



William J. Cahill, Jr.  
Chief Nuclear Officer

January 25, 1996  
JPN-96-003

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Mail Station P1-137  
Washington, DC 20555

Subject: James A. FitzPatrick Nuclear Power Plant  
Docket No. 50-333  
**Proposed Changes to the Technical Specifications Regarding  
Extension of Instrumentation and Miscellaneous Surveillance Test  
Intervals to Accommodate 24-Month Operating Cycles (JPTS-95-001G)**

Reference: 1. NRC Generic Letter 91-04, "Changes in Technical  
Specification Surveillance Intervals to Accommodate  
24-Month Fuel Cycle," dated April 2, 1991.

Dear Sir:

This application for an amendment to the James A. FitzPatrick Nuclear Power Plant Technical Specifications proposes to extend instrumentation and miscellaneous surveillance test intervals to support 24-month operating cycles. The proposed changes follow the guidance of Generic Letter (GL) 91-04 (Reference 1). Approval of these changes will eliminate the need to execute mid-cycle outages to conduct these surveillance tests. The Authority requests approval of these proposed changes to support implementation upon startup from the Reload 12/Cycle 13 refueling outage (Currently scheduled for Fall 1996).

Additionally, this application proposes: (1) revision of the Trip Level Settings for Emergency Bus Loss of Voltage and Degraded Voltage Instrumentation, (2) revision of the Reactor Protection System (RPS) Normal Supply Electrical Protection Assembly (EPA) Undervoltage Trip Setpoint, and (3) editorial revisions, clarification and Bases changes. The instrumentation subject to this application involves the following: RPS, Primary Containment Isolation Systems, Core and Containment Cooling Systems, Control Rod Blocks, Anticipated Transient Without Scram (ATWS) Recirculation Pump Trip, Accident Monitoring and Remote Shutdown, Radiological Effluent Technical Specification (RETS) monitoring, and safety-related plant ventilation systems. Non-instrumentation changes involve surveillance frequency notations, Residual Heat Removal (RHR) cross-tie valve position verification, and instrument line excess flow check valve testing.

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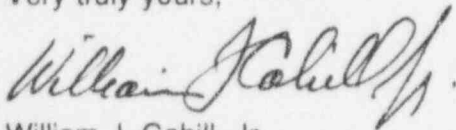
Extended surveillance intervals are identified in the proposed Technical Specifications as being performed "once per 24 months" or "R" for instrumentation located in tables. Surveillance requirements that are to remain at an 18 month surveillance interval are identified as being performed "once per 18 months" or "18M" for instrumentation located in tables. This terminology is consistent with the guidance provided in GL 91-04 and BWR Standard Technical Specifications.

The signed original of the Application for Amendment to the Technical Specifications is enclosed for filing. Attachments I and III to this application contain the proposed changes to the Technical Specifications and a mark-up of the current Technical Specifications pages, respectively. Attachment II contains the associated Safety Evaluation. Attachment IV contains a copy of References that are not docketed for either the FitzPatrick or Indian Point Unit 3 plants. The methodology used in the Instrument Drift Evaluations was developed for the Authority by contract personnel from Raytheon Nuclear, Inc. (Raytheon), formerly Ebasco Services Inc. and is proprietary to Raytheon. For these references, a Proprietary and a Non-Proprietary copy of the Reference are included in Attachment IV, along with an Affidavit signed by Raytheon that addresses the considerations listed in 10 CFR 2.790(b)(4).

In accordance with 10 CFR 50.91, a copy of this application and the associated attachments are being provided to the designated New York State official.

If you have any questions, please contact Ms. C. D. Faison.

Very truly yours,



William J. Cahill, Jr.  
Chief Nuclear Officer

cc: Attachments I, II, III only:

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All Attachments to:

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