-7

3/4 9-8

INSERT

\* 3/4 9-7

3/4 9-8

## REFUELING OPERATIONS

### 3/4.9.5 COMMUNICATIONS

#### LIMITING CONDITION FOR OPERATION

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3.9.5 Direct communication shall be maintained between the control room and refueling platform personnel.

APPLICABILITY: OPERATIONAL CONDITION 5, during CORE ALTERATIONS.\*

ACTION:

When direct communication between the control room and refueling platform personnel cannot be maintained, immediately suspend CORE ALTERATIONS.\*

#### SURVEILLANCE REQUIREMENTS

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4.9.5 Direct communication between the control room and refueling platform personnel shall be demonstrated within one hour prior to the start of and at least once per 12 hours during CORE ALTERATIONS.\*

\*Except movement of incore instrumentation and control rods with their normal drive system.

## REFUELING OPERATIONS

### 3/4.9.6 CRANE AND HOIST

#### LIMITING CONDITION FOR OPERATION

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3.9.6 All cranes and hoists used for handling fuel assemblies or control rods within the reactor pressure vessel shall be OPERABLE.

APPLICABILITY: During handling of fuel assemblies or control rods within the reactor pressure vessel.

#### ACTION:

With the requirements for crane and hoist OPERABILITY not satisfied, suspend use of any inoperable crane or hoist from operations involving the handling of control rods and fuel assemblies within the reactor pressure vessel after placing the load in a safe condition.

#### SURVEILLANCE REQUIREMENTS

---

4.9.6 Each crane or hoist used for handling of control rods or fuel assemblies within the reactor pressure vessel shall be demonstrated OPERABLE within 7 days prior to the start of such operations with that crane or hoist by:

- a. Demonstrating operation of the overload cutoff when the load exceeds:
  1. For the fuel hoist:
    - a) 1600 +100/-0 pounds with the NF500 mast.
    - b) 1200 +50 pounds with the 762E974 mast.
  2. 1000 ± 50 pounds for the auxiliary hoist.
- b. Demonstrating operation of the loaded interlock when the load exceeds:
  1. For the fuel hoist:
    - a) 700 +50/-0 pounds with the NF500 mast.
    - b) 485 +50 pounds and 550 +50 pounds with the 762E974 mast.
  2. 400 ± 50 pounds for the auxiliary hoist.
- c. Demonstrating operation of the fuel hoist downtravel stop when downtravel exceeds 54 feet below the platform rails.
- d. Demonstrating operation of the fuel hoist and auxiliary hoist up-travel stops when the grapple is lower than or equal to 8 feet below the platform rails.
- e. Demonstrating operation of the fuel hoist slack cable cutoff when the hoist is unloaded.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20565

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67  
License No. NPF-18

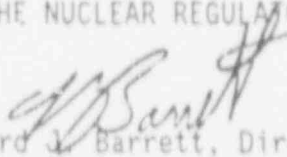
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  - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 67, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective upon date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Richard J. Barrett, Director  
Project Directorate III-2  
Division of Reactor Projects - III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: May 19, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 67

FACILITY OPERATING LICENSE NO. NPF-18

DOCKET NO. 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. Pages identified by an asterisk are provided for convenience.

REMOVE

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## REFUELING OPERATIONS

### 3/4.0.5 COMMUNICATIONS

#### LIMITING CONDITION FOR OPERATION

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APPLICABILITY: OPERATIONAL CONDITION 5, during CORE ALTERATIONS.\*

ACTION:

When direct communication between the control room and refueling platform personnel cannot be maintained, immediately suspend CORE ALTERATIONS.\*

#### SURVEILLANCE REQUIREMENTS

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4.9.5 Direct communication between the control room and refueling platform personnel shall be demonstrated within one hour prior to the start of and at least once per 12 hours during CORE ALTERATIONS.\*

\*Except movement of incore instrumentation and control rods with their normal drive system.

## REFUELING OPERATIONS

### 3/4.9.6 CRANE AND HOIST

#### LIMITING CONDITION FOR OPERATION

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3.9.6 All cranes and hoists used for handling fuel assemblies or control rods within the reactor pressure vessel shall be OPERABLE.

APPLICABILITY: During handling of fuel assemblies or control rods within the reactor pressure vessel.

#### ACTION:

With the requirements for crane and hoist OPERABILITY not satisfied, suspend use of any inoperable crane or hoist from operations involving the handling of control rods and fuel assemblies within the reactor pressure vessel after placing the load in a safe condition.

#### SURVEILLANCE REQUIREMENTS

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4.9.6 Each crane or hoist used for handling of control rods or fuel assemblies within the reactor pressure vessel shall be demonstrated OPERABLE within 7 days prior to the start of such operations with that crane or hoist by:

- a. Demonstrating operation of the overload cutoff when the load exceeds:
  1. For the fuel hoist:
    - a) 1600 +100/-0 pounds with the NF500 mast.
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- b. Demonstrating operation of the loaded interlock when the load exceeds:
  1. For the fuel hoist:
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- c. Demonstrating operation of the fuel hoist downtravel stop when downtravel exceeds 54 feet below the platform rails.
- d. Demonstrating operation of the fuel hoist and auxiliary hoist up-travel stops when the grapple is lower than or equal to 8 feet below the platform rails.
- e. Demonstrating operation of the fuel hoist slack cable cutoff when the hoist is unloaded.