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June 11, 1992

C311-92-2082

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for May 1992

Enclosed are two copies of the May 1992 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

T. G. Broughton
Vice President and Director, TMI-1

WGH

Attachments

cc: Administrator, Region I
TMI Senior Resident Inspector

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PDR ADDCK 05000289
R PDR

OPERATIONS SUMMARY
MAY 1992

The unit entered the month operating at 100% power producing 858 MWe and continued full power operation for the entire month of May. On May 21st the annual emergency plan drill was completed successfully. During the month, the Security department completed an investigation of control room and auxiliary operator logging practices. The investigation was initiated as a result of problems identified at several other plants where operators were found falsifying records associated with routine plant log taking rounds. One abnormality resulted in an auxiliary operator being suspended for a period of five days. As a result of irregularities in some Control Room Operator logs, more definitive guidance was provided for documenting late or missed readings.

MAJOR SAFETY RELATED MAINTENANCE

During May, the following major safety related maintenance activities were performed:

Station Blackout Diesel Generator EG-Y-4

The Station Blackout Diesel EG-Y-4 manufacturer, Coltec Industries, identified cracks resulting from mishandling of piston castings by a supplier during removal of casting risers. TMI-1 was listed as having received two pistons of unknown origin. A review of maintenance records identified that the pistons had been installed in the number 2 and 5 upper cylinders of the SBO diesel. Fiber optic inspection of the number 2 and 5 piston skirts verified that these were the suspect pistons. In accordance with the manufacturer's recommendation, the engine was removed from service to replace the pistons. Reassembly and testing will be completed in early June.

Reactor Coolant Pump RC-P-1A

The Reactor Coolant Pump RC-P-1A horizontal shaft vibration probe indicated high vibrations. Upon investigation, it was determined that the horizontal probe was giving erroneous readings and was declared out-of-service. Input is being supplied via a cable connected from the motor frame vibration terminal box located in the relay room to PLF in the control room. Vibration is now analyzed by collecting synchronous time average data from the good vertical shaft probe and the two motor frame vibration probes.

Feedwater Valve FW-V-16A

Main Feedwater Control Valve FW-V-16A was intermittently cycling approximately 10% without a change in demand. Corrective maintenance actions included cleaning the interior of the pilot valve and replacing the following component parts: the Instrument Air in-line filters, IA-V-2217 (a tubing valve) and a summing module in the ICS. FW-V-16A is currently operating as designed but is being monitored for the short term to ensure continued satisfactory operation.

Feedwater Valve FW-V-92B

Feedwater System block valve FW-V-92B limitorque operator 'closed' torque switch setting was increased to ensure that the operational ΔP would be overcome in the event that valve closure would be required. The Limitorque operator spring pack was replaced during the 9R Outage. The 'closed' setting adjustment made after spring pack replacement was based on a calculation which assumed that the feed water regulating valves would be closed. The torque switch setting was increased after reevaluation to provide additional thrust margin without exceeding the valve motor operator limits as recommended by Plant Engineering.

Auxiliary and Fuel Handling Building Fan AH-E-14D

Auxiliary and Fuel Handling Building exhaust fan AH-E-14D motor was removed to repair a motor ground. The fan motor was sent off-site for rewinding and is due back in June.

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE June 11, 1992
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

OPERATING STATUS

- 1. UNIT NAME: THREE MILE ISLAND UNIT 1
- 2. REPORTING PERIOD: MAY 1992
- 3. LICENSED THERMAL POWER: 2568
- 4. NAMEPLATE RATING (GROSS MWe): 871
- 5. DESIGN ELECTRICAL RATING (NET MWe): 819
- 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
- 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

- 8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____
- 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____
- 10. REASONS FOR RESTRICTIONS, IF ANY: _____

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	3647.0	155568.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	3647.0	80378.0
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	2245.6
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	3647.0	79303.2
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1908127	9353272	193222318
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	632116	3122103	65116362
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	596962	2947926	61105251
19. UNIT SERVICE FACTOR	(%)	100.0	100.0	51.0
20. UNIT AVAILABILITY FACTOR	(%)	100.0	100.0	51.0
21. UNIT CAPACITY FACTOR (USING MDC NET)		102.1	102.8	50.0
22. UNIT CAPACITY FACTOR (USING DER NET)		98.0	98.7	48.0
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	43.3
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60648.7
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

- 25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE June 11, 1992
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

MONTH: MAY

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
1	805	17	800
2	794	18	796
3	796	19	801
4	806	20	804
5	809	21	802
6	809	22	798
7	809	23	794
8	806	24	800
9	803	25	809
10	802	26	806
11	798	27	807
12	802	28	807
13	798	29	805
14	798	30	805
15	802	31	798
16	801		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH May 1992

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE June 11, 1992
 COMPLETED BY W. G. Heysek
 TELEPHONE (717) 948-8191

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report#	System Code " & "	Component Code " & "	Cause & Corrective Action to Prevent Recurrence
						NONE			

1
 F Forced
 S Scheduled

2
 Reason
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensing Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

4
 Exhibit G - Instructions for preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)
 5 Exhibit 1 same source
 6 Actually used exhibits F & II NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: Three Mile Island Nuclear Station, Unit 1
2. Scheduled date for next refueling shutdown: September 17, 1993 (10R)
3. Scheduled date for restart following current refueling: NA
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? NA

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

None planned.

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

GPU Nuclear has installed four Westinghouse Lead Test Assemblies during the reload of the TMI-1 core for cycle 9 operation. Westinghouse fuel technology will be utilized to the extent possible.

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 521

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1496. Reracking of spent fuel pool 'A' to attain the licensed capacity is in progress.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

The 9R (1991) refueling discharge was the last to allow full core off-load capacity (177 fuel assemblies).