



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 14, 1996

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 OPUS Place, Suite 500
Downers Grove, IL 60515

50-254

SUBJECT: EVALUATION OF THE AUGMENTED EXAMINATION OF THE REACTOR PRESSURE VESSEL SHELL WELDS PURSUANT TO 10 CFR 50.55a(g)(6)(ii)(A) FOR QUAD CITIES NUCLEAR POWER STATION, UNIT 1 (TAC NO. M94452)

Dear Mr. Farrar:

The staff has reviewed the information provided by Commonwealth Edison Company (ComEd, the licensee) in its letter dated January 4, 1996, related to the augmented examination of the reactor pressure vessel shell welds pursuant to 10 CFR 50.55a(g)(6)(ii)(A) for Quad Cities Nuclear Power Station, Unit 1. Our evaluation follows.

In its letter dated January 4, 1996, the licensee proposed that the augmented reactor pressure vessel shell weld (American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, Category B-A, Item B1.10 shell welds) examinations be ultrasonically examined (a) remotely from the inside of the reactor vessel to the extent possible, and (b) manually from the outside of the vessel where remote inside examination is not achieved and where scheduled bioshield wall blocks and insulation materials disassembly allows access.

In addition, the licensee proposed that the remote examinations will be performed in accordance with the remote ultrasonic examination procedure developed by General Electric, which has been demonstrated at the Performance Demonstration Initiative Qualification in accordance with the rules of Appendix VIII of the ASME Code, Section XI, 1992 Edition through 1993 Addenda. Based on the results of the Quad Cities, Unit 1, reactor vessel accessibility study performed by the licensee, it concluded that the examination coverage from the inside of the vessel would be approximately 71 percent of the reactor pressure vessel shell weld volume. Furthermore, as noted above, the licensee will supplement the remote internal examinations with manual ultrasonic examinations from the outside surface of the reactor pressure vessel to the extent possible.

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D. L. Farrar

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The staff believes that the licensee's proposed examination plan to meet the augmented reactor pressure vessel shell weld inspection requirements contained in 10 CFR 50.55a(g)(6)(ii)(A) is appropriate. Upon completion of the augmented reactor vessel shell welds examinations, if the examination volume for each weld examined is less than 90 percent the licensee is to submit to the NRC an alternative to its augmented reactor vessel shell weld examinations pursuant to 10 CFR 50.55a(g)(6)(ii)(A).

Sincerely,



Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-254

cc: See next page

D. L. Farrar
Commonwealth Edison Company

Quad Cities Nuclear Power Station
Unit Nos. 1 and 2

cc:

Michael I. Miller, Esquire
Sidley and Austin
One First National Plaza
Chicago, Illinois 60603

Document Control Desk-Licensing
Commonwealth Edison Company
1400 Opus Place, Suite 400
Downers Grove, Illinois 60515

Mr. L. William Pearce
Station Manager
Quad Cities Nuclear Power Station
22710 206th Avenue North
Cordova, Illinois 61242

U.S. Nuclear Regulatory Commission
Quad Cities Resident Inspectors Office
22712 206th Avenue North
Cordova, Illinois 61242

Chairman
Rock Island County Board
of Supervisors
1504 3rd Avenue
Rock Island County Office Bldg.
Rock Island, Illinois 61201

Illinois Department of Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704

Regional Administrator
U.S. NRC, Region III
801 Warrenville Road
Lisle, Illinois 60532-4351

Richard J. Singer
Manager - Nuclear
MidAmerican Energy Company
907 Walnut Street
P.O. Box 657
Des Moines, Iowa 50303

Brent E. Gale, Esq.
Vice President - Law and
Regulatory Affairs
MidAmerican Energy Company
One RiverCenter Place
106 East Second Street
P.O. Box 4350
Davenport, Iowa 52808

The staff believes that the licensee's proposed examination plan to meet the augmented reactor pressure vessel shell weld inspection requirements contained in 10 CFR 50.55a(g)(6)(ii)(A) is appropriate. Upon completion of the augmented reactor vessel shell welds examinations, if the examination volume for each weld examined is less than 90 percent the licensee is to submit to the NRC an alternative to its augmented reactor vessel shell weld examinations pursuant to 10 CFR 50.55a(g)(6)(ii)(A).

Sincerely,

Original signed by:

Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

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cc: See next page

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