Tech. Spec. 6.9.A.2

Boston Edison
Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier
Vice President Nuclear Operations
and Station Director

February 13, 1996
BECo Ltr. #96-008

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

JANUARY 1996 MONTHLY REPORT

In accordance with Pilgrim Nuclear Power Station Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report, please contact me directly.

L. J. Olivier

L.J. Olivier

RLC/dmc/9458

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

OPERATING DATA REPORT

DOCKET NO. 50-293
NAME: Pilgrim
DATE: February 13, 1996
COMPLETED BY: R. L. Cannon
TELEPHONE: (508) 830-8321
REPORT MONTH January, 1996

OPERATING STATUS

NOTES

1. Unit Name — Pilgrim I
2. Reporting Period — January 1996
3. Licensed Thermal Power (MWt) — 1998
4. Nameplate Rating (Gross MWe) — 678
5. Design Electrical Rating (Net MWe) — 655
6. Maximum Dependable Capacity (Gross MWe) — 696
7. Maximum Dependable Capacity (Net MWe) — 670
8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:

No Changes

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

		This Month	Yr-to-Date	Cumulative
11.	Hours in Reporting Period	744.0	744.0	202896.0
12.	Hours Reactor Critical	744.0	744.0	127010.1
13.	Hours Reactor Reserve Shutdown	0.0	0.0	0.0
14.	Hours Generator On-Line	744.0	744.0	122567.9
15.	Hours Unit Reserve Shutdown	0.0	0.0	0.0
16.	Gross Thermal Energy Generated(MWH)	1382358.0	1382358.0	217684528.0
17.	Gross Electrical Energy Generated(MWH)	476220.0	476220.0	73753544.0
18.	Net Electrical Energy Generated(MWH)	458955.0	458955.0	70902577.0
19.	Unit Service Factor	100.0	100.0	60.4
20.	Unit Availability Factor	100.0	100.0	60.4
21.	Unit Capacity Factor (Using MDC Net)	92.1	92.1	52.2
22.	Unit Capacity Factor (Using DER Net)	94.2	94.2	53.4
23.	Unit Forced Outage Rate	0.0	0.0	11.9
24.	Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each) -	NONE		
25.	If Shutdown at End of Report Period, Estimated Date of Startup -	UNIT OPERATING		

AVERAGE DAILY UNIT POWER LEVEL

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DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	669	17	669
2	669	18	668
3	669	19	667
4	667	20	152
5	666	21	152
6	667	22	304
7	666	23	628
8	666	24	663
9	665	25	623
10	666	26	599
11	667	27	664
12	668	28	664
13	667	29	664
14	668	30	665
15	668	31	665
16	669		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

OPERATIONAL SUMMARY

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The unit started the reporting period at 100 percent core thermal power (CTP) where it was maintained until approximately 2325 hours on January 19, 1996, when reactor power was reduced to approximately 17 percent power to conduct preplanned maintenance, surveillance testing, and a backwash of the main condenser. Following these activities, reactor power was increased and the unit obtained 100 percent CTP at 0959 on January 23, 1996. Power was maintained at 100 percent CTP until January 25, 1996, when reactor power was reduced to approximately 70 percent power to perform a rod pattern change and conduct control rod drive (CRD) exercise surveillances. Following these activities, reactor power was increased and the unit achieved 100 percent CTP at approximately 1223 hours on January 26, 1996, where it was maintained for the remainder of the reporting period.

SAFETY RELIEF VALVE CHALLENGES

MONTH OF JANUARY 1996

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

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The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.

2. Scheduled date for next refueling shutdown: February 1, 1997.

3. Scheduled date for restart following next refueling: March 14, 1997.

4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.

5. See #6.

6. The new fuel loaded during the 1995 refueling outage (RFO-10) is of a different design than that loaded in the previous refueling outage and consists of 136 new fuel assemblies.

7. (a) There are 580 fuel assemblies in the core.

   (b) There are 1765 fuel assemblies in the spent fuel pool.

8. (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2891 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.

   (b) The planned spent fuel storage capacity is 3859 fuel assemblies.

9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1103 fuel assemblies.

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

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SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER

No major safety related maintenance was completed during this reporting period.

UNIT SHUTDOWNS AND POWER REDUCTIONS

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NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
11	01/19/96	S	0.0	H	N/A	N/A	N/A	N/A	Power Reduction to facilitate a backwash of the main condenser and preplanned maintenance.
10	01/25/96	S	0.0	B	N/A	N/A	N/A	N/A	Power Reduction to approximately 70%. CTP for CRD exercise surveillance and rod pattern change.

There were no unit shutdowns during the reporting period.

1	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Operator Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)