Docket Nos. 50-277 and 50-278

> Mr. George J. Beck Director- Licensing, MC 52A-5 Philadelphia Electric Company Nuclear Group Headquarters Correspondence Control Desk P. O. Box No. 195 Wayne, Pennsylvania 19087-0195

Dear Mr. Beck:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION RE: MAIN STEAM SAFETY AND RELIEF VALVE SURVEILLANCE TECHNICAL SPECIFICATION CHANGE FOR PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3 (TAC NOS. M83704 AND M83705)

By letter dated May 18, 1992, Philadelphia Electric Company (PECo) requested a revision to the Peach Bottom Atomic Power Station, Units 2 and 3 Technical Specifications regarding the frequency of inspection and replacement of the Main Steam Safety Valves and Relief Valves. After reviewing your submittal, the staff has concluded that some additional information is necessary to make a safety determination. The additional information required is described in the enclosure.

Response to the enclosed Request for Additional Information (RAI) is requested within 30 days from receipt of this letter.

This requirement affects fewer than ten respondents and, therefore, is not subject to Office of Management and Budget Review under P.L. 96-511.

Should you have any questions regarding this RAI, please contact me at (301) 504-2426.

Sincerely,

Joseph W. Shea, Acting Project Manager Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosure:

RAI

cc w/enclosure:

See next page DISTRIBUTION:

Docket File NRC & Local PDRs

PDI-2 Reading SVarga JCalvo CMiller

RClark OGC BRuland, RGN-I

MO'Brien(2) JShea

ACRS(10)

EWenzinger, RGN-I

OFF1CE	POJ/A/CR	PDI-2/PMo	PDI-2/D		
NAME	MO BRICH	JSHEA: TO	CMILLER		
DATE	1 1/1/192	6/11/92	6 /11 /92	1 1	11

OFFICIAL RECORD COPY FILENAME: A:\PB00000.RAI



NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20566

June 11, 1992

Docket Nos. 50-277 and 50-278

Mr. George J. Beck
Director- Licensing, MC 52A-5
Philadelphia Electric Company
Nuclear Group Headquarters
Correspondence Control Desk
P. J. Box No. 195
Wayne, Pennsylvania 19087-0195

Dear Mr. Beck:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION RE: MAIN STEAM SAFETY AND RELIEF VALVE SURVEILLANCE TECHNICAL SPECIFICATION CHANGE FOR PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3 (TAC NOS. M83704 AND M83705)

By letter dated May 18, 1992, Philadelphia Electric Company (PECo) requested a revision to the Peach Bottom Atomic Power Station, Units 2 and 3 Technical Specifications regarding the frequency of inspection and replacement of the Main Steam Safety Valves and Relief Valves. After reviewing your submittal, the staff has concluded that some additional information is necessary to make a safety determination. The additional information required is described in the enclosure.

Response to the enclosed Request for Additional Information (RAI) is requested within 30 days from receipt of this letter.

This requirement affects fewer than ten respondents and, therefore, is not subject to Office of Management and Budget Review under P.L. 96-511.

Should you have any questions regarding this RAI, please contact me at (301) 504-2426.

Sincerely.

Joseph W. Shea, Acting Project Manager

Project Directorate I-2

Division of Reactor Projects - I/II office of Nuclear Reactor Regulation

Enclosure:

cc w/enclosure: See next page Mr. George J. Br.k Philadelphia Electric Company

Peach Bottom Atomic Power Station, Units 2 and 3

CC:

J. W. Durham, Sr., Esquire Sr. V.P. & General Counsel Philadelphia Electric Company 2301 Market Street, S26-1 Philadelphia, Pennsylvania 19101

Philadelphia Electric Company ATTN: Mr. D. B. Miller, Vice President Peach Bottom Atomic Power Station Route 1, Box 208 Delta, Pennsylvania 17314

Philadelphia Electric Company ATTN: Regulatory Engineer, A1-2S Peach Bottom Atomic Power Station Route 1, Box 208 Delta, Pennsylvania 17314

Resident Inspector
U.S. Nuclear Regulatory Commission
Peach Bottom Atomic Power Station
P.O. Box 399
Delta, Pennsylvania 17314

Regional Administrator, Region I U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, Pennsylvania 19406

Mr. Roland Fletcher Department of Environment 201 West Preston Street Baltimore, Maryland 21201

Carl D. Schaefer External Operations - Nuclear Delmarva Power & Light Company P.O. Box 231 Wilmington, DE 19899 Mr. William P. Dornsife, Director Bureau of Radiation Protection Pennsylvania Department of Environmental Resources P. O. Box 2063 Harrisburg, Pennsylvania 17120

Board of Supervisors Peach Bottom Township R. D. \$1 Delta, Pennsylvania 17314

Public Service Commission of Maryland Engineering Division ATTN: Chief Engineer 231 E. Baltimore Street Baltimore, MD 21202-3486

Mr. Richard McLean
Power Plant and Environmental
Review Division
Department of Natural Resources
B-3, Tawes States Office Building
Annapolis, Maryland 21401

REQUEST FOR ADDITIONAL INFORMATION REGARDING PROPOSED REVISIONS
TO THE PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3,
TECHNICAL SPECIFICATIONS RELATING TO MAIN STEAM SAFETY
AND RELIEF VALVE SURVEILLANCE FREQUENCY

Philadelphia Electric Company (PECo or the licensee) requested NRC review of a revision to the Peach Bottom Atomic Power Station (PBAPS), Units 2 and 3, Technical Specifications concerning the frequency of inspection and replacement of Main Steam Safety Valves and Relief Valves. The staff has completed a preliminary review of the licensee's request. Based on that review, the staff has identified the following items for which additional supporting information is necessary to complete the review.

- 1. In the safety discussion, PECo concluded that no time-based failure mechanism is evident from the review of the as-found Surveillance Test (ST) data since 1987. The licensee is requested to provide information on any Safety or Relief Valve (SRV) ST failures seen during that period, including magnitude and direction of failures and a comparison of the observed setpoint drift with applicable ASME Boiler and Pressure Vessel Code requirements and guidelines.
- Discuss the significance of the failures described in question 1 when compared to the SRV performance assumed in the Updated Final Safety Analysis Report, specifically setpoint tolerance.