

February 14, 1996

Mr. Michael W. Lyon  
Director - Licensing  
Clinton Power Station  
P. O. Box 678  
Mail Code V920  
Clinton, IL 61727

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SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING EXTENSION OF DRYWELL BYPASS TEST INTERVALS FOR CLINTON POWER STATION, UNIT 1 (TAC NO. M90224)

Dear Mr. Lyon:

Surveillance Requirement 3.6.5.1.1 of the Clinton Power Station Technical Specifications requires that a drywell bypass leakage rate test (DBLRT) be performed once every 18 months. Amendment No. 96 to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1, issued on March 1, 1995, permitted a one-time only change to delete performance of this test during the fifth refueling outage that began in March 1995. As discussed in the Safety Evaluation to the amendment, the staff concluded that sufficient technical basis existed to permit this one-time approval. However, further relaxation of the DBLRT frequency, upwards to the ten-year frequency that was originally requested, would require further staff examination of alternative, periodic monitoring capabilities.

The staff recently approved a similar request to extend the frequency of the DBLRT for upwards to ten years for the River Bend facility. This is documented in Amendment No. 87 to the River Bend docket dated January 29, 1996. Based on the River Bend review, the staff has determined that the attached request for additional information will be necessary to complete the review for the Clinton Power Station.

This request for information affects nine or fewer respondents, and therefore, is not subject to the Office of Management and Budget review under Public Law 96-511.

If you have any questions, please contact me at (301) 415-1364.

Sincerely,  
Original Signed By:  
Douglas V. Pickett, Senior Project Manager  
Project Directorate III-C  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-461

Enclosure: As stated.

cc: See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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Sincerely,

Douglas V. Pickett, Senior Project Manager  
Project Directorate III-3  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-461

Enclosure: As stated

cc: See next page

REQUEST FOR ADDITIONAL INFORMATION

RELATED TO THE

CLINTON POWER STATION, UNIT NO. 1

DRYWELL BYPASS LEAKAGE RATE TEST FREQUENCY EXTENSION

DOCKET NO. 50-461

1. How is the technical specification verification of closure of the purge valves performed?
2. What are the leakage and closure verification requirements for the drywell equipment hatch? What assurance is there, if no drywell bypass leakage rate test is performed, that the drywell equipment hatch is not leaking excessively?
3. What requirements are there for leakage rate testing of the drywell after modifications to the drywell structure or penetrations?
4. Demonstrate, if possible, that one purge valve can be left open without exceeding the containment failure pressure.
5. Discuss the consequences of a loss of the sealing medium of the electrical penetrations. Estimate the  $A/\sqrt{K}$  value for loss of sealing medium for both a single penetration and for all penetrations.
6. Provide a list of all drywell isolation valves along with their diameters. Indicate which valves close automatically on indication of a LOCA and which valves are locked closed when drywell integrity is required. Indicate which valves have position indication in the control room. For those valves which have neither automatic isolation or are not locked closed, what assurance is there that the valves will be shut if a LOCA occurs? Can it be demonstrated that all valves below a certain diameter can be open during a LOCA without exceeding the design  $A/\sqrt{K}$  value? If relevant material is in the USAR, a reference is sufficient.
7. Performance of a drywell bypass leakage rate test following a refueling outage confirms that all penetrations are properly isolated and that excessive bypass leakage due to seal or valve damage does not exist. In lieu of performing this test, discuss how plant procedures will provide sufficient assurance that unacceptable drywell bypass leakage does not exist.
8. Discuss any operational problems (problems with tests, material problems, etc.) which have occurred and are relevant to extending the drywell bypass leakage rate test interval.

Mr. Michael W. Lyon  
Illinois Power Company

Clinton Power Station  
Unit No. 1

cc:

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