

L I C E N S E E E V E N T R E P O R T ( L E R )

FACILITY NAME (1) Arkansas Nuclear One, Unit Two DOCKET NUMBER (2) | PAGE (3)  
 10151010101 31 61 8110F11

TITLE (4) Reactor Trip On High Steam Generator Level

EVENT DATE (5)			LER NUMBER (6)		REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
Month	Day	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
01	07	21	6	81	4	81	4	--	0   21   0   --   0   0   01   08   21   91   81   41

OPERATING MODE (9) 4 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

POWER LEVEL (10)	0101010	20.402(b)	20.405(a)(1)(i)	20.405(a)(1)(ii)	20.405(a)(1)(iii)	20.405(a)(1)(iv)	20.405(a)(1)(v)	20.405(c)	50.36(c)(1)	50.36(c)(2)	50.73(a)(2)(i)	50.73(a)(2)(ii)	50.73(a)(2)(iii)	X	50.73(a)(2)(iv)	50.73(a)(2)(v)	50.73(a)(2)(vii)	50.73(a)(2)(viii)(A)	50.73(a)(2)(viii)(B)	50.73(a)(2)(x)	73.71(b)	73.71(c)	Other (Specify in Abstract below and in Text, NRC Form 366A)
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Name LICENSEE CONTACT FOR THIS LER (12) Patrick Rogers, Plant Licensing Engineer Telephone Number Area Code 51011916141-1311010

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

Cause	System	Component	Manufacturer	Reportable to NPRDS	Cause	System	Component	Manufacturer	Reportable to NPRDS

SUPPLEMENT REPORT EXPECTED (14) EXPECTED SUBMISSION DATE (15) Month Day Year

Yes (If yes, complete Expected Submission Date)  No

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 7/26/84, at 1139 hours while in Mode 4, an automatic reactor trip occurred due to a high level in 'B' steam generator (2E24B). Heatup was in progress with the steam generators being maintained between 80% and 85% and with reactor coolant system (RCS) temperature at 278° F and RCS pressure at 358 psia. The main steam isolation valves (MSIV) were opened for a valve stroke surveillance. There was approximately a 30 psi differential pressure across the MSIV's at this time. The momentary steam generator pressure decrease resulted in a swell in the steam generator levels. 2E24B level increased enough to reach the high steam generator level trip setpoint thereby actuating a reactor trip. The reactor was not critical at the time of the trip. No operational difficulties or anomalies were noted. To prevent recurrence, plant operating procedure will be revised to include a caution regarding steam generator level control to accommodate swell when opening MSIV's during startup.

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ARKANSAS POWER & LIGHT COMPANY  
POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

August 29, 1984

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U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: Arkansas Nuclear One - Unit 2  
Docket No. 50-368  
License No. NPF-6  
Licensee Event Report  
No. 84-020-00

Gentlemen:

In accordance with 10CFR50.73(a)(2)(iv), attached is the subject report concerning an automatic reactor trip due to a high level in "B" steam generator (2E24B).

Very truly yours,

A handwritten signature in cursive script that reads "John R. Marshall".

John R. Marshall  
Manager, Licensing

JRM:RJS:ac

Attachment

cc: Mr. Richard C. DeYoung  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

Mr. Norman M. Haller, Director  
Office of Management & Program Analysis  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

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