

SNUPPS

Standardized Nuclear Unit
Power Plant System

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Rockville, Maryland 20850
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September 19, 1984

SLNRC 84- 115 FILE: 0541/M-189
SUBJ: Preservice Inspection Relief
Request - Wolf Creek Plant

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
US Nuclear Regulatory Commission
Washington, D.C. 20555

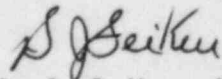
Docket No.: STN 50-482

Ref: SLNRC 84-112 dtd. 9/6/84: Preservice Inspection Relief Request
- Wolf Creek Plant

Dear Mr. Denton:

The reference letter provided data in support of Preservice Inspection (PSI) relief requests involving selected Wolf Creek component and piping systems and indicated that a supplemental submittal addressing relief requests for certain weldments in the reactor pressure vessel will be forthcoming. The purpose of this letter is to forward via Appendices A through E, attached herewith, details concerning partial relief requests for the Wolf Creek vessel. The attached submittal should be considered the final relief request for the Wolf Creek preservice inspection program.

Very truly yours,



S. J. Seiken, Manager
Quality Assurance

SJS/dck/8b16

Attachments: A thru E

cc: G. L. Koester, KGE
J. M. Evans, KCPL
D. F. Schnell, UE
J. H. Neisler/B. H. Little, USNRC/Cal
H. Bundy, USNRC/WC
D. R. Hunter, Region IV
B. L. Forney, Region III
R. C. DeYoung, USNRC/IE:HQ
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APPENDIX A

System: Reactor Pressure Vessel

Category: B-A

Component Description: Flange to Vessel Weld,
(I.D. #1-RV-101-121)

Code Requirement: 100% Volumetric Examination

Areas for Relief: 25% of Weld Volume

Basis for Relief: Parallel scan portion of examination can only be done from lower side due to presence of flange taper above the weld. Complete perpendicular scan was done from flange mating surface.

Alternate Testing: None

APPENDIX B

System: Reactor Pressure Vessel

Category: B-D

Component Description: Outlet Nozzle A to Vessel Weld,
(I.D. #1-RV-107-121-A),
Outlet Nozzle D to Vessel Weld,
(I.D. #1-RV-107-121-B),
Outlet Nozzle E to Vessel Weld,
(I.D. #1-RV-107-121-C),
Outlet Nozzle H to Vessel Weld,
(I.D. #1-RV-107-121-D)

Code Requirement: 100% Volumetric Examination

Areas for Relief: Approximately 10% of total weld volume for each nozzle.

Basis for Relief: Approximately 10% of the total weld volume for each outlet nozzle is obstructed by contact between the examination head and the nozzle knuckle extending from the nozzle opening through the plane of the Reactor Pressure Vessel Inner Diameter.

Alternate Testing: None

APPENDIX C

System: Reactor Pressure Vessel

Category: B-A

Component Description: Lower Head to Dollar Plate Weld,
(I.D. #1-RV-102-151)

Code Requirement: 100% Volumetric Examination

Areas for Relief: A total of approximately 10% of the Lower Head to Dollar
Plate Weld.

Basis for Relief: Obstructions presented by the instrumentation nozzles when
scanning the Lower Head to Dollar Plate Weld.

Alternate Testing: None

APPENDIX D

System: Reactor Pressure Vessel

Category: B-A

Component Description: Lower Head to Shell Weld
(O.D. #1-RV-101-141)

Code Requirement: 100% Volumetric Examination
0°, 45° and 60° Ultrasonic Shear Wave
Examination Angles.

Areas for Relief: Perpendicular Examination (shooting down) for the 60°
ultrasonic shear wave examination not performed. A
45° longitudinal wave examination was performed in
lieu of the 60° shear wave examination.

A total of approximately 10% of the lower head to
shell weld was not examined for the perpendicular
examination (shooting down).

Basis for Relief: O.D. surface taper geometry limits ultrasonic head
contact.

Alternate Testing: None

APPENDIX E

System: Reactor Pressure Vessel

Category: B-A

Component Description: Meridional Welds in Lower Head
(I.D. #1-RV-101-154A)
(I.D. #1-RV-101-154B)
(I.D. #1-RV-101-154C)
(I.D. #1-RV-101-154D)

Code Requirement: 100% Volumetric Examination

Areas for Relief:	<u>Weld Number</u>	<u>Percentage Requesting Relief</u>
	1-RV-101-154-A	22%
	1-RV-101-154-B	27%
	1-RV-101-154-C	12%
	1-RV-101-154-D	14%

Basis for Relief: O.D. surface condition limits ultrasonic head contact.

Alternate Testing: None