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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
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EDWIN I. HATCH, UNIT 2	0  5  0  0  0  3  6	6 8 4 - 0 1 6 - 0 0 0 2 OF 0 2

This 30 day LER is required by 10CFR 50.73(a)(2)(iv) because an event occurred that resulted in the unplanned, automatic actuation of an engineered safety feature during a hydrostatic test of instrument tubing welds.

Notification by this LER also fulfills the special report requirements of Unit 2 Tech. Specs. section 3.5.3.1, ACTION 3. The total core spray injections into the vessel to date has been 1.

On 8/15/84 (with the Unit in a recirculation pipe outage) at approximately 1830 CST, a spurious loss of coolant accident (LOCA) signal initiated the following sequence of events: 4 RHR pumps started, 2 core spray pumps started and injected into the reactor vessel, three diesel generators started, and HPCI and RCIC initiated.

After an investigation of the cause of the spurious LOCA signal, it was learned that the spurious signal was transmitted by one of two reactor water level pressure transmitters (2B21-N091B or 2B21-N091D), which are a part of the newly installed analog transmitter trip system (ATTS).

In preparation for a hydrostatic pressure test of instrument tubing welds, a technician was performing a valve alignment on instrument panel 2H21-P405A (which houses 2B21-N091B & D) per the "PRESSURE TESTING OF PIPING AND COMPONENTS" procedure (HNP-6907). The procedure (HNP-6907) did not require the technician to open the transmitters' pressure equalizing valves prior to valving in the hydrostatic pressure. Consequently, when the hydrostatic pressure was valved into one side of the pressure transmitter, the other side of the transmitter sensed static atmospheric pressure which caused the transmitter to sense a momentary LOCA condition and transmit a LOCA signal.

The cause of this event is personnel error in that the individual who completed the procedure data package of HNP-6907 did not specify that the pressure transmitters' equalizing valves should be opened prior to pressurizing the system.

After the investigation it was determined that no procedure revision was required since the procedure data package was for a one time use, and when the mistake was discovered the weld examination was finished.

RC Form 368A

Georgia Power Company Post Office Box 439 Baxley, Georgia 31513 Telephone 912 367-7781 912 537-9444

Edwin I. Hatch Nuclear Plant

Georgia Power

IELL

September 10, 1984 GM-84-755

<u>PLANT E. I. HATCH</u> Licensee Event Report Docket No. 50-366

United States Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1984-016. This report is required by 10CFR 50.73(a)(2)(iv).

Away H. C. Nix

General Manager

ACN/TLE/VIT

xc: R. J. Kelly R. E. Conway J. T. Beckham, Jr. P. D. Rice K. M. Gillespie Superintendent of Regulatory Compliance R. D. Baker Control Room Document Control