

#### **BOSTON EDISON**

Pilgrim Nuclear Power Station Rocky Hill Road Plymouth, Massachusetts 02360

10 CFR 50.73 10 CFR 21.21

E. T. Boulette, PhD Senior Vice President - Nuclear

> April 21, 1995 BECo Ltr. #95-051

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D.C. 20555

> Docket No. 50-293 License No. DPR-35

The enclosed supplemental Licensee Event Report (LER) 95-002-01, "High Pressure Coolant Injection System and Reactor Core Isolation Cooling System Separately Inoperable on Different Dates Due to Failed Power Inverters", is submitted in accordance with 10 CFR Part 50.73 and Part 21.

In this report we made the following commitments:

- The RCIC inverter in the Control Room will be replaced with a refurbished inverter having additional design margin, prior to restart from RFO-10.
- A diverse design for the HPCI inverter in the Control Room is planned for implementation and installation prior to restart from RFO-10.
- We are reviewing the CGI procedure for changes due to this event.

Please do not hesitate to contact me if there are any questions regarding this report.

ETBoulet

E.T. Boulette, PhD

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Page 2 BECo Ltr. 95- 051

cc: Mr. Thomas T. Martin Regional Administrator, Region I U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, PA 19406

Sr. NRC Resident Inspector - Pilgrim Station

Standard BECo LER Distribution

NRC Fo (5-92)	rm 366	LIC		U.S. NUCLEAR RI	PORT (I	.ER)	ISSION		APPROVED BY OMB EXPIRES 5/3 ESTIMATED BURDEN PER RESPONSE INFORMATION COLLECTION REQUEST 50. REGARDING BURDEN ESTIMATE TO THE MANAGEMENT BRANCH (MNBB 7714), COMMISSION, WASHINGTON, DC 20555-00 REDUCTION PROJECT (3150-0104), OFFICE O WASHINGTON, DC 20503	1/95 TO COMPLY WITH THIS 0 HRS. FORWARD COMMENTS INFORMATION AND RECORDS U.S. NUCLEAR REGULATORY 01, AND TO THE PAPERWORK
FACILI	TY NAM		GRIMN	UCLEAR POW	EF STA	TION			DOCKET NUMBER (2) 05000-293	PAGE(3) 1 of 7
Differ	Press	ates Du		njection System ailed Power Inv	erters		DRT DAT		Cooling System Separately	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME N/A	DOCKET NUMBER 05000
02	0.2	0.5	05	002	01	04	21	95	FACILITY NAME	DOCKET NUMBER

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		C	OMPLETE ONE	LINE FOR EACH	CON	ONENT FAILUI	RE DESC	RIB	ED IN	THIS REP	ORT (13)			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS			CAUSE	SYST	Mai	COMPONENT	MANUFAC	TURER		ABLE TO RDS
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THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR: (Check one or more) (11)

20.45(c)

50.73(a)(2)(iv)

73.71(b)

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

20.402(b)

On February 2, 1995, at 2159 hours, the High Pressure Coolant Injection (HPCI) System became inoperable when the inverter which supplies power to the HPCI flow control circuitry tripped. The inverter automatically reset at 2159 hours, however, the inverter tripped and would not reset ten minutes later at 2209 hours. The HPCI Inverter tripped and failed due to an apparent internal fault on the "Converter Power Supply" board. The inverter was replaced and the HPCI System was tested and declared operable on February 3, 1995. On February 14, 1995, at 1255 hours, the Reactor Core Isolation Cooling (RCIC) System became inoperable when the inverter which supplies power to the RCIC flow control circuitry tripped. The RCIC Inverter tripped and failed due to an internal fault on the power module. The inverter was replaced and the RCIC System was tested and declared operable on February 14, 1995.

Corrective action taken included sending the failed inverters to the manufacturer for further evaluation. We have determined, through root cause analysis, that a probable common cause failure existed due to the inverter power supply resistors not being sufficiently elevated above the circuit boards to provide adequate heat dissipation capability.

The events occurred during power operation with the reactor mode selector switch in the RUN position. The Reactor Vessel pressure was approximately 1038 psig for the HPCI event and 1037 psig for the RCIC event with Reactor Vessel water temperature at the saturation temperature for the noted pressures. This report is submitted in accordance with 10 CFR 50.73 subparts (a)(2)(v)(D) and (a)(2)(vii)(D), and 10 CFR 21.21(c)(3)(ii). These events posed no threat to the public health and safety.

**OPERATING** 

MODE (9)

NRC Form 366 (5-92)	U.S. NUCLEAR REGU	LATORY COMMISSION		API		OMB NO.3150-01 ES 5/31/95	04
	E EVENT REPO			INFORMATION CO REGARDING BURT MANAGEMENT BI COMMISSION, WA	LLECTION REQUENTE STIMATE CANCH (MNBB SHINGTON, DC CT (3150-0104), O	SPONSE TO COMP JEST 50.0 HRS FORW TO THE INFORMATION 7714), U.S. NUCLEA 20555-0001, AND TO T FFICE OF MANAGEME	ARD COMMENTS N AND RECORDS R REGULATORY THE PAPERWORK
FACILITY NAM	E (1)	DOCKET NUMBER (2)	T	LERN	UMBER (6)		PAGE (3)
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DIL CRIM NUICI EAR DO	WER STATION	05000.293	95	(	02	01	2017

TEXT (If more space is required, use additional copies of NRC Form 366A) (17) REASON FOR SUPPLEMENT

This supplemental report is submitted because the root cause analysis and related corrective actions had not been finalized when the initial report was submitted. The initial report committed to a supplement by June 30, 1995.

# BACKGROUND

The High Pressure Coolant Injection (HPCI) System is designed to provide high pressure reactor core cooling in the event of a small break loss of cooling accident. The Reactor Core Isolation Cooling (RCIC) System is designed to provide high pressure makeup water to the reactor vessel following reactor vessel isolation. The HPCI flow control circuitry operates on 120V AC converted from 125V DC by inverter 2340-13. Inverter 2340-13 is located within Main Control Room Panel C903 and was manufactured by Abacus Controls Inc. (ACI), Model 452-4-120M5. An identical inverter, 1340-16, performs a similar function for the Reactor Core Isolation Cooling (RCIC) System and is located within Main Control Room Panel C904. Inverters 2340-13 and 1340-16 were replaced via Plant Design Change (PDC) 91-63 in November of 1991. These inverters have wider operating ranges and greater high voltage trip setpoints than the former inverters. The inverters are also equipped with an automatic reset feature for a high or low voltage trip. Each inverter is connected to an alarm in the Main Control Room. The inverter replacements were a portion of the corrective action taken for events described in Licensee Event Reports (LERs) 50-293/91-006-00, 91-021-00, and 91-025-00. A further description of these events is located in the 'Similarity to Previous Events' section of this LER.

# EVENT DESCRIPTION

On February 2, 1995, at 2159 hours, plant operators received an alarm on Main Control Room Panel C903 annunciator I-4, "HPCI Inverter Circuit Failure". The alarm immediately cleared indicating the inverter had automatically reset. The shift licensed operators investigated the alarm and found the HPCI Inverter 2340-13 hot to the touch when compared to the RCIC Inverter 1340-16. Ten minutes after the first alarm, at 2209 hours, plant operators received the "HPCI Inverter Circuit Failure" alarm again and the alarm did not clear. At that time, the licensed operators determined the HPCI System was inoperable upon the receipt of the first alarm at 2159 hours.

Problem Report (PR) 95.9048 and Maintenance Request 19500429 were written on February 2, 1995, to document and correct the problem. The NRC Operations Center was notified in accordance with 10 CFR 50.72, at 2234 hours on February 2, 1995.

U.S. NUCLEAR	REGULATORY	COMMISSION
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### LICENSEE EVENT REPORT (LER)

### TEXT CONTINUATION

#### APPROVED BY OMB NO.3150-0104 EXPIRES 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (\$150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)		PAGE (3)
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PILGRIM NUCLEAR POWER STATION	05000-293	95	002	01	3 of 7

TEXT (If more space is required, use additional copies of NRC Form 365A) (17)

On February 14, 1995, at 1255 hours, and with the HPCI system operable, plant operators received an alarm on Main Control Room Panel C904 for "RCIC Inverter Circuit Failure". The alarm did not clear. The Licensed Operators determined the RCIC System was inoperable upon receipt of the alarm.

PR95.9073 and Maintenance Request 19500552 were written on February 14, 1995, to document and correct the problem. The NRC Operations Center was notified in accordance with 10 CFR 50.72 at 1325 hours on February 14, 1995.

These separate events occurred during 100 percent reactor power operation with the reactor mode selector switch in the RUN position. The Reactor Vessel pressure was approximately 1038 psig for the HPCI event and 1037 psig for the RCIC event with Reactor Vessel water temperature at the saturation temperature for the noted pressures.

## CAUSE

NRC Farm 366

(5-92)

The direct cause of the inoperability of the HPCI System and RCIC System was the separate failure of inverters 2340-13 and 1340-16.

Maintenance personnel replaced Inverter 2340-13 with an identical, spare inverter on February 3, 1995. The HPCI System was tested and declared operable at 1320 hours on February 3, 1995. Engineering personnel inspected the failed inverter and found the converter power supply board had apparently failed. The engineers contacted the inverter manufacturer, Abacus Controls Inc. (ACI). The manufacturer reported being unaware of similar failures of inverters manufactured by the company. The failed HPCI inverter was sent to the manufacturer for failure analysis.

The failed RCIC inverter 1340-16 was replaced with an identical, spare inverter and the RCIC System was tested and declared operable at 2238 hours on February 14, 1995. Because of the HPCI inverter failure on February 2, 1995, which occured on the "Converter Power Supply Board", the failed RCIC inverter "Converter Power Supply board" was inspected. Although the RCIC "Converter Power Supply Board" was heat discolored, resistance measurements of key components resulted in expected, satisfactory readings. There was no visual evidence of heat related discoloration of any of the remaining three circuit boards of the inverters.

With the occurrence of failure of the HPCI and RCIC inverters within two weeks of each other, potential common cause failure due to external cause(s) was evaluated. The input DC voltages to the HPCI and RCIC inverters are independent and have both been in a normal range at approximately 133 VDC. The ambient temperatures in the vicinity of the inverters are within the specified operating temperatures of the inverters. No system modifications have been performed that have the potential to directly or indirectly affect HPCI or RCIC inverter performance. The electrical loading on the HPCI and RCIC inverters is approximately the same. We concluded that there was no potential for external common cause failure.

NEC FORM 366A (5-92)

(5-92)	U.S. NUCLEAR REGUL	LATORY COMMISSION			/ OMB NO.3150-01 RES 5/31/95	04
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inverters had signific possible $\epsilon$ rcess hea was also sent to the over three years for inverters, were evalu The root cause anal 452-4-120M5 inverte board to provide ade and subsequently de Inspection and testir	led HPCI and RCIC inve- cant discoloration of the it loading of this circuit b manufacturer for furthe the failures to develop, uated operable for a min ysis determined a proba- ers. The inverter power equate heat disipation. edicated by BECo for us ing of the failed inverters pufacturers facility. Infor- oship between the circuit	converter power s board which could er evaluation. Since the HPCI and RC nimum of 45 days able common caus supply power resi These inverters w se in safety-related s was completed b	supply b cause a e the fail IC replac (i.e., unt se failure stor was ere purc d applica y the ma	oard. This discolora n age related failure lures appear to be a cement inverters, ide if the 1995 refueling may exist on Abac not sufficiently elev- chased as commerci- ations.	ation is indicat e. The RCIC in age related an entical to the f poutage). us Controls In vated above t ial grade item	tive of inverter nd it took failed nc. Model he circuit s (CGI) ECo
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PILC	SRIM NUCLEAR POWER STATION	05000-295	95		002		5017
TEXT ()	f more space is required, use additional copies of NRC Form 366A)	(17)					
	A Part 21 evaluation, completed on deviation that could have created a March 23, 1995. The 10 CFR 50.72 and February 14, 1995 and the sub- requirements of Part 21.	substantial safety has notifications to the	NRC	Exe Opera	cutive Managem ations Center on	ent was not February 2,	ified on 1995
•	A review of records determined thes The HPCI and RCIC inverters in the manufacturer and are not affected.						Station.
•	The records review also determined dedicated component) to any other					cial grade or	as a
•	The failed inverters were refurbished increasing the distance (elevation) of enhances power resistor heat dissip	of the power resistor					n
	Commercial Grade Item (CGI) docur inverters was revised (to Rev. 4). The resistors greater than 2 watts. This result of the failures documented in recommendation provided by the magnetic the magnetic states and the magnetic states and the states	he focus of the revis inspection point agr this report. It was a	ion wa	as to i ith the	include an inspe e manufactures d	ction of all p design chan	ower ge as a
Corre	ctive actions planned include the follo	wing:					
•	The RCIC inverter in the Control Ro design margin, prior to restart from R		with a	refur	bished inverter h	aving additi	onal
	A diverse design for the HPCI invert	er in the Control Ro	om is	plann	ed for implemen	tation and ir	stallation
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TEXT (If more space is required, use additional copies of NRC Form 366A) ()	17)	-				
SAFETY CONSEQUENCES These separate events posed no threat to th	ne public health ar	nd safe	ty.			
The Core Standby Cooling System (CSCS) of (ADS), Core Spray System, and Residual He not part of the CSCS, the RCIC System is ca core cooling, similar to the HPCI System. Do Systems and the RCIC System were operab Systems including the HPCI System were op were to become inoperable and core cooling function to reduce reactor vessel pressure for Spray System and/or the RHR System/LPCI	eat Removal/Low apable of providing uring the time peri- le. During the time perable. In the un g was necessary, a pr low pressure co	Pressu g wate od HP e perio likely e an ADS	to the CI was od RC work of RC	bolant Injection (L ne reactor vessel as inoperable, the CIC was inoperabl the HPCI System uation (automatic	PCI) mode. for high pres other CSCS e, the CSCS and RCIC S or manual)	Although ssure S System would
The initial report was submitted in accordance inoperable due to the failure of Inverter 2340 accordance with 10 CFR 50.73(a)(2)(v)(D) b Inverter 1340-16 on February 14, 1995. The provided in NUREG 1022 Supplement 1 Sec	0-13 on February 2 ecause the RCIC e two events are re	2, 1995 System	5. Th n bec d in a	e initial report wa ame inoperable o single LER due t	s also subm lue to the la	itted in ilure of
This supplemental report is also being subm 10 CFR 21.21(c)(3)(ii) because the root cause						se failure.
SIMILARITY TO PREVIOUS EVENTS						
A review was conducted of Pilgrim Station L review focused on LERs involving RCIC or H events reported via LERs 50-293/85-029-00 involved inverters manufacturered by Topaz	IPCI 5tem inver , 91-006-00, 91-02	ter pro	blem	s. The review ide	entified previ	ous
For LER 85-029-00, the HPCI inverter trippe cause of the HPCI inverter trip was fluctuation seconds restoring the HPCI System operability	on of the inverter in					
For LER 91-006-00, the RCIC inverter and the The inverters tripped when the Recirculation time of the event, the 125V DC Battery 'A' and via 125 VDC Bus 'A'. The 125 VDC Battery to the HPCI Inverter via 125 VDC Bus 'B'. The being powered from Safety-Related 4160 VA actions taken are described in the following states.	System Loop 'B' nd Battery Charge 'B' and the 125 VI he 125 VDC batte AC Bus A5 via Bus	r 'A' w DC bac ry chai 8 B1 ar	gene ere s ckup rger ' nd Bu	rator set/pump wa upplying power to battery charger w A' and backup ba	as restarted. the RCIC in rere supplyin ttery charge	At the nverter og power r were

NRC FORM 366A (5-92)

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PILGRIM NUCLEAR POWER STATION	05000-293	95		002	01	7 of 7
via the 'A' 125 VDC Bus. Final corrective ac LER 91-025-00.	tions taken are des	cribed		and the second sec	g the RCIC I mary of	nverter
LER 91-025-00. For LER 91-025-00, the RCIC Inverter trippe overspeed trip on October 30, 1991. The in (RHR) Pump was started. The RHR pump s transient which caused the RCIC Inverter to was implemented to replace the RCIC and H inverters (ACI Model 452-4-120M5) having a the three 12 CDC Battery Chargers were re approved to expected AC voltage transie	ed while the RCIC S iverter had previous start caused an AC trip. Corrective Ac HPCI Inverters (Top automatic reset, hig eplaced via PDC 92 ents.	System sly tripp voltage tion tal az Ele her trip 2-38 wi	was be ed whe transie ctronics setpoir	ing restarted in the 'A' Res ent that resuluded the follo ) in the Contin ths and wider	following an idual Heat R ted in a DC v owing. PDC rol Room with ranges. Ad	emoval voltage 91-63 n new ditionally
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LER 91-025-00. For LER 91-025-00, the RCIC Inverter trippe overspeed trip on October 30, 1991. The in (RHR) Pump was started. The RHR pump s transient which caused the RCIC Inverter to was implemented to replace the RCIC and H inverters (ACI Model 452-4-120M5) having a the three 12 CC Battery Chargers were re	ed while the RCIC Soverter had previous start caused an AC trip. Corrective Ac HPCI Inverters (Top automatic reset, hig eplaced via PDC 92 ents.	System sly tripp voltage tion tal az Ele her trip 2-38 wi	was be ed whe transie ctronics setpoir	ing restarted in the 'A' Res ent that resuluded the follo ) in the Contin ths and wider	following an idual Heat R ted in a DC v owing. PDC rol Room with ranges. Ad	emoval voltage 91-63 n new ditionally