



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

OMAHA PUBLIC POWER DISTRICT

DOCKET NO. 50-285

FORT CALHOUN STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 84
License No. DPR-40

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Omaha Public Power District (the licensee) dated June 8, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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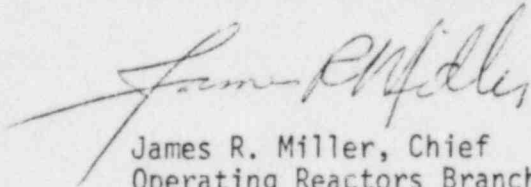
2. Accordingly, Facility Operating License No. DPR-40 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-40 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 84, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 7, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 84

FACILITY OPERATING LICENSE NO. DPR-40

DOCKET NO. 50-285

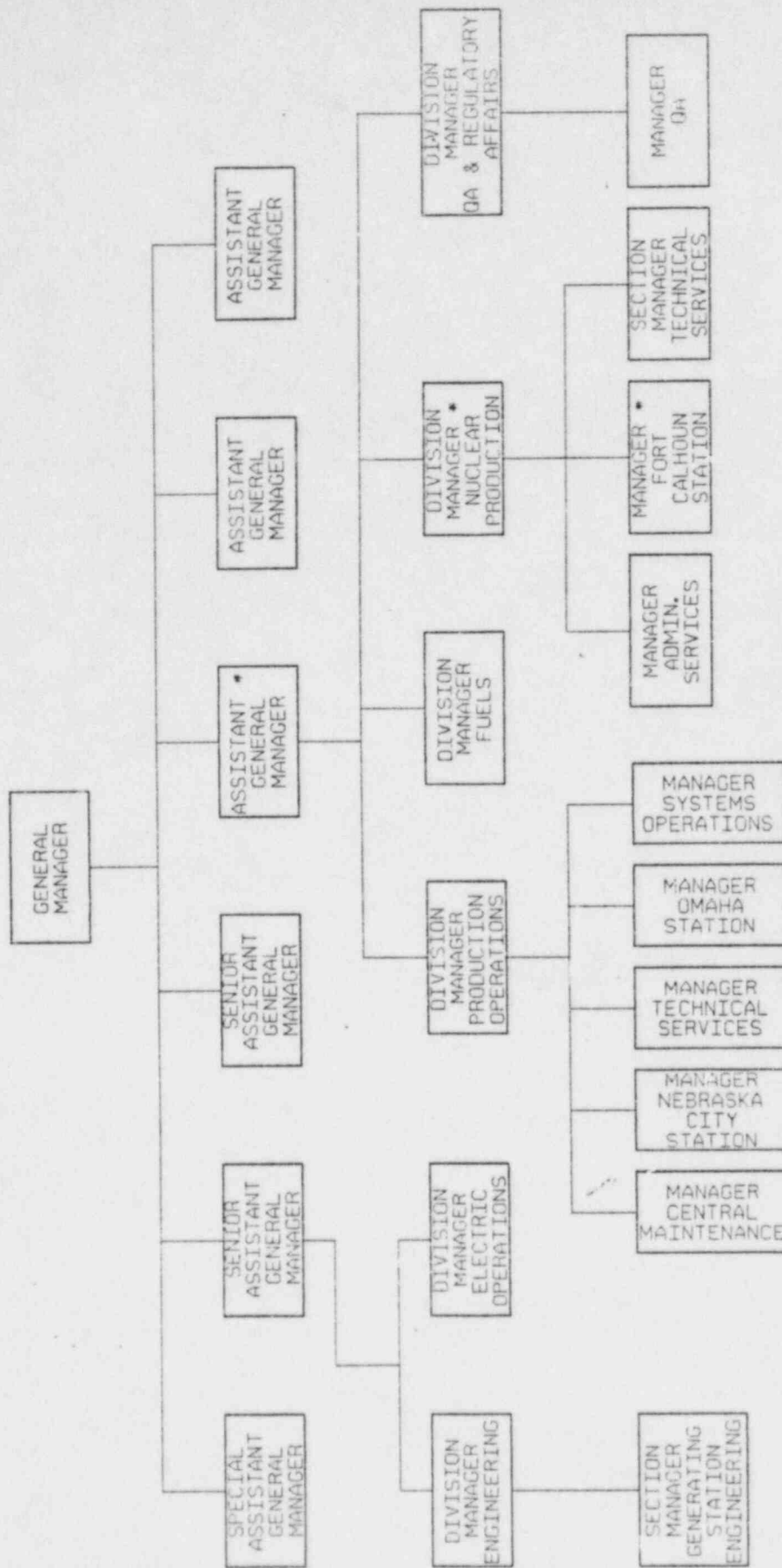
Revise Appendix "A" Technical Specifications as indicated below. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages

ii
Figure 5-1
Figure 5-2
5-3
5-4
5-5
5-8
5-9

Insert Pages

ii
Figure 5-1
Figure 5-2
5-3
5-4
5-5
5-8
5-9



* POSITIONS POSSESSING FIRE PROTECTION RESPONSIBILITIES.

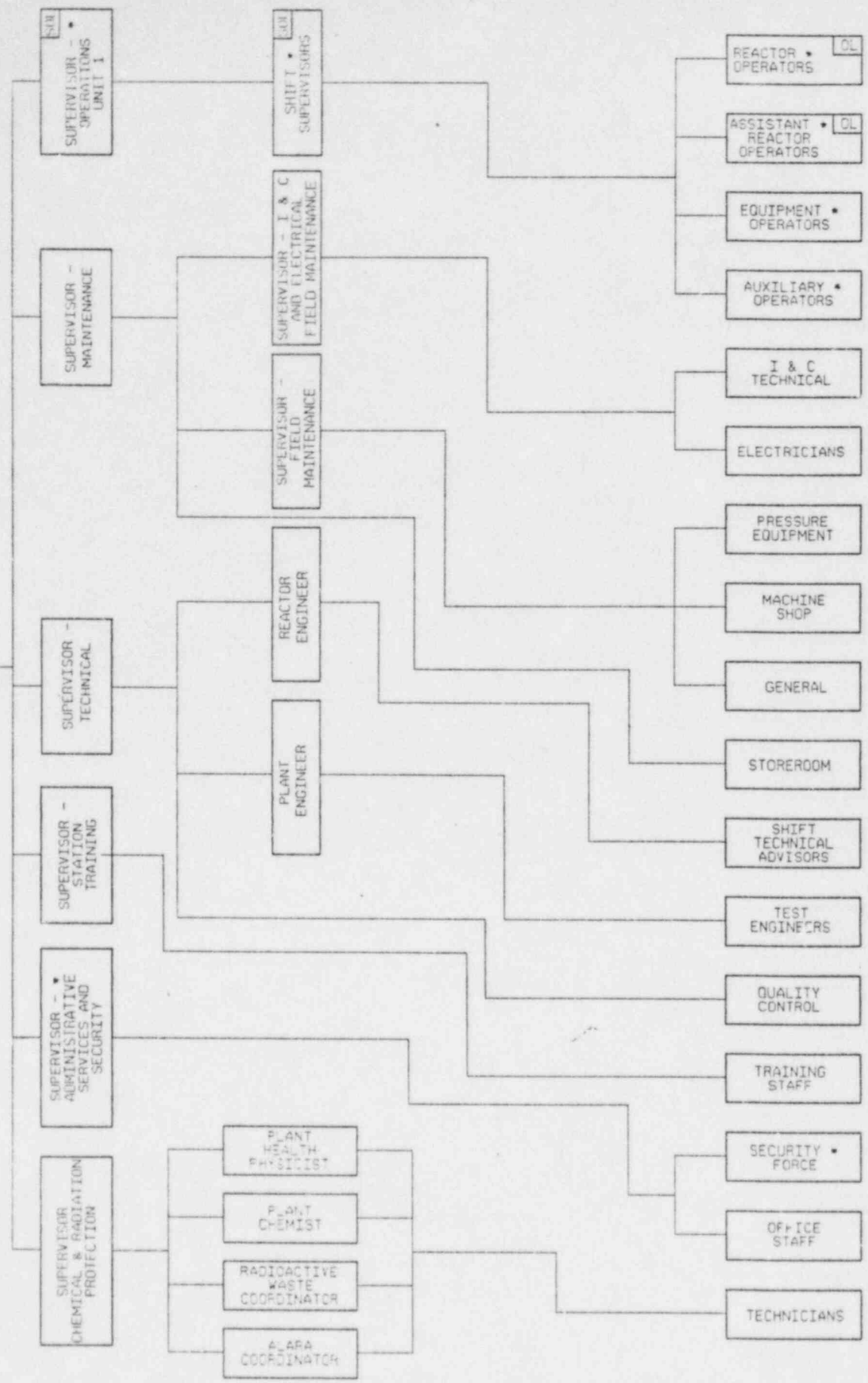
OMAHA PUBLIC POWER DISTRICT
 FORT CALHOUN STATION
 UNIT No. 1
 Amendment No. 9, 10, 25, 60, 76, 84

OPPD SUPPORT STAFF

FIGURE 5-1

CLASSIFICATION
NRC LICENSE

MANAGER -
FORT CALHOUN
STATION



* POSITIONS POSSESSING FIRE PROTECTION RESPONSIBILITIES.

FORT CALHOUN
TECHNICAL
SPECIFICATIONS
Amendment No. 9, 19, 28, 60, 78, 84

FORT CALHOUN STATION
ORGANIZATION CHART

FIGURE
5-2

TABLE OF CONTENTS (continued)

	<u>Page</u>
3.0 SURVEILLANCE REQUIREMENTS.....	3-1
3.1 Instrumentation and Control.....	3-1
3.2 Equipment and Sampling Tests.....	3-17
3.3 Reactor Coolant System, Steam Generator Tubes, and Other Components Subject to ASME XI Boiler & Pressure Vessel Code Inspection and Testing Surveillance.....	3-21
3.4 Reactor Coolant System Integrity Testing.....	3-36
3.5 Containment Test.....	3-37
3.6 Safety Injection and Containment Cooling Systems Tests.....	3-54
3.7 Emergency Power System Periodic Tests.....	3-58
3.8 Main Steam Isolation Valves.....	3-61
3.9 Auxiliary Feedwater System.....	3-62
3.10 Reactor Core Parameters.....	3-63
3.11 Environmental Radiological Monitoring.....	3-64
3.12 Radioactive Materials.....	3-69
3.13 Radioactive Material Sources Surveillance.....	3-76
3.14 Shock Suppressors (Snubbers).....	3-77
3.15 Fire Protection System.....	3-80
4.0 DESIGN FEATURES.....	4-1
4.1 Site.....	4-1
4.2 Containment Design Features.....	4-1
4.2.1 Containment Structure.....	4-1
4.2.2 Penetrations.....	4-1
4.2.3 Containment Structure Cooling Systems.....	4-2
4.3 Nuclear Steam Supply System (NSSS).....	4-3
4.3.1 Reactor Coolant System.....	4-3
4.3.2 Reactor Core and Control.....	4-3
4.3.3 Emergency Core Cooling.....	4-3
4.4 Fuel Storage.....	4-4
4.4.1 New Fuel Storage.....	4-4
4.4.2 Spent Fuel Storage.....	4-4
4.5 Seismic Design for Class I Systems.....	4-5
5.0 ADMINISTRATIVE CONTROLS.....	5-1
5.1 Responsibility.....	5-1
5.2 Organization.....	5-1
5.3 Facility Staff Qualifications.....	5-1a
5.4 Training.....	5-3
5.5 Review and Audit.....	5-3
5.5.1 Plant Review Committee (PRC).....	5-3
5.5.2 Safety Audit and Review Committee (SARC).....	5-5
5.5.3 Fire Protection Inspection.....	5-8
5.6 Reportable Occurrence Action.....	5-9
5.7 Safety Limit Violation.....	5-9
5.8 Procedures.....	5-9

5.0 ADMINISTRATIVE CONTROLS

5.4 Training

- 5.4.1 A retraining and replacement training program for the plant staff shall be maintained under the direction of the Supervisor - Station Training and shall meet or exceed the requirements of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55.
- 5.4.2 A training program for the fire brigade shall be maintained under the Manager - Fort Calhoun Station and shall meet or exceed the requirements of Section 27 of NFPA Code-1975, except that the meeting frequency may be quarterly.

5.5. Review and Audit

5.5.1 Plant Review Committee (PRC)

Function

- 5.5.1.1 The Plant Review Committee shall function to advise the Manager - Fort Calhoun Station on all matters related to nuclear safety.

Composition

- 5.5.1.2 The official Plant Review Committee shall be composed of the:

Chairman: Manager - Fort Calhoun Station
Member: Supervisor - Operations
Member: Supervisor - Technical
Member: Supervisor - Maintenance
Member: Supervisor - Instrument & Control and Electrical
Member: Reactor Engineer
Member: Supervisor - Chemistry and Radiation Protection
Member: Plant Engineer

Alternates

- 5.5.1.3 Alternate members shall be appointed in writing by the Plant Review Committee Chairman to serve on a temporary basis; however, no more than two alternates shall participate in Plant Review Committee activities at any one time.

Meeting Frequency

- 5.5.1.4 The Plant Review Committee shall meet at least once per calendar month and as convened by the Plant Review Committee Chairman.

Quorum

- 5.5.1.5 A quorum of the Plant Review Committee shall consist of the Chairman and four members including alternates.

5.0 ADMINISTRATIVE CONTROLS

Responsibilities

5.5.1.6 The Plant Review Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 5.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Manager - Fort Calhoun Station to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and shall prepare and forward a report covering evaluation and recommendations to prevent recurrence to the Division Manager - Nuclear Production and to the Chairman of the Safety Audit and Review Committee.
- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Safety Audit and Review Committee.
- h. Review of the Site Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Safety Audit and Review Committee.
- i. Review of the Site Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Safety Audit and Review Committee.

Authority

5.5.1.7 The Plant Review Committee shall:

- a. Recommend in writing to the Manager - Fort Calhoun Station approval or disapproval of items considered under 5.5.1.6(a) through (d) above.

5.0 ADMINISTRATIVE CONTROLS

- 5.5.1.7 b. Render determinations in writing with regard to whether or not each item considered under 5.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide immediate written notification to the Division Manager - Nuclear Production and the Safety Audit and Review Committee of disagreement between the Plant Review Committee and the Manager - Fort Calhoun Station; however, the Manager - Fort Calhoun Station shall have responsibility for resolution of such disagreements pursuant to 5.1.1 above.

Records

- 5.5.1.8 The Plant Review Committee shall maintain written minutes of each meeting and copies shall be provided to the Division Manager - Nuclear Production and Chairman of the Safety Audit and Review Committee.

5.5.2 Safety Audit and Review Committee (SARC)

Function

- 5.5.2.1 The Safety Audit and Review Committee shall function to provide the independent review and audit of designated activities in the areas of:
- a. nuclear power plant operation
 - b. nuclear engineering
 - c. chemistry and radiochemistry
 - d. metallurgy
 - e. instrumentation and control
 - f. radiological safety
 - g. mechanical and electrical engineering
 - h. quality assurance

Composition

- 5.5.2.2 The Safety Audit and Review Committee shall be composed of:

Chairman: Division Manager - Quality Assurance and Regulatory Affairs
Member: Assistant General Manager - Nuclear Production, Production Operations, Fuels, and QA&RA
Member: Assistant General Manager - Electric Operations and Engineering
Member: Division Manager - Engineering
Member: Division Manager - Nuclear Production
Member: OPPD Operations, Engineering, and Technical Support staff
Member: Qualified Non-District Affiliated Consultants as Required and as Determined by SARC Chairman

5.0 ADMINISTRATIVE CONTROLS

- 5.5.2.8 e. The Fort Calhoun Station Emergency Plan and implementing procedures at least once every twelve months.
- f. The Site Security Plan and implementing procedures at least once every twelve months.
- g. The Safeguards Contingency Plan and implementing procedures at least once every twelve months.
- h. Any other area of facility operation considered appropriate by the Safety Audit and Review Committee or the Assistant General Manager - Nuclear Production, Production Operations, Fuels, and Quality Assurance & Regulatory Affairs.

Authority

- 5.5.2.9 The Safety Audit and Review Committee shall report to and advise the Assistant General Manager - Nuclear Production, Production Operations, Fuels, and Quality Assurance & Regulatory Affairs on those areas of responsibility specified in Sections 5.5.2.7 and 5.5.2.8.

Records

- 5.5.2.10 Records of Safety Audit and Review Committee activities shall be prepared, approved and distributed as indicated below:
- a. Minutes of each Safety Audit and Review Committee meeting shall be prepared, approved and forwarded to the Assistant General Manager - Nuclear Production, Production Operations, Fuels, and Quality Assurance & Regulatory Affairs within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 5.5.2.7e, f, g, and h above shall be prepared, approved and forwarded to the Assistant General Manager - Nuclear Production, Production Operations, Fuels, and Quality Assurance & Regulatory Affairs within 14 days following completion of the review.
- c. Audit reports encompassed by Section 5.5.2.8 above shall be forwarded to the Assistant General Manager - Nuclear Production, Production Operations, Fuels, and Quality Assurance & Regulatory Affairs and to the responsible management positions designated by the Safety Audit and Review Committee within 30 days after completion of the audit.

5.5.3 Fire Protection Inspection

- a. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified off-site licensee personnel or an outside fire protection firm. The audit and inspection program responsibility shall rest with the Safety Audit and Review Committee.
- b. An inspection and audit of the fire protection and loss prevention program by an outside qualified fire consultant shall be performed at intervals no greater than 3 years.

5.6 Reportable Occurrence Action

5.6.1 The following actions shall be taken in the event of a REPORTABLE OCCURRENCE:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 5.9.
- b. Each Reportable Occurrence requiring 24 hour notification to the Commission shall be reviewed by the Plant Review Committee and submitted to the Safety Audit and Review Committee and the Division Manager - Nuclear Production.

5.7 Safety Limit Violation

5.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The safety limit violation shall be reported to the Commission, the Division Manager - Nuclear Production and to the Safety Audit and Review Committee immediately.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the Plant Review Committee. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the Safety Audit and Review Committee and the Division Manager - Nuclear Production within 10 days of the violation.

5.8 Procedures

5.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the minimum requirements of sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix A of USNRC Regulatory Guide 1.33 except as provided in 5.8.2 and 5.8.3 below.

5.8.2 Each procedure and administrative policy of 5.8.1 above, and changes thereto, shall be reviewed by the Plant Review Committee and approved by the Manager - Fort Calhoun Station prior to implementation and periodically as set forth in each document.

5.8.3 Temporary changes to procedures of 5.8.1 above may be made provided: