

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Peach Bottom Atomic Power Station - Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 7 8	PAGE (3) 1 OF 0 3
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TITLE (4)  
Actuation of Unit 3 PCIS, RPS, and ECCS Systems

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME		
0 8	2 1	8 4	8 4	0 1 1	0 0	0 9	2 0	8 4	DOCKET NUMBER(S) 0 5 0 0 0		

OPERATING MODE (9)  N

POWER LEVEL (10) 1 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(e)	<input checked="" type="checkbox"/> 60.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 60.36(a)(1)	<input type="checkbox"/> 60.73(a)(2)(v)	<input type="checkbox"/> 73.71(a)
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 60.36(a)(2)	<input type="checkbox"/> 60.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366-A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 60.73(a)(2)(ii)	<input type="checkbox"/> 60.73(a)(2)(vii)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 60.73(a)(2)(iv)	<input type="checkbox"/> 60.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 60.73(a)(2)(iii)	<input type="checkbox"/> 60.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME B. L. Clark, Senior Engineer - Special Projects	TELEPHONE NUMBER 2 1 5 8 4 1 1 - 5 0 1 7
AREA CODE	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	J, K	LC		N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-spaced typewritten lines) (16)

Abstract: 3-84-11

On August 21, 1984, with Unit 3 at 100% power, a sudden runback of all three reactor feedpumps resulted in a reactor low level scram, Groups I, II, and III isolations, and automatic initiation of the High Pressure Coolant Injection (HPCI), and Reactor Core Isolation Cooling (RCIC) systems. Normal reactor level was immediately restored. The Group I, II, and III isolations were reset and essential systems were promptly returned to service. The cause of the feedpump runback is believed to be intermittent failure of a component in the feedwater control system.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	0 1 1	0 0	0 2	OF	0 3

TEXT (if more space is required, use additional NRC Form 366A's) (17)

Description of the Event:

On August 21, 1984, at 2:01 p.m., with Unit 3 at 100% power, the "Reactor Hi/Lo Water Level" annunciator alarmed in the Control Room. The reactor operator observed that all three reactor feedpump flows and reactor level were rapidly decreasing. Within seconds a reactor low level scram, a Group II isolation, and a Group III isolation occurred. As reactor level continued to decrease, both reactor recirculation pumps tripped, the High Pressure Coolant Injection (HPCI) and Reactor Core Isolation Cooling (RCIC) systems automatically started, and a Group I isolation occurred. HPCI and RCIC continued to inject water into the reactor until both of these systems tripped on high reactor level. The HPCI system cycled off and on to maintain reactor level. Torus cooling was established, the isolations were reset, drywell instrument nitrogen was restored, and the main steam isolation valves were opened to re-establish the main condenser as the primary heat sink. The 'C' reactor feedpump was reset from the high reactor level trip and placed in service for reactor level control.

Consequences of the Event:

The reactor protection and the primary containment isolation systems operated properly during the reactor water level transient. The HPCI and RCIC systems started automatically and immediately restored reactor level to normal. All other safety systems responded properly. Therefore, there were no adverse consequences.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Cause of the Event:

The decrease in reactor water level was the result of a sudden speed decrease of all three reactor feedpumps. The cause of the speed decrease is believed to be intermittent failure of a component in the feedwater control system.

Corrective Actions:

The feedwater master controller and three proportional amplifiers in the feedwater control system were calibration checked and found to be fully operational. A visual inspection of the wires connecting the master controller, the proportional amplifiers, and the individual feedpump turbine speed controllers indicated that all connections were intact. The ribbon cables connecting the master controller, proportional amplifiers, and speed controllers were inspected, continuity checked, and found acceptable. The feedwater runback instrumentation setpoints were checked and found acceptable.

Because testing could not reproduce the failure, components capable of producing a similar failure were replaced. The replaced components included the master controller, three proportional amplifiers, and four ribbon cables. In addition, a recorder was installed to monitor signals to the master controller.

Upon return to power operation, the 'B' reactor feedpump was placed in manual control and the automatic control of the 'A' and 'C' reactor feedpumps was observed for proper control ability. No feedwater control deficiencies have been identified since the unit was returned to power operation.

PHILADELPHIA ELECTRIC COMPANY

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September 20, 1984

Docket No. 50-278

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Washington, DC 20555

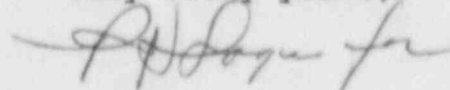
SUBJECT: Licensee Event Report

This LER deals with the full scram and actuation of the Primary Containment Isolation System, High Pressure Coolant Injection System, and Reactor Core Isolation Cooling System on Unit 3.

Reference:	Docket No. 50-278
Report Number:	3-84-11
Revision Number:	00
Event Date:	August 21, 1984
Report Date:	September 20, 1984
Facility:	Peach Bottom Atomic Power Station RD #1, Box 208, Delta, PA 17314

This LER is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(IV).

Very truly yours,



W. T. Ullrich  
Superintendent  
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator  
Region I, USNRC

Mr. A. R. Blough, Site Inspector

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