

AVERAGE DAILY UNIT POWER LEVEL

Completed by J. P. Ronafalvy

Docket No. 50-311
 Unit Name Salem # 2
 Date Sept. 10, 1984
 Telephone 609-935-6000
 Extension 4455

Month August 1984

Day Average Daily Power Level
(MWe-NET)

Day Average Daily Power Level
(MWe-NET)

| | | | |
|----|-------------|----|-------------|
| 1 | <u>0</u> | 17 | <u>1068</u> |
| 2 | <u>0</u> | 18 | <u>1072</u> |
| 3 | <u>0</u> | 19 | <u>1071</u> |
| 4 | <u>0</u> | 20 | <u>1089</u> |
| 5 | <u>0</u> | 21 | <u>1091</u> |
| 6 | <u>0</u> | 22 | <u>1099</u> |
| 7 | <u>0</u> | 23 | <u>1098</u> |
| 8 | <u>395</u> | 24 | <u>1090</u> |
| 9 | <u>1020</u> | 25 | <u>1096</u> |
| 10 | <u>1103</u> | 26 | <u>770</u> |
| 11 | <u>1082</u> | 27 | <u>348</u> |
| 12 | <u>1084</u> | 28 | <u>936</u> |
| 13 | <u>1104</u> | 29 | <u>1096</u> |
| 14 | <u>1087</u> | 30 | <u>1101</u> |
| 15 | <u>1080</u> | 31 | <u>1100</u> |
| 16 | <u>1100</u> | | |

Pg. 8, 1-7 R1

8407200403 840831
 PDR ADOCK 05000311
 R PDR

IEQ4
 111

OPERATING DATA REPORT

Docket No. 50-311
 Date Sept. 10, 1984
 Telephone 935-6000
 Extension 4455

Completed by J. P. Ronafalvy

Operating Status

| | | |
|--|--------------------|--------------|
| 1. Unit Name | <u>Salem No. 2</u> | <u>Notes</u> |
| 2. Reporting Period | <u>August 1984</u> | |
| 3. Licensed Thermal Power (Mwt) | <u>3411</u> | |
| 4. Nameplate Rating (Gross MWe) | <u>1162</u> | |
| 5. Design Electrical Rating (Net MWe) | <u>1115</u> | |
| 6. Maximum Dependable Capacity (Gross MWe) | <u>1149</u> | |
| 7. Maximum Dependable Capacity (Net MWe) | <u>1106</u> | |
| 8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason | <u>N/A</u> | |

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

| | <u>This Month</u> | <u>Year to Date</u> | <u>Cumulative</u> |
|--|-------------------|---------------------|-------------------|
| 11. Hours in Reporting Period | <u>744</u> | <u>5855</u> | <u>25296</u> |
| 12. No. of Hrs. Reactor was Critical | <u>575.8</u> | <u>2630.9</u> | <u>14339.5</u> |
| 13. Reactor Reserve Shutdown Hrs. | <u>0</u> | <u>1442.9</u> | <u>3533.6</u> |
| 14. Hours Generator On-Line | <u>555.8</u> | <u>2455.2</u> | <u>13872.5</u> |
| 15. Unit Reserve Shutdown Hours | <u>0</u> | <u>0</u> | <u>0</u> |
| 16. Gross Thermal Energy Generated (MWH) | <u>1823350</u> | <u>7966695</u> | <u>41437767</u> |
| 17. Gross Elec. Energy Generated (MWH) | <u>605460</u> | <u>2643560</u> | <u>13511850</u> |
| 18. Net Elec. Energy Generated (MWH) | <u>577255</u> | <u>2481315</u> | <u>12798563</u> |
| 19. Unit Service Factor | <u>74.7</u> | <u>41.9</u> | <u>54.8</u> |
| 20. Unit Availability Factor | <u>74.7</u> | <u>41.0</u> | <u>54.8</u> |
| 21. Unit Capacity Factor (using MDC Net) | <u>70.2</u> | <u>38.3</u> | <u>45.7</u> |
| 22. Unit Capacity Factor (using DER Net) | <u>69.6</u> | <u>38.0</u> | <u>45.4</u> |
| 23. Unit Forced Outage Rate | <u>25.3</u> | <u>58.1</u> | <u>36.1</u> |
| 24. Shutdowns scheduled over next 6 months (type, date and duration of each) | <u>N/A</u> | | |

25. If shutdown at end of Report Period, Estimated Date of Startup:

8-8-84

26. Units in Test Status (Prior to Commercial Operation):

| | | |
|----------------------|-----------------|-----------------|
| | <u>Forecast</u> | <u>Achieved</u> |
| Initial Criticality | <u>6/30/80</u> | <u>8/2/80</u> |
| Initial Electricity | <u>9/1/80</u> | <u>6/3/81</u> |
| Commercial Operation | <u>9/24/80</u> | <u>10/13/81</u> |

8-1-7.R2

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH August 1984

Docket No. 50-311
Unit Name Salem No.2
Date Sept. 10, 1984
Telephone 609-935-6000
Extension 4455

Completed by J.P. Ronafalvy

| No. | Date | Type 1 | Duration Hours | Reason 2 | Method of Shutting Down Reactor | License Event Report | System Code 4 | Component Code 5 | Cause and Corrective Action to Prevent Recurrence Power Operated Relief and Safety/ Relief Valves Reactor |
|--------|------|-----------|-------------------|-------------|--|----------------------------|------------------|---------------------|---|
| 84-218 | 7-25 | F | 173.4 | A | 3 | - | CJ | VALEX | |
| 84-232 | 8-26 | F | 15.0 | A | 3 | - | CH | PUMPXX | Feedwater Pump |
| 84-234 | 8-27 | F | 19.9 | A | 5 | - | CH | PUMPXX | Feedwater Pump |

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of
Previous Outage
5-Load Reduction
9-Other

4 Exhibit G
Instructions
for Prepara-
tion of Data
Entry Sheets
for Licensee
Event Report
(LER) File
(NUREG 0161)

5 Exhibit 1
Salem as
Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH August 1984

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: September 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/935-6000 X4455

| *DCR NO. | PRINCIPLE SYSTEM | SUBJECT |
|----------|---|--|
| 2EC-1327 | Circulating Water | Install permanent discharge water sampling system for continuous on grab sampling of circulating water discharge system. |
| 2SC-0483 | #21 & 23 Chiller Cond. Recirc. Pumps | Change shaft sleeve material from stainless steel (316) to monel. |
| 2SC-0606 | Service Water Pump Motors | Vent the oil level columns in the bearing oil lubrication systems on Nos. 21, 22, 23, 24, 25 and 26 Service Water Pump Motors. |

* Design Change Request

MAJOR PLANT MODIFICATIONS
REPORT MONTH August 1984

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: September 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

DCR NO. 10CFR 50.59 SAFETY EVALUATION

- 2EC-1327 This change installs a sampling system off of the Circulating Water Discharge System. No unreviewed safety or environmental questions are involved.
- 2SC-0483 This change involves the Chiller-Condenser Water Recirculation Pump sleeves. The material change does not affect the basis of the Technical Specifications. No unreviewed safety or environmental questions are involved.
- 2SC-0606 The modification of the oil sight gauge vent will improve accuracy of the oil level indication. No unreviewed safety or environmental questions are involved.

* Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 2

WO NO DEPT UNIT EQUIPMENT IDENTIFICATION

84-07-12-199-1
SMD

2 CHECK VALVE 21SW77

FAILURE DESCRIPTION: RADIOGRAPHY SHOWS VALVE OPEN WHEN IT SHOULD BE CLOSED

CORRECTIVE ACTION: INSTALLED NEW SPRING AND BELZONED LINING IN VALVE

84-08-01-163-4

SMD

2 VALVE 22SJ43

FAILURE DESCRIPTION: BONNET BOLTS REQUIRE TORQUING

CORRECTIVE ACTION: BONNET BOLTS TORQUED

84-08-01-201-1
SMD

2 VALVE 2CC183

FAILURE DESCRIPTION: BROKEN HANDWHEEL

CORRECTIVE ACTION: REPLACED BONNET ASSEMBLY

84-07-09-799-3
SMD

2 21 BAT PUMP

FAILURE DESCRIPTION: PUMP LEAKS

CORRECTIVE ACTION: REBUILT PUMP

0099024594

SMD

2 21SW223 PIPING

FAILURE DESCRIPTION: PIPING BELOW VALVE LEAKS

CORRECTIVE ACTION: FABRICATED NEW SPOOL PIECE AND INSTALLED A NEW SPACER

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

UNIT 2

WO NO DEPT UNIT EQUIPMENT IDENTIFICATION

84-07-09-699-7
SMD

2

22SW223

FAILURE DESCRIPTION: FLANGE LEAK ON VALVE

CORRECTIVE ACTION: REBUILT 8" FLANGE AND REPLACED SPACER

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 2
AUGUST 1984

Unit 2 began the month in Mode 5 as review of the Reactor Trip/Safety Injection occurrence on July 25, 1984 continued. Investigation of this occurrence has revealed that valve 2PR47 was open when the motor operated isolation valve, 2PR6, was cycled open at completion of the test of the POPS system. This resulted in the rapid pressure drop experienced by the Reactor Coolant System. The 2PR6 valve did not close for approximately 4.5 minutes after the operator initiated the close signal. This allowed the reactor pressure to drop to Reactor Trip pressure of 1865 psig and subsequently Safety Injection pressure of 1765 psig. The apparent cause of the delay in valve closure was found to be a combination of a broken wire on the closing torque switch and a close torque switch setting. The 2PR6 wiring has been repaired and the torque switch setting adjusted. Valve 2PR47 has been removed and the line capped. Unit heatup commenced on August 5, 1984 at 1700 hours. The Unit attained criticality on August 7, 1984 at 1243 hours. The Unit was synchronized on August 8, 1984 at 0523 hours. Full power was reached on August 8, 1984 at 1153 hours. The Unit remained at full power until August 26, 1984, when No. 21 SGFP tripped due to an erroneously high output on the magnetic speed pick-up which resulted in an erroneous overspeed trip. The Control Room Operators began reducing Reactor Power level to less than 60% to recover levels in the Steam Generators; however, the Unit experienced a Reactor Trip at 1711 hours due to No. 24 Steam Generator Lo-Lo level. On August 27, 1984 at 0808 hours the Unit was synchronized and full power operation was reached on August 28, 1984.

REFUELING INFORMATION

COMPLETED BY: J. Ronafalvy DOCKET NO.: 50-311
 UNIT NAME: Salem 2
 DATE: September 10, 1984
 TELEPHONE: 609/935-6000
 EXTENSION: 4455

Month August 1984

1. Refueling information has changed from last month:
 YES _____ NO X
2. Scheduled date for next refueling: January 5, 1985
3. Scheduled date for restart following refueling: March 17, 1985
4. A) Will Technical Specification changes or other license amendments be required?
 YES _____ NO X
 NOT DETERMINED TO DATE 8/1/84
- B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
 YES _____ NO X
 If no, when is it scheduled? November 1984
5. Scheduled date(s) for submitting proposed licensing action:
December 1984 (if required)
6. Important licensing considerations associated with refueling:
NONE
7. Number of Fuel Assemblies:
 A) Incore 193
 B) In Spent Fuel Storage 72
8. Present licensed spent fuel storage capacity: 1170
 Future spent fuel storage capacity: 1170
9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: March 2004

8-1-7.R4



Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

September 10, 1984

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO. 50-311

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of August 1984 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

LKM:sbh

cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosure
8-1-7.R4
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The Energy People