

U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Report No. 50-305/89010(DRP)

Docket No. 50-305

License No. DPR-43

Licensee: Wisconsin Public Service Corporation  
P. O. Box 19002  
Green Bay, Wisconsin 54307-9002

Facility Name: Kewaunee Nuclear Power Plant

Inspection At: Kewaunee Site, Kewaunee, WI

Inspection Conducted: June 16 through July 31, 1989

Inspectors: R. L. Nelson

J. S. Stewart

Approved By: *R. DeFayette*  
R. DeFayette, Chief  
Reactor Projects Section 3A

9/6/89  
Date

Inspection Summary

Inspection from June 16 through July 31, 1989 (Report No. 50-305/89010(DRP))

Areas Inspected: Routine unannounced inspection by resident inspectors of operational safety; surveillance; maintenance; licensee event reports; and licensee self-assessment.

Results: No significant safety issues were identified. In general, the inspection results indicated a continuing good performance.

## DETAILS

### 1. Persons Contacted

- \*M. L. Marchi, Plant Manager
- K. H. Weinbauer, Assistant Manager, Plant Maintenance
- C. A. Schrock, Assistant Manager, Plant Operations
- \*R. E. Draheim, Assistant Manager, Plant Support
- J. J. Wallace, Superintendent, Plant Instrument and Control
- D. R. Berg, Superintendent, Plant Quality Control
- D. T. Braun, Superintendent, Plant Operations
- M. T. Reinhart, Superintendent, Plant Radiation Protection
- R. P. Pulec, Superintendent, Plant Technical
- D. S. Nalepka, Plant Licensing Supervisor
- \*P. M. Lindberg, Plant Engineering Projects Supervisor
- G. J. Youngworth, Plant Electrical Maintenance Supervisor
- \*F. D. Evitch, Plant Security Supervisor

The inspectors also talked with and interviewed members of the Operation, Maintenance, Health Physics, Instrument and Control, Quality Control, and Security groups.

\*Denotes personnel attending exit interviews.

### 2. Operational Safety Verification (71707)

The inspector observed control room operations, reviewed applicable logs and conducted discussions with control room operators throughout the inspection period. The inspector verified the operability of selected safety-related systems, reviewed tagout records, and verified proper return to service of affected components. Tours of the auxiliary and turbine buildings were conducted. During these tours, observations were made relative to plant equipment conditions, fire hazards, fire protection, adherence to procedures, radiological control and conditions, housekeeping, security, tagging of equipment, ongoing maintenance and surveillance, containment integrity, and availability of safety-related equipment.

The plant was taken off-line on June 17, 1989, to replace the #1 seal on reactor coolant pump 1B. The seal flow had slowly decreased since startup of the plant in April from the 1989 refueling outage. The seal flow had decreased to approximately 0.75 GPM, which was approaching the seal manufacturer's recommended low limit of 0.60 GPM. Following replacement of the seal, the plant was restarted and placed on line on June 22, 1989. The outage and associated activities were performed in an excellent manner.

All activities were conducted in a satisfactory manner and no violations or deviations were identified.

3. Monthly Surveillance Observation (61726)

The inspector reviewed/observed the following Technical Specification required surveillance testing:

<u>Surveillance Procedure</u>	<u>Test</u>
05B-105	Turbine Driven Auxiliary Feedwater Pump and Valve - IST
38-101	Station Battery Monthly Test
54-086	Turbine Stop and Governor Valve Op Test
14-026	Auxiliary Building Special Vent Monthly Test
42-109	Diesel Generator Manual Test
87-125	Shift Instrument Channel Checks

The following items were considered during the inspection: the testing was performed in accordance with adequate procedures, that test instrumentation was calibrated, that test results conformed with technical specifications and procedure requirements and were reviewed by personnel other than the individual directing the test, and that any deficiencies identified during the testing were reviewed and resolved by appropriate management personnel.

All activities were conducted in a satisfactory manner and no violations or deviations were identified.

4. Monthly Maintenance Observation (62703)

Station maintenance activities of safety related systems and components listed below were observed/reviewed to ascertain that they were conducted in accordance with approved procedures, regulatory guides, industry codes or standards, and in conformance with technical specifications.

The following items were considered during this review: the limiting conditions of operation were met while components or systems were removed from service; approvals were obtained prior to initiating the work; activities were accomplished using approved procedures and were inspected as applicable; functional testing and/or calibrations were performed prior to returning components or systems to service; parts and materials used were properly certified; radiological controls were implemented; and fire prevention controls were implemented.

The following maintenance activities were observed/reviewed:

<u>No.</u>	<u>Activity</u>
42939	Reset current limiter on battery chargers BRA-108 and BRB-108 from 172.5 to 155 amperes per Temporary Change Request No. 89-32
43598	Conducted ultrasonic examination of Service Water Pump 1A1 discharge check valve
44059	Replaced spring pack assembly in actuator for RHR-1B
45045	Reactor coolant system root stop valve RC-103B had excessive packing leakage to pressurizer relief tank. Tightened packing and backseated the valve.
45132	Replaced three-valve manifold for steam generator 1B outlet pressure transmitter

All activities were conducted in a satisfactory manner and no violations or deviations were identified.

5. Onsite Followup of Written Reports of Nonroutine Events (92700)

The inspector, through observations, discussions with licensee personnel, and review of records, reviewed the following event reports to determine that reportability requirements were satisfied; that corrective action was implemented; that the response to the event was adequate and met regulatory requirements, license conditions, and commitments, as applicable.

(CLOSED) LER 305/89011 Sealed Source not Tested Within Time Interval Specified by the Technical Specifications Due to a Procedural Weakness On May 23, 1989, with the plant at 100% power, a Radiation Technologist identified a radioactive source that had exceeded the prescribed frequency for leak testing as required by 10 CFR 35.59(b) and Technical Specifications 4.13.4. These documents require that a sealed source, in use, be leak tested every 6 months. On April 13, 1988, a Thallium-204 sealed source was received at the Kewaunee Nuclear Power Plant. The source had been successfully tested using a standard wipe test prior to shipment. However, following receipt, periodic leak tests were not performed. The root cause of this oversight was determined to be an inadequate review of the accompanying paperwork upon receipt. Apparently, an error had been made in the source identification when it was initially added to the inventory. The Thallium (TI) source had been mistakenly identified as Titanium (Ti). Immediately upon discovery, the TI-204 source was satisfactorily leak tested. Subsequent corrective actions included: (1) all the sealed sources were reviewed and verified as having been leak tested as necessary, (2) the applicable surveillance procedure was revised to include the TI-204 source in the leak testing inventory, (3) a preventive maintenance card requiring leak testing of