



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

June 18, 1992

Docket Nos. 50-313
and 50-368

Mr. Neil S. Carns
Vice President, Operations ANO
Entergy Operations, Inc.
Route 3 Box 137G
Russellville, Arkansas 72801

Dear Mr. Carns:

SUBJECT: REVIEW OF RESPONSE TO GENERIC LETTER 88-20, SUPPLEMENT NO. 4 -
INDIVIDUAL PLANT EXAMINATIONS FOR EXTERNAL EVENTS - ARKANSAS
NUCLEAR ONE, UNITS 1 AND 2 (ANO-1&2) (TAC NOS. M83588 AND 83589)

Supplement 4 to Generic Letter 88-20 was issued on June 28, 1991, to require each licensee and each Construction Permit holder to conduct an individual plant examination of external events (IPEEE). Guidance was provided with the generic letter supplement in the form of NUREG-1407, "Procedural and Submittal Guidance for the Individual Plant Examination of External Events (IPEEE) for Severe Accident Vulnerabilities." The supplement requested a 180-day response (December 26, 1991) that would (1) identify the method and approach selected for the IPEEE, (2) describe the method to be used if it has not previously been submitted for staff review, and (3) identify the milestones and schedule for performing the IPEEE and submittal of the results to the NRC. Licensees were requested in the supplement to submit the IPEEE results to the NRC for review by June 28, 1994 (3 years after issuance of the supplement), to ensure that the intent of the Commission's Severe Accident Policy Statement will be met by mid-1995.

We have reviewed your letter dated December 19, 1991, submitted in response to Generic Letter 88-20, Supplement 4. We find that your selected IPEEE methods are acceptable. However, your submittal did not provide projected milestones and schedules. The reason given in your submittal for not providing this information was that the staff had not issued the Supplemental Safety Evaluation Report (SSER) approving the USI A-46 Generic Implementation Procedure (GIP). This SSER was issued by the staff on May 22, 1992, via Supplement 1 to Generic Letter 87-02. Therefore, we request that you update your IPEEE plans and provide your projected milestones and schedules to the NRC no later than September 18, 1992. The basis for that response date is that those plants covered under USI A-46 are required to respond by September 18, 1992, with their USI A-46 program. Most licensees for these plants have linked their IPEEE response to USI A-46.

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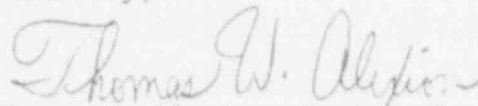
June 18, 1992

If your submittal schedule is not consistent with the NRC's requested date of June 1994, you should forward your justification with sufficient discussion to provide the NRC staff with a basis for determining the acceptability of your schedules. However, we request that your submittal be provided no later than mid-1995. By doing so, your efforts will more closely reflect the level of safety significance attributed to this effort by the Commission when the goal of closing severe accident issues by June 1995 was established.

The NRC, with assistance from the Lawrence Livermore Laboratory and the utility Seismicity Owners Group, assisted by the Electric Power Research Institute, has conducted probabilistic seismic hazard studies for nuclear power plant sites east of the Rocky Mountains. Although no plants were found to have exceptional vulnerabilities, eight plants at five sites were identified as needing further evaluation by the NRC staff based on comparison of their probabilistic seismic hazard to their deterministic response spectra. ANO-1 was one of those plants so identified. The plant design is believed to meet the 10 CFR Part 50 General Design Criteria (Appendix A) and other applicable NRC regulations. The result of this review indicated that the NRC does not need to reestablish the seismic licensing basis and no immediate action on your part is necessary at this time. However, the staff will further examine the plant vulnerability with respect to seismic hazard upon receipt of your IPEEE. From that examination, the staff will have a better understanding of what seismic vulnerabilities, if any, warrant consideration for corrective action. The staff may then, if appropriate, consider backfit under 10 CFR 50.109 for proposed plant improvements.

Note that, through your IPE process, we expect that you are now familiar with Level 1 Probabilistic Risk Assessment (PRA) technology. The information you obtained and expertise you acquired through the IPE process should be used and built upon to fulfill part of the IPEEE requirements. The staff believes that, with the safety systems and equipment being identified through the IPE process, you should be able to complete the IPEEE within the requested time frame.

Sincerely,



Thomas W. Alexion, Project Manager
Project Directorate IV-1
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

cc: See next page

Mr. Neil S. Carns
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Little Rock, Arkansas 72205-3867

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Sincerely,

Original signed by:

Thomas W. Alexion, Project Manager
Project Directorate IV-1
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

cc: See next page

DISTRIBUTION:

Docket File	NRC & Local PDRs	PD4-1 Reading
S. Peterson	M. Virgilio	J. Larkins
S. Little	PD4-1 Plant File	T. Alexion
OGC	ACRS (10) (P-315)	A. B. Beach, RG IV
BBoger	R. Hernan	C. Ader
J. Chen		

OFC	LA:PD4-1	PM:PD4-1	PM:PD4-1	D:PD4-1
NAME	SLittle	TAlexion	SPeterson	JLarkins
DATE	6/18/92	6/18/92	6/18/92	6/18/92