

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Callaway Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 8 1 3	PAGE (3) 1 OF 0 2
---	---	-----------------------------

TITLE (4)
Inadvertent Loss of Power to Radiation Monitors

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		
0 8	1 3	8 4	8 4	0 2	7 0	0 9	1 1	8 4			
									DOCKET NUMBER(S) 0 5 0 0 0 0		

OPERATING MODE (9) 4	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0	20.402(b)	20.405(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)					
	20.405(a)(1)(ii)	50.36(c)(1)		50.73(a)(2)(v)	73.71(c)					
	20.405(a)(1)(iii)	50.36(c)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)					
	20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)						
	20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)						
20.405(a)(1)(iv)	50.73(a)(2)(iii)		50.73(a)(2)(ix)							

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME Andrew P. Neuhalfen - Assistant Manager Operations and Maintenance		AREA CODE 3 1 4	6 7 6 - 8 2 0 2

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDPS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDPS

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

This report involves two incidents of personnel inadvertently removing power from radiation monitors resulting in Engineered Safety Feature (ESF) actuations.

The first incident occurred on 8/13/84 at approximately 1640 CDT. Instrumentation and Control (I&C) technicians were in the process of performing a functional check on radiation monitor GT-RT-31. This requires that GT-RT-31 be bypassed at the ESF cabinets prior to removing power from the radiation monitor skid. Located parallel to GT-RT-31 is radiation monitor GT-RT-33. The I&C technicians had placed the channel for GT-RT-31 in bypass as required, however, they inadvertently removed power from GT-RT-33 instead. This resulted in a Control Room Ventilation Isolation Signal (CRVIS) and a Containment Purge Isolation Signal (CPIS).

The second incident occurred on 8/17/84 at approximately 1525 CDT. Maintenance support personnel were upgrading wiring in the 480 VAC NG01B bus. While the personnel were reinstalling the panel cover to NG01B they inadvertently hit and opened the breaker providing power to radiation monitor GG-RT-27. This loss of power to GG-RT-27 resulted in a Fuel Building Isolation Signal (FBIS) and a CRVIS.

No radiation above normal background was present and this event has in no way affected the health and safety of the public.

8409240001 840911
PDR ADOCK 05000483
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Callaway Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 8 3 8 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		0	2	7	0	0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 8/13/84 and 8/17/84 the plant experienced inadvertent Engineered Safety Feature (ESF) actuations, due to loss of power to radiation monitors GT-RT-33 and GG-RT-27 respectively. The plant was in Mode 4, Hot Shutdown, 0% power at the time of the incidents. These inadvertent ESF actuations are classified as reportable under 10 CFR 50.73(a)(2)(iv).

The first incident occurred on 8/13/84 at approximately 1640 CDT. Instrumentation and Control (I&C) technicians were in the process of performing a functional check on radiation monitor GT-RT-31. This requires that GT-RT-31 be bypassed at the ESF cabinets prior to removing power from the radiation monitor skid. Located parallel to GT-RT-31, on the 2047' level of the Auxiliary Building, is radiation monitor GT-RT-33. The I&C technicians had placed the channel for GT-RT-31 in the bypass mode, as required to perform the functional check. However, when they were ready to remove power from GT-RT-31 they inadvertently removed power from GT-RT-33 instead. This resulted in a Control Room Ventilation Isolation Signal (CRVIS) and a Containment Purge Isolation Signal (CPIS). Power was returned to GT-RT-33 and the CRVIS and CPIS were restored, per plant operating procedures.

The second incident occurred on 8/17/84 at approximately 1525 CDT. Maintenance support personnel were upgrading wiring in the 480 VAC bus NBO1B. When the personnel were reinstalling the panel cover to NBO1B they inadvertently hit and opened the breaker providing power to radiation monitor GG-RT-27. This loss of power to GG-RT-27 resulted in a Fuel Building Isolation Signal (FBIS) and a CRVIS. By 1640 CDT the power to GG-RT-27 was restored and the FBIS and CRVIS were reset per plant operating procedures.

I&C and Maintenance supervisors have reviewed these incidents with departmental personnel. It was stressed that personnel must use the utmost of caution when performing tests or maintenance to avoid inadvertently de-energizing or upsetting plant equipment. In addition, the Plant Manager and/or Assistant Plant Managers have also held a series of meetings with plant staff and support personnel. The purpose of these meetings was to enhance the communication of management's philosophy to all levels of the work force, highlight the recent Callaway events, and stress the necessity for improving individual performance, group performance, and communication. These meetings were completed 9/7/84.

No radiation above normal background was present and this event has in no way affected the health or safety of the public.

Previous occurrences: LER 84-019-00

UNION ELECTRIC COMPANY
CALLAWAY PLANT

MAILING ADDRESS:
P. O. BOX 620
FULTON, MO. 65251

September 11, 1984

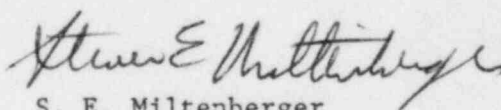
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ULNRC-923

DOCKET NUMBER 50-483
CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-25
LICENSEE EVENT REPORT 84-027-00
INADVERTENT ENGINEERED SAFETY FEATURES ACTUATION

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(iv) concerning inadvertent Engineered Safety Features actuations.


S. E. Miltenberger
Manager, Callaway Plant

APN/CDN/WRR/RCW/drs
Enclosure

cc: Distribution attached

IE22
11

cc distribution for ULNRC-923

James G. Keppler
USNRC Region III Office
799 Roosevelt Road
Glen Ellyn, IL 60137

American Nuclear Insurers
c/o Dottie Sherman, Library
The Exchange Suite 245
270 Farmington Aveune
Farmington, CT 06032

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

NRC Resident Inspector
Missouri Public Service Commission
D. F. Schnell
J. F. McLaughlin
J. E. Davis (Z40LER)
D. W. Capone
F. D. Field
R. L. Powers
A. C. Passwater/D. E. Shafer/D. J. Walker
G. A. Hughes
W. R. Robinson (QA Record)
C. D. Naslund
R. H. Leuther
A. P. Neuhalfen
R. A. McAleenan
L. K. Robertson (470)(NSRB)
Merlin Williams, Wolf Creek
SEM Chrono
3456-0021.6
3456-0260
Z40ULNRC
G56.37
N. Date