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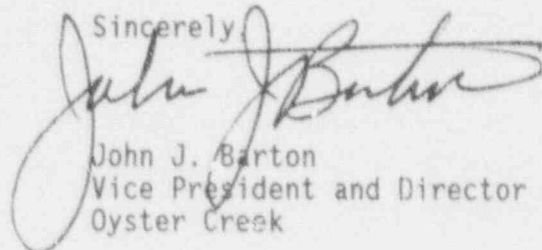
U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report

This letter forwards one (1) copy of Licensee Event Report 92-006.

Sincerely,



John J. Barton
Vice President and Director
Oyster Creek

JJB/BDEM:jc
Enclosure

cc: Administrator, Region 1
Senior NRC Resident Inspector
Oyster Creek NRC Project Manager

(LER-COVLTRS)

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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1): Oyster Creek
DOCKET NUMBER (2): 0 5 0 0 0 2 1 9 1 OF 13

TITLE (4): Electromatic Relief Valve High Pressure Relief Setpoint Exceeded
Technical Specification Limit Due to Drift

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
05	10	92	92	006	000	06	03	92			0 5 0 0 0
<p>THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)</p>											

OPERATING MODE (8)	20.402(b)	20.406(e)	50.73(a)(2)(v)	73.71(b)
POWER LEVEL (10): 110.0	20.402(a)(1)(i)	20.406(a)(1)	50.73(a)(2)(v)	73.71(a)
	20.402(a)(1)(ii)	20.406(a)(2)	50.73(a)(2)(v)	
	20.406(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(v)(i)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(v)(ii)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(v)(iii)	

LICENSEE CONTACT FOR THIS LER (12):
NAME: P. Cervenka
TELEPHONE NUMBER: AREA CODE 609, NUMBER 971-4894

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14):
YES () / NO (X)
EXPECTED SUBMISSION DATE (15):

ABSTRACT (16):

On May 10, 1992 while performing an Electromatic Relief Valve (EMRV) Pressure Sensor surveillance, the "As Found" trip setpoint for the high pressure relief function on one EMRV was above that specified in the Technical Specifications. The cause of this occurrence is attributed to setpoint repeatability and instrument drift. The design setpoint repeatability can tolerate instrument drift up to 2.5 psig of the Technical Specification limit. Previous surveillance records indicate that these instruments frequently undergo additional drift within Technical Specification limits due to changing plant and environmental conditions. This occurrence is considered to have minimal safety significance as the automatic depressurization function of the EMRVs is not affected by these pressure switches, all five EMRVs would have actuated to relieve pressure, and the Isolation Condenser System and turbine bypass valves were fully operable. The pressure switch was adjusted to actuate within the Technical Specification limit. A new pressure sensing system is to be installed in accordance with the Oyster Creek Integrated Schedule, which currently specifies the Cycle 15 refueling outage for completion of this project.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Oyster Creek	DOCKET NUMBER (2) 0 15 10 10 10 12 1 19 9 1 2	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		01	06	010	012	OF 013

TEXT (if more space is required, use additional NRC Form 308A's) (17)

DATE OF DISCOVERY

The condition described within this report was discovered on May 10, 1992.

IDENTIFICATION OF OCCURRENCE

During the performance of an Electromatic Relief Valve (EIRC RV) Pressure Sensor Test and Calibration surveillance, one relief valve had a high pressure relief setpoint above that specified in the Plant Technical Specifications. This event is considered to be reportable as defined in 10 CFR.73(a)(2)(i)(B).

CONDITIONS PRIOR TO DISCOVERY

The reactor was at approximately 100% power, with a generator load of approximately 650 megawatts electric.

DESCRIPTION OF OCCURRENCE

On May 10, 1992 at approximately 2200 hours, while performing the Electromatic Relief Valve (EMRV) Pressure Sensor Test and Calibration surveillance, the "As Found" trip setpoint for the high pressure relief function on one EMRV was above that specified in the Technical Specifications. The technical specification limit for EMRV IA83A pressure sensor is 1079.1 psig. Sensor IA83A test results determined the setpoint was at 1081.0 psig.

APPARENT CAUSE OF OCCURRENCE

The cause of this occurrence is attributed to instrument setpoint repeatability and instrument drift due to changing plant and environmental conditions. The tolerance between the "As Left" instrument setpoint and the Technical Specification is 10 psig. The design accuracy for these sensors is ± 7.5 psig. Therefore, design setpoint repeatability allows the "As Found" value to be within 2.5 psig of the Technical Specification limit.

A review of previous surveillance records indicated that these instruments frequently undergo additional drift during changes in plant pressure and operational conditions. A 2.5 psig additional drift beyond the 7.5 design tolerance would cause the instrument setpoint to exceed the Technical Specification limit. Instrument repeatability and drift cannot be improved due to the design of the switch used in this application.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 2150-0104

EXPIRES 8/31/85

NAME (1) Oyster Creek	DUCKET NUMBER (2) 0151010101219	LER NUMBER (6)			PAGE (3)	
		YEAR 1 92	SEQUENTIAL NUMBER 006	REVISION NUMBER 00	03	OF 03

TEXT IF MORE SPACE IS REQUIRED, USE SEPARATE NRC Form 3054 (1/77)

ANALYSIS OF OCCURRENCE AND SAFETY SIGNIFICANCE

The EMRVs are part of the Automatic Depressurization System (ADS). This system is designed to depressurize the Reactor Coolant System (EISS AB) during small break Loss-of-Coolant Accident conditions so that the low pressure Core Spray System (EISS BM) can inject. The ADS function of the EMRVs is not affected by these switches. The EMRVs also provide a high pressure relief function for the reactor pressure vessel. Together with the high pressure scram function and the Isolation Condenser System (EISS BL), the EMRVs limit pressure in the Reactor Coolant System during high pressure transients to prevent exceeding the system pressure safety limit of 1375 psig.

This occurrence is considered to have minimal safety significance since the ADS function of the EMRVs is not affected by these pressure switches, all five EMRVs would have actuated to relieve pressure, and the Isolation Condenser System and turbine bypass valves were fully operable. In addition, the reactor safety valves are designed to prevent reaching the Reactor Coolant System pressure safety limit of 1375 psig on a complete loss of EMRV relief capability.

CORRECTIVE ACTION

The pressure switch was immediately adjusted to actuate within the Technical Specification limit. A new pressure sensing system has been selected to replace the present switch used in this application. The sensors selected will have an accuracy that will improve setpoint repeatability. This modification is currently included in the Oyster Creek Integrated Schedule as a Cycle 15 refueling outage project.

SIMILAR EVENTS

- LER 81-40 EMRV Pressure Sensor Test and Calibration
- LER 81-51 EMRV High Pressure Sensor
- LER 81-57 Reactor High Pressure Switch "B" EMRV
- LER 82-24 EMRV Switch Out of Tech Spec Limit
- LER 90-10 EMRV High Pressure Relief Setpoints Exceeded Tech Spec Limit Due to Drift