



Commonwealth Edison
LaSalle County Nuclear Station
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Telephone 815/357-6761

June 10, 1992

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for May, 1992.

Very truly yours,

for G. J. Diederich
Station Manager
LaSalle County Station

GJD/MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MAY 1992

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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(UNIT 1)

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I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 850 Mwe.
	0600	Increased power level to 1000 Mwe (power level held due to the 'B' Turbine Driven Reactor Feed Pump out of service).
2	1400	Increased power level to 1125 Mwe.
3	0130	Reduced power level to 950 Mwe due to system load.
	2330	Reduced power level to 850 Mwe due to system load.
4	1200	Increased power level to 1120 Mwe.
5	0100	Reduced power level to 850 Mwe to perform weekly and monthly surveillances.
	1100	Increased power level to 1125 Mwe.
7	0100	Reduced power level to 1085 Mwe due to the 'B' Reactor Recirculation Flow Control Valve drifting.
	1130	Increased power level to 1120 Mwe.
10	0400	Reduced power level to 1085 Mwe due to the 'B' Reactor Recirculation Flow Control Valve drifting.
	1000	Increased power level to 1110 Mwe.
12	0100	Reduced power level to 800 Mwe for rod pattern adjustment.
	1200	Increased power level to 1118 Mwe.
13	0030	Reduced power level to 1000 Mwe to transfer the Condensate/Condensate Booster Pumps.
	0900	Increased power level to 1115 Mwe.
14	0100	Reduced power level to 850 Mwe due to system load.
	0800	Increased power level to 1080 Mwe.
15	0200	Reduced power level to 850 Mwe due to system load.
	0500	Increased power level to 1090 Mwe.
17	1400	Increased power level to 1115 Mwe.

II. MONTHLY REPORT (CONTINUED)

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
18	0100	Reduced power level to 950 Mwe due to system load.
	0300	Increased power level to 1090 Mwe.
	2300	Reduced power level to 850 Mwe due to system load.
19	1100	Increased power level to 1100 Mwe.
20	1800	Reduced power level to 1000 Mwe for Main Turbine performance test.
	2200	Increased power level to 1100 Mwe.
22	0200	Reduced power level to 1000 Mwe for Main Turbine performance test.
	0600	Increased power level to 1120 Mwe.
	1700	Reduced power level to 1000 Mwe due to cooling problems with the Main Power Transformer.
23	0630	Increased power level to 1115 Mwe.
25	1800	Reduced power level to 1075 Mwe to transfer the Condensate/Condensate Booster Pumps.
26	0800	Increased power level to 1100 Mwe.
	0900	Reduced power level to 1000 Mwe due to cooling problems with the Main Power Transformer.
	2200	Increased power level to 1125 Mwe.
27	2330	Reduced power level to 900 Mwe for scheduled maintenance on the '1B' Turbine Driven Reactor Feed Pump.
28	0430	Increased power level to 950 Mwe.
	0730	Increased power level to 1015 Mwe (power level held due to the '1B' Turbine Driven Reactor Feed Pump off-line).
31	0000	Reduced power level to 960 Mwe to place the '1B' Turbine Driven Reactor Feed Pump back in service.
	1300	Increased power level to 1125 Mwe.
	2300	Reduced power level to 850 Mwe to perform scram timing and rod set.
	2400	Reactor critical, Generator on-line at 850 Mwe.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

Addition of refuel platform hoist interlock setpoints to Technical Specification 4.9.6 for the cylindrical mast modification.

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORT (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
92-007-00	05/31/92	Auto-start of the 'B' Control Room/Auxiliary Electric Equipment Room Ventilation Emergency Make-up Train.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L14435	Electrical Switchgear Heat Removal System Fan 1VX06C.	Alarm relay.	Improper indication.	Replaced alarm relay.
L15077	Low Power Range Monitor 32-17B	Low power range monitor card.	No downscale indication.	Replaced low power range monitor card.
L15345	Control Room Ventilation Radiation Monitor Recorder 1D18-R800	Servo motor.	Unable to obtain accurate readings.	Replaced servo motor.
L15525	Reactor Building Equipment Drain Valve 1RE026	Air Solenoid.	Valve failed to close.	Replaced solenoid.
L15762	Control Room Ventilation System Ammonia Detector OXY-VC125B	Tape drive.	Tape not advancing.	Replaced tape drive motor.
L15819	Residual Heat Removal Pump Seal Cooler 1E12-C002B	Baseplate gasket leakage.	None.	Replaced baseplate gasket.
L15820	Residual Heat Removal Pump Seal Cooler 1E12-C002C	Baseplate gasket leakage.	None.	Replaced baseplate gasket.
L15898	Average Power Range Monitor Channel 'A'	Upscale alarm relay.	Abnormal alarms.	Replaced relay.

(No SOR Failures this month.)
ZCADTS/5+

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE June 10, 1992
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD: May 1992
GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3,323
MAX DEPENDABLE CAPACITY (Mwe-Net): 1,036
DESIGN ELECTRICAL RATING (Mwe-Net): 1,078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY): N/A
4. REASON FOR RESTRICTION (IF ANY):

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
	-----	-----	-----
5. REACTOR CRITICAL TIME (HOURS)	744.0	3,585.5	50,942.0
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	744.0	3,552.8	49,931.1
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWh _t)	2,360,801	11,459,638	146,616,617
10. ELECTRICAL ENERGY GENERATED (MWh _e -Gross)	796,712	3,904,944	48,994,716
11. ELECTRICAL ENERGY GENERATED (MWh _e -Net)	769,848	3,783,919	46,979,064
12. REACTOR SERVICE FACTOR (%)	100.0	98.3	69.0
13. REACTOR AVAILABILITY FACTOR (%)	100.0	98.3	71.3
14. UNIT SERVICE FACTOR (%)	100.0	97.4	67.7
15. UNIT AVAILABILITY FACTOR (%)	100.0	97.4	67.7
16. UNIT CAPACITY FACTOR (USING MDC) (%)	99.9	100.1	61.4
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	96.0	96.2	59.1
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	2.6	7.3
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):	Refuel Outage, 09/26/92, 12 Weeks		
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE June 10, 1992
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

REPORTING PERIOD: May 1992

DAY	POWER	DAY	POWER
---	-----	---	-----
1	928	17	1,035
2	1,032	18	1,033
3	931	19	996
4	1,022	20	1,066
5	1,019	21	1,076
6	1,088	22	1,044
7	1,075	23	1,073
8	1,078	24	1,083
9	1,078	25	1,074
10	1,075	26	1,034
11	1,075	27	1,077
12	1,017	28	957
13	1,049	29	988
14	1,008	30	986
15	1,021	31	1,021
16	1,038		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
(None.)						

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system loading and maintenance activities.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations
(None.)
2. ECCS System Outages
(See Table 5)
3. Changes to the Off-Site Dose Calculation Manual
(None.)
4. Major changes to Radioactive Waste Treatment Systems.
(None.)
5. Indications of Failed Fuel Elements.
(None.)

(Unit 1)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
(U-0)		
0394	ODG08CA	Air compressor inspection.
0395	ODG08CB	Air compressor inspection.
0397	ODG01P	Coupling lubrication.
0398	ODG01K	Change soak back oil filters.
0399	ODG01K	Transducer calibration.
0400	ODG01K	Rewire annunciator logic.
0402	ODG01P	Replace feed breakers.
(U-1)		
0319	1E51-C001	Coupling lubrication.
0320	1E51-F010	VOTES testing.
0321 0336	1E51-C003	Motor replacement
0340	1E22-C302A	Inspection.
0344	1E22-C302B	Inspection.
0345	1E22-S001	Conduit repair.
0355	1E22-C302A	Repair air dryer.
0365	1E12-C002B	Relay calibration.
0366	1E12-C002C	Relay calibration.
0370	1E12-C002B	Megger motor.
0371	1E12-C002C	Megger motor.
0372	1E12-F040B	EQ inspection.
0373	1E12-C300C	Megger motor.

(Unit 1)
Table 5 (Continued)

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

0374	1E12-C3000	Megger motor.
0381	1DG01K	Breaker inspection.
0385	1DG01K	Transducer calibrations.
0395	1E12-C002B 1E12-C002C	Repair seal leakage.
0415	1E51-C004 1E51-C005	Motor inspection.

LASALLE NUCLEAR POWER STATION
UNIT 2
MONTHLY PERFORMANCE REPORT
MAY 1992
COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 2. Average Daily Unit Power Level
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- F. UNIQUE REPORTING REQUIREMENTS
 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System
 - 5. Indications of Failed Fuel Elements

I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1000 Mwe (power level held due to the '2A' Turbine Driven Reactor Feed Pump being off-line).
2	0400	Increased power level to 1135 Mwe.
	1900	Reduced power level to 1070 Mwe due to low discharge pressure of the Condensate pump.
3	0300	Increased power level to 1135 Mwe.
7	0200	Reduced power level to 940 Mwe due to system load.
	1200	Increased power level to 1135 Mwe.
15	2100	Reduced power level to 545 Mwe due to Feedwater Heater high level, placed the '2B' Turbine Driven Reactor Feed Pump off-line.
	2300	Increased power level to 702 Mwe.
16	2100	Increased power level to 993 Mwe.
18	1400	Increased power level to 1130 Mwe.
19	0100	Reduced power level to 980 Mwe due to system load.
	1100	Increased power level to 1135 Mwe.
20	0530	Reduced power level to 950 Mwe to place the Circulating Water pump off-line.
	1130	Increased power level to 1125 Mwe.
23	1800	Reduced power level to 950 Mwe due to system load.
	2330	Reduced power level to 850 Mwe due to system load.
24	1100	Increased power level to 970 Mwe.
25	0100	Reduced power level to 850 Mwe due to system load.
	1500	Increased power level to 1100 Mwe (power level held due to high vibration on the #11 Main Turbine bearing).
26	0500	Reduced power level to 960 Mwe due to system load.
	0900	Increased power level to 1115 Mwe.
27	0100	Reduced power level to 1060 Mwe due to system load.
	1400	Increased power level to 1115 Mwe.
31	2400	Reactor critical, Generator on-line at 1115 Mwe.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

Addition of refuel platform hoist interlock setpoints to Technical Specification 4.9.6 for the cylindrical mast modification.

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)
LER Number Date Description

(None.)

E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.
(See Table 2)
2. Average Daily Unit Power Level.
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.
(See Table 4)

C. 7. LE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST NUMBER</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L11672	Auxiliary Electric Equipment Room Ventilation Compressor OVE04CB	Oil temperature switch.	Incorrect temperature indication.	Replaced temperature switch.
L15423	Average Power Range Monitor Channel 'E'	Thermal trip unit card.	Thermal trip unit failed calibration.	Replaced thermal trip unit card.
L15580	Control Room Ventilation Radiation Recorder	Ink wheel capacitor.	Slow indication response.	Replaced capacitor.
L15765	Main Steam Line Leak Detection High Flow Differential Pressure Switch 2E31-N011A	Setpoint drift.	Switch tripped below rejection limit.	Replaced switch.

(See attached SOR Failure Report.)

SOR dp SWITCH FAILURE DATA SHEET

Equipment Piece Number: 2E31-N011A

Model Number: 102-AS-B305-NX-JJTTX6

Serial Number: 85-1-2449

Application: Main Steam High Flow Isolation Differential Pressure

Date and Time of Discovery: 05/19/92 0200 hours

Reactor Mode: 1 (Run) Power Level: 100%

Calibration Tolerance: 102.0 - 104.0 psid

Nominal Setpoint: 103.0 psid

Action Limits: < 98.2 or > 107.9 psid

Reject Limits: < 94.8 or > 111.2 psid

Technical Specification

Limits: 116.0 +/- 0.0 psid

As Found Setpoint: 91.0 psid

Date and Time of Return to Service: 05/22/92 0415 hours

Model Number of Replacement Switch: 102-AS-B403-NX-JJTTX6

Serial Number of Replacement Switch: 86-10-560

DVR Number: 1-2-92-056

Cause: Switch was found out of calibration. Apparent cause is instrument setpoint drift.

Corrective Action: The switch was replaced.

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE June 10, 1992
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

- | | |
|--|----------|
| 1. REPORTING PERIOD: | May 1992 |
| GROSS HOURS IN REPORTING PERIOD: | 744 |
| 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): | 3,323 |
| MAX DEPENDABLE CAPACITY (Mwe-Net): | 1,036 |
| DESIGN ELECTRICAL RATING (Mwe-Net): | 1,078 |
| 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY): | N/A |
| 4. REASON FOR RESTRICTION (IF ANY): | |

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
	-----	-----	-----
5. REACTOR CRITICAL TIME (HOURS)	744.0	1,297.2	46,233.1
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	744.0	1,135.4	45,339.9
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWht)	2,375,735	3,224,677	135,467,076
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	806,618	1,093,624	44,983,374
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	779,823	1,026,568	43,158,072
12. REACTOR SERVICE FACTOR (%)	100.0	35.6	69.2
13. REACTOR AVAILABILITY FACTOR (%)	100.0	35.6	71.8
14. UNIT SERVICE FACTOR (%)	100.0	31.1	67.9
15. UNIT AVAILABILITY FACTOR (%)	100.0	31.1	67.9
16. UNIT CAPACITY FACTOR (USING MDC) (%)	101.2	27.2	62.4
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	97.2	26.1	59.9
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	8.2	13.0
19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):			
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE June 10, 1992
COMPLETED BY M. CIALKOWSKI
TELEPHONE (815) 357-6761

REPORTING PERIOD: May 1992

DAY	POWER	DAY	POWER
---	-----	---	-----
1	997	17	946
2	1,079	18	1,035
3	1,092	19	1,045
4	1,092	20	1,064
5	1,096	21	1,075
6	1,094	22	1,081
7	1,043	23	1,026
8	1,088	24	901
9	1,088	25	973
10	1,090	26	1,039
11	1,094	27	1,059
12	1,087	28	1,065
13	1,086	29	1,067
14	1,088	30	1,071
15	987	31	1,071
16	879		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%
(UNIT 2)

<u>YEARLY SEQUENTIAL DATE NUMBER</u>	<u>DATE (YYMMDD)</u>	<u>TYPE F: FORCED S: SCHEDULED</u>	<u>DURATION (HOURS)</u>	<u>REASON</u>	<u>METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER</u>	<u>CORRECTIVE ACTIONS/COMMENT: (LER/DVR # if applicable)</u>
06	920615	F	0.0	H	4	Power level reduced due to feedwater heater high level.

SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required due to system loading, maintenance activities, and a feedwater heater transient.

F. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

(None.)

2. ECCS System Outages

(See Table 5.)

3. Changes to the Off-Site Dose Calculation Manual.

(None.)

4. Major changes to Radioactive Waste Treatment Systems.

(None.)

5. Indications of Failed Fuel Elements.

(None.)

(Unit 2)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
1702 1703	2DG08DA	Repair air dryer compressor.
1732 1733	2DG09DA	Replace air dryer compressor.
1735	2E22-S001	Replace cylinder thermocouples.
1744	2DG09DB	Recalibrate temperature switches.
1754	2E51-C003	Coupling lubrication.