

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) DONALD C. COOK NUCLEAR PLANT, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 3 1 1 6	PAGE (3) 1 OF 0 2
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TITLE (4)
IMPROPERLY ISOLATED CARDOX FIRE PROTECTION SYSTEM

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 4	0 5	8 4	8 4	0 2 2	0 0 0	0 9	1 3	8 4			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)											
POWER LEVEL (10) 0 0 0	20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)		
	20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)		
	20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
	20.405(a)(1)(iii)			X 50.73(a)(2)(i)			50.73(a)(2)(viii)(A)					
	20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)					
20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)

NAME L. J. MATTHIAS - ADMINISTRATIVE SUPERVISOR	TELEPHONE NUMBER 6 1 6 4 6 5 5 9 0 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14) <input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) MONTH: DAY: YEAR:
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ON 4-5-84, AT 1235 HOURS, WITH UNIT 2 IN REFUELING MODE, SECURITY PERSONNEL DISCOVERED THAT THE CARDOX FIRE PROTECTION SYSTEM ON THE UNIT 2 REACTOR CABLE TUNNEL WAS ISOLATED WITH NO FIREWATCH PRESENT. THE DURATION THIS AREA WAS ISOLATED WITH NO FIREWATCH PRESENT WAS CALCULATED TO BE TWO HOURS AND FORTY (40) MINUTES. THIS IS IN VIOLATION OF TECHNICAL SPECIFICATION 3.7.9.3 LCO VALUE OF ONE HOUR.

THIS EVENT WAS ORIGINALLY CLASSIFIED AS NON-REPORTABLE. DURING THE PLANT NUCLEAR SAFETY REVIEW COMMITTEE CLOSEOUT REVIEW, ON AUGUST 27, 1984, OF THIS EVENT IT WAS DETERMINED THAT THE LCO HAD BEEN EXCEEDED.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) DONALD C. COOK NUCLEAR PLANT, UNIT 2	DOCKET NUMBER (2) 0500031684				LER NUMBER (6)			PAGE (3)		
	YEAR	SEQUENTIAL NUMBER		REVISION NUMBER						

TEXT (If more space is required, use additional NRC Form 365A's) (17)

ON 4-5-84 WHILE UNIT 2 WAS IN REFUELING MODE, THERE WAS WORK BEING PERFORMED IN THE REACTOR CABLE TUNNEL AND ADJACENT DIESEL GENERATOR ROOMS WITH FIREWATCHES INSIDE AND ONE SECURITY OFFICER WATCHING THE CO₂ SWITCHES (KP) WHICH WERE ISOLATED ON THE OUTSIDE OF THESE ROOMS. AT APPROXIMATELY THE SAME TIME THE WORK IN THE CABLE TUNNEL WAS COMPLETED, A RELIEF SECURITY OFFICER ASSUMED DUTIES AS THE SWITCH WATCH FOR THIS AREA. BY PLANT PROCEDURES, THE FIRE WATCH ASSIGNED THE CABLE TUNNEL IS TO RETURN THE CO₂ ISOLATE SWITCH TO THE NORMAL POSITION AND THE SECURITY OFFICER IS TO VERIFY THE SWITCH IS IN THE NORMAL POSITION.

AT APPROXIMATELY 1235 HRS. A SUBSEQUENT SECURITY OFFICER CHECKED THE ISOLATION SWITCH AND DISCOVERED IT WAS ISOLATED AND NO FIRE WATCH WAS INSIDE. DURING THE TIME THE CO₂ SWITCH WAS ISOLATED THE FIRE DETECTION SYSTEM WAS STILL OPERABLE. HOWEVER, THE SUPPRESSION SYSTEM WOULD HAVE REQUIRED MANUAL ACTIVATION.

IN CONCLUSION, THE FIREWATCH ERRED BY NOT RETURNING THE SWITCH TO THE NORMAL POSITION AND THE SECURITY OFFICER ERRED BY NOT CHECKING (INDEPENDENT VERIFICATION) THE SWITCH IN THE NORMAL POSITION. APPROPRIATE ADMINISTRATIVE MEASURES HAVE SINCE BEEN TAKEN TO ELIMINATE THIS TYPE ERROR GOING UNDETECTED.

A SIMILAR EVENT OCCURRED ON 4-10-84 REPORTED UNDER RO 84-0009-00.



INDIANA & MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106
(616) 465-5901

September 13, 1984

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

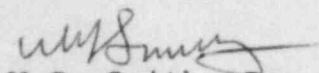
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Document Control Manager:

In accordance with the criteria established by 10CFR50.73
entitled Licensee Event Reporting System, the following
report/s are being submitted:

RO 84-022-0

Sincerely,


W.G. Smith, Jr.
Plant Manager

/cbm

Attachment

cc: John E. Dolan
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