

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Oyster Creek, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 1 1 9	PAGE (3) 1 OF 0 3
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TITLE (4)
SGTS Low Flow

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																																																															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																																																																													
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<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td colspan="12">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)</td> </tr> <tr> <td colspan="3">OPERATING MODE (9) N</td> <td colspan="3">20.402(b)</td> <td colspan="3">20.405(c)</td> <td colspan="3">50.73(a)(2)(iv)</td> <td colspan="3">73.71(b)</td> </tr> <tr> <td colspan="3">POWER LEVEL (10) 0 1 0 1 0</td> <td colspan="3">20.405(a)(1)(i)</td> <td colspan="3">50.36(e)(1)</td> <td colspan="3">50.73(a)(2)(v)</td> <td colspan="3">73.71(e)</td> </tr> <tr> <td colspan="3"></td> <td colspan="3">20.405(a)(1)(ii)</td> <td colspan="3">50.36(e)(2)</td> <td colspan="3">50.73(a)(2)(vii)</td> <td colspan="3" rowspan="4">OTHER (Specify in Abstract below and in Text, NRC Form 365A)</td> </tr> <tr> <td colspan="3"></td> <td colspan="3">20.405(a)(1)(iii)</td> <td colspan="3">50.73(a)(2)(i)</td> <td colspan="3">50.73(a)(2)(viii)(A)</td> </tr> <tr> <td colspan="3"></td> <td colspan="3">20.405(a)(1)(iv)</td> <td colspan="3">50.73(a)(2)(ii)</td> <td colspan="3">50.73(a)(2)(viii)(B)</td> </tr> <tr> <td colspan="3"></td> <td colspan="3">20.405(a)(1)(v)</td> <td colspan="3">50.73(a)(2)(iii)</td> <td colspan="3">50.73(a)(2)(ix)</td> </tr> </table>												THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)												OPERATING MODE (9) N			20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)			POWER LEVEL (10) 0 1 0 1 0			20.405(a)(1)(i)			50.36(e)(1)			50.73(a)(2)(v)			73.71(e)						20.405(a)(1)(ii)			50.36(e)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 365A)						20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)						20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)						20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)		
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LICENSEE CONTACT FOR THIS LER (12)

NAME Michael Fitzgerald, Operations Engineer	TELEPHONE NUMBER 6 1 0 9 9 7 1 1 - 1 4 8 9 1 6
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On August 9, 1984, the flow rate through Standby Gas Treatment System (SGTS) No. 2 was found to be below the limit given in section 4.5.K.1.b.3 of the Technical Specifications. The flow rate was determined using a 16-point traverse method as opposed to installed instrumentation. The Technical Specification range of acceptable flow rates is 2340-2860 CFM; flow rate during the test was found to be 2334 CFM. The manual damper in the system was adjusted, the system was retested, and the resultant flow rate was 2687 CFM.

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TEXT (If more space is required, use additional NRC Form 368A's) (17)

DATE OF OCCURRENCE

The event occurred on August 9, 1984.

IDENTIFICATION OF OCCURRENCE

During testing SGTS No. 2, the flow rate was found to be below the Technical Specification limit as given in section 4.5.K.1.b.3.

This event is considered to be a reportable occurrence per 10CFR50.73(a)(2)(i).

CONDITIONS PRIOR TO OCCURRENCE

The reactor mode switch was in the Shutdown position and reactor coolant was less than 212°F.

DESCRIPTION OF OCCURRENCE

In order to fully describe this event, events which occurred in March 1984 need explanation to provide a basis. A Secondary Containment Leak Rate Test was initiated on March 18, 1984, prior to refueling operations, and a SGTS 10-hour run was performed concurrently. During secondary containment leak rate testing it was noted that SGTS No. 2 flow was excessive. Secondary containment capability was demonstrated satisfactorily with SGTS No. 1. The pitot tube was found to have a cracked joint, due to the installation of a new annubar to provide digital flow indication. The annubar was removed and the pitot tube was repaired. The Secondary Containment Leak Rate Test and SGTS 10-hour run were performed again on March 19, 1984. Also at this time, the pitot tube was checked against the flow readings from the 16-point traverse and verified to be reading correctly. After completion of the Secondary Containment Leak Rate Test, the flow was adjusted down to 2550 CFM on SGTS No. 2, because it had been above the 2860 CFM Technical Specification limit during the secondary containment test.

During the next several months, SGTS No. 2 was run for 10 hours and the flow rate was measured 7 (seven) times. The flow rate during each test was between 2500-2600 CFM, which is expected since the damper was positioned to give 2550 CFM. On August 8, 1984, a 10-hour run was scheduled to be

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TEXT (If more space is required, use additional NRC Form 306A's) (17)

performed. However, the pitot tube had been removed to install new annubar flow measuring instrumentation. A flow of 2334 CFM was measured, using a 16-point traverse of the duct, which is 6 CFM below the Technical Specification limit and approximately 150-200 CFM below what the pitot tube readings had been.

APPARENT CAUSE OF OCCURRENCE

The cause of this event is indeterminable. It could not be determined if the damper position had been changed between July 8 and August 9, 1984. However, the damper appeared to be in the same position on August 9, 1984 as it was on March 19, 1984 as a result of a visual inspection, but a slight change in damper position could have caused the change in flow rate. Although the pitot tube readings and the 16-point traverse readings were correlated on March 19, 1984, a similar comparison could not be performed for the 16-point traverse readings during the August 9, 1984 test.

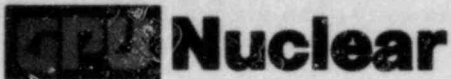
ANALYSIS OF OCCURRENCE and SAFETY SIGNIFICANCE

The design flow rate of the SGTS is 2600 CFM, and the Technical Specification requirement is to demonstrate 2600 CFM +/- 10 percent at a frequency of once per 18 months. The SGTS is also required to operate at less than 4000 CFM and maintain the Reactor Building at a minimum of 0.25 inches water column vacuum.

The safety significance of the event is minimal. The reading obtained was within 6 CFM (0.26 percent) of the Technical Specification limit. Also, there is difficulty associated with the performance of a 16-point traverse method for flow determination. The pitot tube must be held perpendicular to the duct so the end of the tube is parallel to the flow path. If it is not, the readings will be slightly lower than the actual values (conservative direction). The installed pitot tube had already been mechanically positioned to the proper orientation.

CORRECTIVE ACTION

The immediate corrective action was to open the damper further to meet the design flow limit. Flow was increased to 2698 CFM by slightly opening the damper. Upon further analysis, the damper will be full open during the acceptance testing of the new flow instrumentation to determine the flow rate obtainable with a full-open damper. Also, additional controls will be implemented to insure the position of the damper remains fixed once baseline testing has determined the proper damper position.



GPU Nuclear Corporation
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Forked River, New Jersey 08731-0388
609 971-4000
Writer's Direct Dial Number:

September 10, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report

This letter forwards one (1) copy of Licensee Event Report (LER)
No. 84-018.

Very truly yours,

Peter B. Fiedler
Vice President and Director
Oyster Creek

PBF:dsm
Enclosures

cc: Dr. Thomas E. Murley, Administrator
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

NRC Resident Inspector
Oyster Creek Nuclear Generating Station
Forked River, NJ 08731