

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Duane Arnold Energy Center	DOCKET NUMBER (2) 0 5 0 0 0 0 3 3 1 1	PAGE (3) 1 OF 0 1 2
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TITLE (4)  
RCIC Steam Supply Valve Inoperability

EVENT DATE (5)				LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YFAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)				
0 8	0 8	8 4	8 4	0 2	5	0 0	0 9	0 7	8 4	None		0 5 0 0 0 0			
												None		0 5 0 0 0 0	

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																						
POWER LEVEL (10) 0 9 3	20.402(b)	20.405(a)(1)(i)	20.405(a)(1)(ii)	20.405(a)(1)(iii)	20.405(a)(1)(iv)	20.405(a)(1)(v)	20.405(c)	50.36(c)(1)	50.36(c)(2)	50.73(a)(2)(i)	50.73(a)(2)(ii)	50.73(a)(2)(iii)	50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vi)	50.73(a)(2)(vii)	50.73(a)(2)(viii)(A)	50.73(a)(2)(viii)(B)	50.73(a)(2)(ix)	73.71(b)	73.71(c)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	

LICENSEE CONTACT FOR THIS LER (12)										TELEPHONE NUMBER		
NAME Michael S. Harris - Technical Engineer										AREA CODE 3 1 1 9		
										8 5 1 1 - 1 7 1 3 1 0 1 6		

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS		
	Bin	2101	L121010	Yes							

SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)										NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 08/08/84, while performing a routine Reactor Core Isolation Cooling System (RCIC) surveillance test, the electrical supply breaker for the RCIC steam supply valve was found to trip each time the valve was cycled closed. Investigation revealed that the torque switch in the Limitorque motor operator was out of adjustment. The torque switch was readjusted and the valve cycled satisfactorily with no further supply breaker trips.

Throughout the brief period RCIC was inoperable for repair of the valve, all necessary safety systems were operable and the safe operation of the plant was not compromised.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  Duane Arnold Energy Center	DOCKET NUMBER (2)  0   5   0   0   0   3   3   1	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8   4	-   0   2   5	-   0   0	0   2	OF	0   2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 08/08/84 at 0319 hours, while the reactor was in the run mode at approximately 93% power, a routine monthly operability surveillance test of the Reactor Core Isolation Cooling System (BN) was being performed in accordance with Tech Spec 4.5.E. During the test, the electrical supply breaker to the RCIC steam supply valve (BN-20-2404) tripped on overcurrent immediately after the valve was cycled closed. Although the valve would have opened on an actual RCIC initiation signal, the valve would not have reopened after RCIC was secured; thereby rendering RCIC inoperable for a second initiation. In accordance with Tech Spec 3.5.E.2, the HPCI system was immediately demonstrated operable, RCIC was declared inoperable, and a 7 day LCO was initiated.

Troubleshooting revealed that the torque switch in the Limitorque motor operator was out of adjustment and was failing to de-energize the motor after the valve had closed and seated. The torque switch was reset and the valve was cycled 10 times with no further trips of the electrical supply breaker. The RCIC surveillance test was then resumed and completed without further incident. RCIC was subsequently declared operable and the 7 day LCO was terminated at 1414 hours the same day.

Throughout the brief period that RCIC was inoperable (approximately 11 hours), the reactor remained in steady state operation and all necessary safety systems were fully operable. There was no affect on the health and safety of the public and the safe operation of the plant was not adversely affected. There has been one previous occurrence of RCIC being inoperable in 1984 (see LER 84-018).

Iowa Electric Light and Power Company

DAEC-84- 567

September 7, 1984

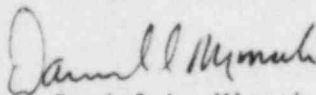
U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Subject: Duane Arnold Energy Center  
Docket No. 50-331  
Op. License DPR-49  
Licensee Event Report No. 84-025

Gentlemen:

In accordance with 10 CFR 50.73 please find attached a copy of the  
subject Licensee Event Report.

Very truly yours,



Daniel L. Mineck  
Plant Superintendent - Nuclear  
Duane Arnold Energy Center

DLM/MSH/kp

attachment

cc: Mr. James G. Keppler  
Regional Administrator  
Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

NRC Resident Inspector - DAEC

File A-118a

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