



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 20

TO FACILITY LICENSE NO. R-95

DOCKET NO. 50-193

RHODE ISLAND ATOMIC ENERGY COMMISSION

1.0 INTRODUCTION

By letter of August 24, 1994, the Rhode Island Atomic Energy Commission (the licensee), requested a change to Technical Specification 3.7.1 to allow a time period of one shift (6 hours) for replacement of the particulate activity or the gaseous activity monitors when either is not operating prior to shut down of the research reactor. This is a change from the current Technical Specification in that currently no time period is allowed. Therefore, the change is to provide for an adequate time to evaluate and take action on failure of the particulate activity radiation monitor or the gaseous activity radiation monitor.

2.0 EVALUATION

The particulate activity and gaseous activity monitors provide an alarm function to inform operations personnel of potential radiological releases from the facility stack. The proposed time limit for allowing replacement or repair of these radiation monitors prior to shut down of the facility is consistent with reactor licensing practices and should not have an adverse effect on reactor safety since there are alternate monitors that are required to alert personnel to potential radiological release conditions.

Specifically, the other monitors include a reactor bridge radiation monitor, a continuous air monitor, and several experimental and process radiation monitors which also provide alarm functions. Technical Specification 3.2.1, Table 3.2 requires at least one monitor over the reactor pool and one monitor on the experimental level of the reactor building which ensures that potential radiological releases would be detected and alarmed. Further, the provisions to allow some time for assessment and action would provide reactor personnel with time to verify conditions, systems and functions prior to shut down.

Based on the above, potential radiological release conditions will be acceptably monitored and alarmed as required by Technical Specifications. Therefore, the NRC staff finds the proposed change to Technical Specification 3.7.1 acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes in inspection and surveillance requirements. The staff has determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and no significant increase in individual or cumulative occupational radiation exposure. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously evaluated, or create the possibility of a new or different kind of accident from any accident previously evaluated, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by the proposed changes, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public.

Principal Contributors: Marvin M. Mendonca

Date: September 22, 1995