



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

OMAHA PUBLIC POWER DISTRICT

DOCKET NO. 50-285

FORT CALHOUN STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83
License No. DPR-40

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Omaha Public Power District (the licensee) dated October 3, 1983 as supplemented by letter dated June 22, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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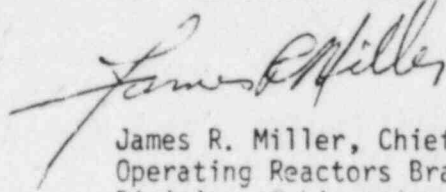
2. Accordingly, Facility Operating License No. DPR-40 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-40 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 83, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 7, 1984.

ATTACHMENT TO LICENSE AMENDMENT NO. 83

FACILITY OPERATING LICENSE NO. DPR-40

DOCKET NO. 50-285

Revise Appendix "A" Technical Specifications as indicated below. The revised page is identified by amendment number and contains vertical lines indicating the area of change.

Remove Page

3-27

2-98

Insert Page

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TABLE 2-9

Post-Accident Monitoring Instrumentation Operating Limits

<u>Instrument</u>	<u>Minimum Operable Channels</u>	<u>Action</u>
1. Containment Wide Range Radiation Monitors (RM-091A & B)	2	(a)
2. Wide Range Noble Gas Stack Monitor		
RM-063L (Noble Gas Portion Only)	1	(a)
RM-063M (Noble Gas Portion Only)	1	(a)
RM-063H (Noble Gas Portion Only)	1	(a)
3. Main Steam Line Radiation Monitor (RM-064)	1	(a)
4. Containment Hydrogen Monitor (VA-81A & B)	2	(b)(c)
5. Containment Water Level		
Narrow Range (LT-599 & LT-600)	1	(d)
Wide Range (LT-387 & LT-388)	2	(b)(c)
6. Containment Wide Range Pressure	2	(b)(c)

- (a) With the number of OPERABLE channels less than required by the minimum channels operable requirements, initiate the pre-planned alternate method of monitoring the appropriate parameter(s) within 72 hours, and
1. either restore the inoperable channel(s) to OPERABLE status within 7 days of the event, or
 2. prepare and submit a special report to the Commission pursuant to specification 5.9.3 within 14 days following the event outlining the action taken, the cause of the inoperability, and the plans and schedules for restoring the system to OPERABLE status.
- (b) With one channel inoperable, restore the inoperable monitor to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours.
- (c) With both channels inoperable, restore at least one channel to OPERABLE status within 72 hours or be in at least HOT SHUTDOWN within the next 12 hours.
- (d) With the number of OPERABLE channels less than required by the minimum channels operable requirements, operation may continue until the next cold shutdown, at which time the required channel(s) shall be made operable.

TABLE 3-6

REACTOR COOLANT PUMP SURVEILLANCE

<u>REQUIREMENT</u>	<u>METHOD</u>	<u>FREQUENCY</u>
1.1 Reactor Coolant Pump Flywheels	Visual inspection of upper surface of top disc and bottom surface of bottom disc; volumetric inspection from circumference of <u>all</u> disc segments.	When motor is disassembled for maintenance purposes.

TABLE 3-7

CAPSULE REMOVAL SCHEDULE

<u>REMOVAL SEQUENCE</u>	<u>REFUELING SCHEDULE EFPY**</u>	<u>CAPSULE REMOVED</u>
1	2.4	225°
2	5.9	265°
3	20	45°
4	21	85°
5	27	95°
6	28	265°*
7	32	225°*
8	Standby	275°

* Replacement capsule assemblies were installed in the 225° and 265° locations following early removal of the 265° capsule. These capsules benchmark the change in core loading design initiated at 5.9 EFPY.

** Based on a rated power level of 1500 Mwt.