OWNER'S DATA REPORT

FOR

INSERVICE INSPECTION

EDWIN I. HATCH NUCLEAR PLANT

UNIT 2

SEPTEMBER 1995 - NOVEMBER 1995

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NOTE: Portions of this report are compiled from Southern Nuclear Operating Company report; "Nondestructive Examination of Selected Class 1, 2, and 3 Components", issued for the Fall 1995 Refueling Outage at E. I. Hatch Nuclear Plant, Unit 2. This report is available for review in the Records Management Department at the plant site.

LIST OF ABBREVIATIONS

ANI/ANII Authorized Nuclear Inspector/Authorized Nuclear Inservice Inspector

ASME American Society of Mechanical Engineers
ASNT American Society of Nondestructive Testing

BC Branch Connection
BWR Boiling Water Reactor

CH Closure Head CONT Containment

CPI Containment Purge and Inerting

CRD Control Rod Drive

CS Core Spray
CU Clean-up

C&L Cramer and Lindell
DCR Design Change Request

ECCS Emergency Core Cooling Systems
ET Eddy Current Examination

EPRI Electric Power Research Institute

FB Flange Bolting
FPC Fuel Pool Cooling

FW Feedwater

GE General Electric

GPC Georgia Power Company

HL Hanger Lug

HPCI High Pressure Coolant Injection INF Indication Notification Form

IGSCC Intergranular Stress Corrosion Cracking

ISI Inservice Inspection

LD-I Longitudinal Seam Weld Extending Downstream
LD-I Longitudinal Weld Downstream on Inside of Elbow
LD-O Longitudinal Weld Downstream on Outside of Elbow

Lo Zero Reference Location

LMT Lambert, MacGill, Thomas, Inc.
LPCI Low Pressure Coolant Injection

LU-I Longitudinal Seam Weld Extending Upstream
LU-I Longitudinal Weld Upstream on Inside of Elbow
LU-O Longitudinal Weld Upstream on Outside of Elbow

MSIV Main Steam Isolation Valve

MS Main Steam

MSA Main Steam Auxiliary

MT Magnetic Particle Examination

MWe Megawatt Electric

MWO Maintenance Work Order

MWt Megawatt Thermal

NDE Nondestructive Examination

Abbreviations - cont.

NI No Indication

NRC Nuclear Regulatory Commission

OL Overlay
PL Pipe Lug
PLT Plant

PR Pipe Restraint
PROD Product
PS Pipe Support
PSW Plant Service Water

PT Liquid Penetrant Examination

QC Quality Control
RC Reactor Recirculation

RCIC Reactor Core Isolation Cooling

RHR Residual Heat Removal

RHRSW Residual Heat Removal Service Water

RI Recordable Indication

RINTSA Recirculation Inlet Nozzle Thermal Sleeve Attachment

RL Refracted Longitudinal

RL Restraint Lug

RPV Reactor Pressure Vessel

RX Reactor

RWCU Reactor Water Cleanup SBLC Standby Liquid Control

SIAI Structural Integrity Associates, Inc.

SER Service

SRV Safety Relief Valve

SNC Southern Nuclear Operating Company

TBP Turbine Bypass

TDP Torus Drainage and Purification

UT Ultrasonic Examination

VLV Valve

VT Visual Examination

This list is comprised of standard abbreviations used in Inservice Inspection Documentation. All of these abbreviations may not appear in this report.

FORM NIS-1 OWNERS' DATA REPORT FOR INSERVICE INSPECTIONS As Required By the Provisions of the ASME Codes Rules

- Owner Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
- 2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
- 3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
- 5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A
- 7. Components Inspected:

Component of Appurtenance or System	Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Province Number	National Board No.
Rx. Pressure Vessel	Combustion Eng.	70101	N/A	11570
Rx. Pressure Vessel	Combustion Eng.	70101	N/A	11570
2B21 Feedwater	Pullman Power Prod.	*	N/A	N/A
2B31 Reactor Recirc.	Pullman Power Prod.	*	N/A	N/A
2C11 CRD	Pullman Power Prod.	*	N/A	N/A
2E11 RHR	Pullman Power Prod.	*	N/A	N/A
2E21 CS	Pullman Power Prod.	*	N/A	N/A
2E41 HPCI	Pullman Power Prod.	*	N/A	N/A
2E51 RCIC	Pullman Power Prod.	*	N/A	N/A
2G31 RWCU	Pullman Power Prod.	*	N/A	N/A
2N11 MS Auxiliary	Pullman Power Prod.	*	N/A	N/A
2N21 FW	Pullman Power Prod.	*	N/A	N/A
2T47 Primary Cont. Clg.	Pullman Power Prod.	*	N/A	N/A
2T48 Cont. Purge	Pullman Power Prod.	*	N/A	N/A

Note: Supplemental sheets in form of lists, sketches, or drawings may be used provided (1) size is 8 1/2 in. x 11 in., (2) information in items 1 through 6 on this data report is included on each sheet, and (3) each sheet is numbered and the number of sheets are recorded at the top of this form.

^{*} Spool piece or fitting numbers too numerous to list for each specific system. Material certifications for all piping, fittings, etc., are available for review in the Records Management Department at the plant site.

FORM NIS-1 (Back)

- Examination Dates 04/24/94 to 11/19/95.
- 9. Inspection Interval from 01/86 to 01/96.
- Abstract of Examination. Include a list of examinations and a statement concerning status of 10. work required for current interval. ***
- Abstract of Conditions Noted. *** 11.
- Abstract of Corrective Measures Recommended and Taken. *** 12

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date Jan 31, 1996 Signed Georgia Power Company By CAMOONE

Certificate of Authorization No. (if applicable) N/A

Expiration Date N/A

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of GA and employed by **** of Hartford, CT have inspected the components described in this Owner's Data Report during the period 4/94 to 11/95 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owners' Data Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owners' Date Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date 31-JAN 1996

DR Jan 200 Commissions GEORGIA - GA 00115

Inspector's signature

National Board, State, Province, & No.

The following NIS-1 Form supplementary information and report includes the responses to NIS guestions #10, #11, and #12.

Hartford Steam Boiler Inspection and Insurance Company.

Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302
 Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, QA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

NIS-1 Form Supplementary Information

Owner's Data Report

for

Inservice Inspection

Owner Name & Address:

Georgia Power Company 333 Piedmont Avenue, N.E.

P. O. Box 4545

Atlanta, Georgia 30302

Name & Address of Nuclear Generating Plant:

Edwin I. Hatch Nuclear Plant

Route 1, Box 278 Baxley, Georgia 31513

Name Assigned to Nuclear Plant Station:

Edwin I. Hatch Nuclear Plant

Unit 2

Commercial Service Date:

September 5, 1979

Gross Generating Capacity:

2558 MWt, 850 MWe

State, province, or Municipality Assigned Number:

N/A

National Board Number Assigned by manufacturer:

N/A

System Pressure/Leakage Tests

System Reactor Pressure Vessel and Associated Class 1 Piping and Class

Test Required 1 Leakage

Components

- 1. Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
- Plant: Edwin I. Hatch Nuclear Plant, Route 1. Box 278, Baxley, GA 31513.
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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

Name & address of Manufacturer of Components: 1.

Reactor Pressure Vessel and Closure Head: Combustion Engineering, Inc. Chattanooga, TN

- 2. Piping (Classes 1, 2, and 3)
 - Pullman Power Products Division of Pullman-Kellogg Williamsport, PA
 - b. General Electric Company San Jose, CA

Note: Piping purchased by General Electric and Pullman and installed by Pullman. Material certifications and manufacturer information are available for review in the Records Management Department at the Hatch Plant Site.

- 3. Piping Supports and Hangers (Classes 1, 2, and 3)
 - Bergen-Paterson Pipe Support Corporation Laconia, NH
 - b. E-Systems Enertech Orange, CA
 - C. Pacific Scientific Anaheim, CA
 - d. Liesega USA Laconia, NH
- Valves, Pumps, and Heat Exchangers 4.
 - Byron-Jackson, Inc. Los Angeles, CA
 - b. Crane New York, NY
 - C. Wm. Powell Company Cincinnati, OH
 - d. General Electric San Jose, CA

Date of Inservice Inspection:

September 1995 to November 1995

Owner: Georgia Power Company, 3, 1 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
 Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

Completion Date of Inservice Inspection:

November 19, 1995

Name of Inspector:

Donald R. Laakso (ANI/ANII)

Name & Mailing Address of Inspector's Employer:

The Hartford Steam Boiler Inspection and Insurance Company 200 Ashford Center - North Suite 300 Atlanta, Georgia 30338

- 1. Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
- Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
- 3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
- 5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

ABSTRACT

An Inservice Inspection of selected Class 1, 2, and 3 components at Georgia Power Company's Edwin I. Hatch Nuclear Plant Unit 2 was performed during the Fall 1995 Maintenance/Refueling Outage. The components were examined in accordance with the applicable ISI Outage Plan, including any changes made during the outage as approved by GPC.

Edwin I. Hatch Unit 2 is currently in the third period of the second 10-Year Inspection Interval. The required examinations are presently on schedule as specified in the Second Ten-Year Inspection Plan.

The Nondestructive examinations were performed using VT, PT, MT, and UT examination techniques. SNC and GPC personnel and their contractor GE, performed NDE of the selected welds and components. In addition, GE personnel performed the VT examination of selected RPV internal components. SNC, GE, or GPC NDE procedures were utilized for all ASME Section XI examinations. GE personnel were qualified to the applicable SNC procedures. EPRI qualified inspectors were utilized for all examinations involving IGSCC susceptible materials. GE procedures were used for mechanized ultrasonic examination and exams were performed by GE inspectors.

In addition to NDE testing of Class 1 and 2 welds and components, pressure testing, visual examination of Class 1 component internal surfaces and visual examination of pipe supports and hangers were also performed. Third party review (e.g. an ANII) was utilized for all examinations of ASME Section XI components.

Selected components were examined in accordance with GPC commitments to the following documents:

- Section XI of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, "Rules for Inservice Inspection of Nuclear Power Plant Components," 1980 Edition with Addenda through Winter 1981.
- United States Nuclear Regulatory Commission, Generic Letter 88-01, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping" which invokes much of NUREG 0313, Revision 2, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping".
- United States Nuclear Regulatory Commission, Generic Letter 81-11, which modifies and invokes NUREG 0619, "BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking".
- United States Nuclear Regulatory Commission, I&E Bulletin 80-13 Visual Examination of Core Spray Spargers has been closed out via NUREG/CR-4523. The required bulletin actions have been performed satisfactorily and will be continued during every refueling outage.
- SNC "Inservice Inspection Outage Plan, Edwin I. Hatch Nuclear Plant, Unit 2 1995 Fall Refueling Outage."

- 1. Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
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- 3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A. 5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A
- SNC "Second Ten-Year Examination Plan, Edwin I. Hatch Nuclear Plant Unit 2."
- United States Nuclear Regulatory Commission NUREG 0803, "Generic Safety Evaluation Report Regarding Integrity of BWR SCRAM System".

Representative samples of the following systems, comprised of selected Class 1 and 2 Components, were examined using various NDE techniques, in accordance with the above documents:

Class 1

Reactor Pressure Vessel (2B11) Feedwater System (2B21) Reactor Recirculation System (2B31) Residual Heat Removal System (2E11) Core Spray System (2E21) High Pressure Coolant Injection System (2E41)

Class 2

Control Rod Drive System (2C11) Residual Heat Removal System (2E11) Core Spray System (2E21) High Pressure Coolant Injection System (2E41) Reactor Core Isolation Cooling System (2E51) Reactor Water Cleanup System (2G31) Main Steam Auxiliary (2N11) Feedwater System (2N21) Primary Containment Cooling System (2T47) Containment Purge and Inerting (2T48)

Other - Augmented (Non ASME Section XI)

Seven (7) Non-Safety RWCU System welds were examined using ultrasonic examinations techniques (UT) due to commitments made by GPC. These augmented exams are not required by the ASME Section XI Code but were performed due to commitments to GL 88-01.

1. Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302

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Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

CLASS I EXAMINATIONS

NUREG 0313

GPC is committed to the performance of surface and volumetric examinations on IGSCC susceptible welds in accordance with NUREG 0313. This commitment is formalized in GPC response to NRC Generic Letter 88-01. The below listed summary gives the total number of exams performed by outage end.

Category A

Two (2) Category A wids were examined. No indications were found that exceeded Code allowable limits.

Category D

Four (4) RINTSA welds were examined by UT and no indications were found that exceeded Code allowable limits.

Other Class 1 Examinations

Forty-five (45) ASME Section XI component examinations were performed utilizing UT, MT, PT and VT as applicable. These examinations included: RPV welds, piping welds, valve internals, and valve bolting materials.

Per I&E Bulletin 80-13 (NUREG/CR-4523), the core spray sparger and associated piping were VT examined. Indications are acceptable as is for one more cycle of operation per General Electric.

Per ASME Section XI, selected RPV internals were examined. These examinations included portions of the vessel interior, interior attachments beyond the belt line region and RPV internal components. See the Invessel Inspection section of this report for more detailed information.

Nine (9) Class 1 valves were disassembled for maintenance/inspection during the outage. The internals of these valves were VT inspected by GPC QC personnel.

Per SIL 433, all Shroud Head Bolts were examined. Indications of cracking were recorded in two (2) of these bolts. All indications were evaluated by General Electric and found to be acceptable for continued service.

CLASS II EXAMINATIONS

Sixty-eight (68) welds were examined using surface and/or volumetric NDE techniques as applicable per ASME Section XI requirements. Three (3) welds were examined per NUREG-0619 (UT only), fourteen (14) welds were augmented examinations (MT only), and the remaining fifty-one (51) welds were examined per ASME Section XI requirements.

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Pressure Testing

One (1) Class 1 system leakage test was performed satisfactorily. The Pressure Test Section of this report provides specific test identification and details.

Component Support Examinations (Class 1 and 2)

Twenty-nine (29) component supports were VT examined per the requirements of ASME Section XI during the outage with no recordable indications.

Snubber Examinations (Class 1 and 2)

Forty-eight (48) snubbers were VT examined per GPC QC procedures. All of the snubbers were determined to be acceptable.

Augmented Examinations

Seven (7) welds in the non-safety portion of the RWCU System were examined by UT per a GPC commitment to the NRC for NUREG 0313, Rev. 2, augmented requirements. No reportable indications were detected.

Repairs and Replacements (Class 1 and 2)

Numerous repair/replacement activities were performed prior to and during the outage. An itemized list of the repair/replacement activities is included in the Repair/Replacement Section of this report.

Reportable Indications

Following is an itemized list of all welds and components which were reported with indications of which were considered unacceptable. All of these items were either repaired and/or evaluated and then determined to be acceptable.

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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

Summary of Indications

Identification	Indication	Corrective Action				
Support 2T41-B003A-S01	Bolt found missing from base plate to floor.	Repaired. MWO 2-95-3197.				
Core Spray Internal Piping N-2A	Crack-like indication found in heat affected zone of weld 17.	Acceptable for continued operation per GE				
Lower Core Spray B M-1B	Linear indications recorded on bracket to shroud attachment welds.	Acceptable for continued operation per GE				
Shroud Head Bolts #10 and #24	Suspect indications found.	Acceptable for continued operation per GE				
Valve Bodies 2B21-F010B	Minor gouges found in the body above the spacer ring.	Evaluated as acceptable per GPC Engineering				
Valve Bodies 2E11-F015A	Gouge on valve stem.	Replaced. MWO 2-95-2847.				
Valve Bolting 2E11-F015B	Thread damage on 3 studs.	Evaluated as acceptable per GPC engineering.				
CRD Bolts	Minor pitting localized in area near the bolt head.	Replaced. MWO 2-95-1442.				
CRD Bolts	Small cracks near the head of the bolt.	Replaced. MWO 2-95-1442.				

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The following sections contain a summary of the NDE Examinations performed, provide additional information, and provide the results of those examinations.

- Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
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 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

SUMMARY

OF

CLASS 1 EXAMINATIONS

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results		Remarks
REAC	TOR PRESSURE	VESSEL ASSEMBLY						
B1.30	A-1/05	2C-1	UT-HAT-702V0/00	62-H 27-H	C-G2K-13 HA2018	CAL	NON-GEOMETRIC	16 NON-GEOMETRIC INDICATIONS WERE FOUND
B-A ASME		VESSEL-TO-FLANGE	UT-HAT-306V0/01	27-11	RPV001	NRI		AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. LIMITED EXAM DUE TO FLANGE - 86.5% COVERAGE.
B1.12 B-A ASME	A-1/05	2C-1-A LONGITUDINAL WELD ON UPPER SHELL	UT-HAT-702V0/00 UT-HAT-300VC/00	62-H	HA2011	RI	NON-GEOMETRIC	2 NON-GEOMETRIC INDICATIONS WERE FOUND AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. 93.7% COVERAGE.
B1.12 B-A ASME	A-1/05	2C-1-B LONGITUDINAL WELD ON UPPER SHELL	UT-HAT-702V0/00 UT-HAT-300V0/00	62-H	HA2014	RI	NON-GEOMETRIC	5 NON-GEOMETRIC INDICATIONS WERE FOUND AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. 91.2% COVERAGE.
B1 12 B-A ASME	A-1/05	2C-1-C LONGITUDINAL WELD ON UPPER SHELL	UT-HAT-702V0/00 UT-HAT-300V0/00	62-H	HA2013	RI	NON-GEOMETRIC	23 NON-GEOMETRIC INDICATIONS WERE FOUND AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. 92.1% COVERAGE.
B1.11 B-A ASME	A-1/05	2C-2 RPV UPPER SHELL-TO- UPPER MIDDLE SHELL	UT-HAT-702V0/00 UT-HAT-300V0/00	62-H	HA2016	RI	NON-GEOMETRIC	7 NON-GEOMETRIC INDICATIONS WERE FOUND AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. LIMITED EXAM DUE TO STABILIZER BRACKETS. 83.7% COVERAGE

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^{1.} Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302

^{2.} Plant: Edwin I. Hatch Nuclear Plane, Route 1, Box 278, Baxley, GA 31513.
3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A
5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results		Remarks
B1.12 B-A ASME	A-1/05	2C-2-A LONGITUDINAL WELD ON UPPER MID SHELL	UT-HAT-702V0/00 UT-HAT-300V0/00	62-H	HA2010	RI	NON-GEOMETRIC	3 NON-GEOMETRIC INDICATIONS WERE FOUND AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. 95.7% COVERAGE.
B1.12 B-A ASME	A-1/C5	2C-2-B LONGITUDINAL WELD ON UPPER MID SHELL	UT-HAT-702V0/00 UT-HAT-300V0/00	62-H	HA2015	RI	NON-GEOMETRIC	5 NON-GEOMETRIC INDICATIONS WERE FOUND AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. 95.6% COVERAGE.
B1 12 B-A ASME	A-1/05	2C-2-C LONGITUDINAL WELD ON UPPER MID SHELL	UT-HAT-702V0/00 UT-HAT-300V0/00	62-H	HA2012	RI	NON-GEOMETRIC	3 NON-GEOMETRIC INDICATIONS WERE FOUND AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. 90.4% COVERAGE.
B1 11 B-A ASME	A-1/05	2C-3 RPV UPPER MIDDLE SHELL- TO- LOWER MIDDLE SHELL	UT-HAT-702V0/00 UT-HAT-300V0/00	62-H	HA2009	RI	NON-GEOMETRIC	4 NON-GEOMETRIC INDICATIONS WERE FOUND AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. 95.8% COVERAGE.
B1 12 B-A ASME	A-1/05	2C-3-A LONGITUDINAL WELD ON LOWER MID SHELL	UT-HAT-702V0/00 UT-HAT-300V0/00	62-H	HA2006	RI	NON-GEOMETRIC	1 NON-GEOMETRIC INDICATION WAS FOUND AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI 91.4% COVERAGE
B1.12 B-A ASME	A-1/05	2C-3-B LONGITUDINAL WELD ON LOWER MID SHELL	UT-HAT-702V0/00 UT-HAT-300V0/00	62-H	HA2007	RI	NON-GEOMETRIC	9 NON-GEOMETRIC INDICATIONS WERE FOUND AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI 94.3% COVERAGE.

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302
 Plant: Edwin I. Hatch Nuclear Plans, Route 1, Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Consmercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results		Remarks
B1.12 B-A	A-1/05	2C-3-C	UT-HAT-702V0/00 UT-HAT-300V0/00	62-H	HA2008	RI	GEOMETRY	93.6% COVERAGE
ASME		LONGITUDINAL WELD ON LOWER MID SHELL						
B1.11	A-1/05	2C-4 RPV	UT-HAT-702V0/00	62-H	HA2005	RI	NON-GEOMETRIC	9 NON-GEOMETRIC
B-A			UT-HAT-300V0/00					INDICATIONS WERE FOUND AND EVALUATED AS
ASME		LOWER MIDDLE SHELL- TO- LOWER SHELL WELD						ACCEPTABLE TO ASME SECTION XI. 96.2% COVERAGE.
B1.12	A-1/05	2C-4-A	UT-HAT-702V0/00	61-H	HA2001	RI	NON-GEOMETRIC	14 NON-GEOMETRIC
B-A			UT-HAT-300V0/00					INDICATIONS WERE FOUND AND EVALUATED AS
ASME		LONGITUDINAL WELD ON LOWER SHELL						ACCEPTABLE TO ASME SECTION XI. 90.9% COVERAGE
D1 12	A-1/05	2^4B	UT-HAT-702V0/00	61-H	HA2002	RI	NON-GEOMETRIC	9 NON-GEOMETRIC
B1.12 B-A	A-1703	2 40	UT-HAT-300V0/00					INDICATIONS WERE FOUND
ASME		LONGITUDINAL WELD ON LOWER SHELL						AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. 91.9% COVERAGE
B1.12	A-1/05	2C-4-C	UT-HAT-702V0/00	61-H	HA2003	RI	NON-GEOMETRIC	25 NON-GEOMETRIC
B-A			UT-HAT-300V0/00					INDICATIONS WERE FOUND AND EVALUATED AS
ASME		LONGITUDINAL WELD ON LOWER SHELL						ACCEPTABLE TO ASME SECTION XL 93.7% COVERAGE.
B1.11	A-1/05	2C-5 RPV	UT-HAT-702V0/00	62-H	HA2004	RI	NON-GEOMETRIC	8 NON-GEOMETRIC INDICATIONS WERE FOUND
B-A ASME		LOWER SHELL-TO- BOTTOM HEAD TORUS	UT-HAT-300V0/00					AND EVALUATED AS ACCEPTABLE TO ASME SECTION XI. 95 ^% COVERAGE.

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1. Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results		Remarks	
B1 21 B-A	A-1A/05	2C-7 RPV	UT-HAT-702V0/00 UT-HAT-300V0/00	63-H 61-H	HA2017	RI	NON-GEOMETRIC	93 NON-GEOMETRIC INDICATIONS WERE FOUND AND EVALUATED AS	
ASME		BOTTOM HEAD TORUS- TO- BOTTOM HEAD DOME						ACCEPTABLE TO ASME SECTION XI. 95.3% COVERAGE.	
B1.40	A-3/04	2HC-2 (20-39)	MT-H-500/06	64-H	S95H2M060	NRI		LIMITED EXAM DUE TO FLANGE CONFIGURATION	
B-A			UT-H-410/06		S95H2U149	NRI		89% COVERAGE.	
ASME		CLOSURE HEAD-TO-FLG CENTERLINE STUD 20 TO STUD			S95H2U150 S95H2U151	NRI NRI			
B6 40	A-33/01	2LIG-21 THRU 40	UT-H-419/03	61-H	S95H2U210	NRI		LIMITED EXAM DUE TO	
B-G-1	A-33/01							FLANGE SEATING SURFACE - 95% COVERAGE	
ASME		FLANGE LIGAMENTS						30% 0010.000	
		2N2A (RINTSA)	UT-H-415/06	125-H	S95H2U118	RI	GEOMETRY		
NUREG-03	313D	RINTSA WELD							
							GEOMETRY		
		2N2C (RINTSA)	UT-H-415/06	125-H	S95H2U119	RI	GEOMETRY		
NUREG-03	313D	RINTSA WELD							
			177 11 415/00	125-H	S95H2U120	RI	GEOMETRY		
-		2N2D (RINTSA)	UT-H-415/06	123-11	3331120120				
NUREG-0	313D	RINTSA WELD							

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 3030.
 Plant: Edwin I, Hetch Nuclear Plant, Route 1, Box 278, Bexley, QA 31513.
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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 Class 1 Components

ASME ection XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
		2N2E (RINTSA)	UT-H-415/06	125-H	S95H2U121	RI GEOMETRY	
JREG-03	13D	RINTSA WELD					
20	A-1/05	2N10 (N-SE)	VT-H-720/03		S95H2V059	SAT	
ME		RPV INST. NOZZLE TO SAFE-END					
1.13	A-1A/05	2N10 (N-SH)	VT-H-720/03		S95H2V060	SAT	
E SME		RPV INSTRUMENTATION NOZZLE TO SHELL					
i.20	A-1/05	2N11A (N-SE)	UT-H-400/13	158-H	S95H2U172 S95H2U173	NRI NRI	
-5/1		RPV INST. NOZZLE TO SAFE-END					
	A-1/05	2N11A (N-SH)	VT-H-720/03		S95H2V061	SAT	
13 E	A-1/05	RPV INSTRUMENTATION NOZZLE	774772000				
		TO SHELL					
5.20 F	A-1/05	2N11B (N-SE)	UT-H-400/13	158-H	S95H2U174 S95H2U175	NRI NRI	
L-571		RPV INST. NOZZLE TO SAFE-END					

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302
 Plant: Edwin I, Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B4 13 B-E	A-1/05	2N11B (N-SH)	VT-H-720/03		S95H2V062	SAT	
ASME		RPV INSTRUMENTATION NOZZLE TO SHELL					
B5.20	A-1/05	2N12A (N-SE)	UT-H-400/13	158-H		NRI	
B-F SIL-571		RPV INST. NOZZLE TO SAFE-END			S95H2U177	NRI	
B4 13 B-E	A-1/05	2N12A (N-SH)	VT-H-720/03		S95H2V063	SAT	
ASME		RPV INSTRUMENTATION NOZZLE TO SHELL					
B5.20 B-F	A-1/05	2N12B (N-SE)	UT-H-400/13	158-H	S95H2U178 S95H2U179	NRI NRI	
SIL-571		RPV INST. NOZZLE TO SAFE-END					
B4.13 B-E	A-1/05	2N12B (N-SH)	VT-H-720/03		S95H2V064	SAT	
ASME		RPV INSTRUMENTATION NOZZLE TO SHELL					
B5 20 B-F	A-1/05	2N16A (N-SE)	VT-H-710/03		S95H2V048	SAT	
SIL-571		RPV INST. NOZZLE TO SAFE-END					

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E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B4 13 B-E	A-1/05	2N16A (N-SH)	VT-H-720/03		S95H2V065	SAT	
ASME		RPV INSTRUMENTATION NOZZLE TO SHELL					
B5.20 B-F	A-1/05	2N16B (N-SE)	VT-H-710/03		S95H2V049	SAT	
SIL-571		RPV INST. NOZZLE TO SAFE-END					
B4 13	A-1/05	2N16B (N-SH)	VT-H-720/03		S95H2V066	SAT	
B-E ASME		RPV INSTRUMENTATION NOZZLE TO SHELL					
B6.10 B-G-1	A-33/01	2NUT-21 THRU 40	MT-H-500/06		S95H2M034	NRI	EXAMINED ONE DIRECTION ONLY DUE TO CONFIGURATION.
ASME		CLOSURE HEAD NUTS					
B6 20 B-G-1	A-33/01	2STUD-21 THRU 40	UT-H-420/06	156-H	S95H2U211 S95H2V058	NRI SAT	
ASME		CLOSURE HEAD STUDS					
B6.50	A-33/01	2WASHER-21 THRU 40	VT-H-710/03		S95H2V046	SAT	
B-G-1 ASME		CLOSURE HEAD WASHERS					

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2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exem/Cal Sheet No.	Results		Remarks
B15.10		CLASS 1 LEAKAGE TEST	GPC		N/A			PERFORMED PER 42-IT-TET- 006-2S Rev. 7.
3-P ASME		PRESSURE RETAINING BOUNDARY LEAKAGE TEST						
37 80	A-29/03	FLANGE BOLTING	GPC		N/A		SEE PAGE 27	EXAMINED BY GPC QC.
3-G-2 ASME		CRD HOUSING						
38.10	A-3/04	RPV (HEAD LUGS)	MT-H-500/06		S95H2M032	NRI		
ASME		4 LIFTING LUGS						
		SHROUD HEAD BOLTS	UT-HAT-501V2/00	LFK 009-1	CM-001 CM-002	CAL		2 BOLTS WERE FOUND WITH SUSPECT INDICATIONS, THESE WERE LATER
SIL-433		1 THRU 36			DM001 DM002 DM003	RI RI	GEOMETRY GEOMETRY	EVALUATED AS NOT CRACKED.
	FEEDWATE	ER SYSTEM						
B9.11 B-J	A-1/05	2B21-1FW-18B-1	UT-H-400/13 MT-H-500/06	77-H	S95H2M066 S95H2U188	NRI NRI		ONE-SIDED EXAM DUE TO VALVE.
ASME		VALVE TO PIPE			S95H2U189 S95H2U190	RI N/A T/C	GEOMETRY	

RECIRCULATION SYSTEM

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B9 31	A-16 (DRAFT)	2B31-1RCM-28AS-9BC-4	PT-H-600/05 UT-H-400/13	154-H	S95H2U146 S95H2U147	NRI NRI	SURFACE EXAM PERFORMED BY GPC QC. ONE-SIDED EXAM DUE TO BRANCH
NUREG-03	113A	BC TO PIPE			S95H2U148	NRI	CONNECTION CONFIGURATION.
B7.50 B-G-2	A-16 (DRAFT)	2B31-1RCM-28AS-9BC-4 FB	VT-H-710/03		S95H2V068	SAT	
ASME		FLANGE BOLTING					
89 31 B-J	A-18 (DRAFT)	2B31-1RCM-28BS-8BC-4	PT-H-600/05 UT-H-4C0/13	154-H	S95H2U185 S95H2U186	NRI	SURFACE EXAM PERFORMED BY GPC QC. ONE-SIDED EXAM DUE TO BRANCH
NUREG-03	113A	BC TO PIPE			S95H2U187		CONNECTION CONFIGURATION.
B7.50 B-G-2	A-18 (DRAFT)	2B31-1RCM-28BS-8BC-4 FB	VT-H-710/03		\$95H2V	SAT	
ASME		FLANGE BOLTING					
	CORE SPR	AV SVSTEM					
PO 44		2E21-1CS-10B-4	UT-H-400/13	137-H	S95H2M064	NRI	ONE-SIDED EXAM DUE TO
B9.11 B-J	A-25/07		MT-H-500/06		S95H2U134 S95H2U135	NRI NRI	PENETRATION CONFIGURATION
ASME		PIPE TO PENETRATION			S95H2U136	N/A T/C	

HIGH PRESSURE COOLANT INJECTION SYSTEM

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302
 Plant: Edwin I. Hatch Nuclear Plant. Route I. Box 278, Baxley, QA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Exam. nation/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B9.11	A-26:06	2E41-1HPCI-10-D-21	UT-H-400/13	54-H	S95H2M065	NRI	
B-J			MT-H-500/06		S95H2U191	NRI	
ASME		PENETRATION TO PIPE			S95H2U192	NRI	
	VALVE E	BOLTING					
B7.70	A- 13	2B21-F010B BOLTING	GPC		N/A		EXAMINED BY GPC QC. MWO
B-G-2			45QC-INS-010-OS				2-95-3015. SAT.
ASME		VALVE BOLTING					
B7.70	A- 5	2B21-F013D BOLTING	GPC		N/A		EXAMINED BY GPC QC MWO
B-G-2	A- 3	2821-F0130 BOLTING	45QC-INS-010-OS		INA		2-94-3430. SAT.
ASME		VALVE BOLTING	4300-115-010-03				
B7.70	A- 5	2B21-F013H BOLTING	GPC		N/A		EXAMINED BY GPC QC. MWO
B-G-2			45QC-INS-010-OS				2-95-2204 SAT.
ASME		VALVE BOLTING					
B7.70	A- 5	2B21-F013L BOLTING	GPC		N/A		EXAMINED BY GPC QC. MWO
B-G-2			45QC-INS-010 S				2-94-3436. SAT.
ASME		VALVE BOLTING					
B7.70	A- 5	2B21-F013M BOLTING	GPC		N/A		EXAMINED BY GPC QC. MWO
B-G-2			45QC-INS-010-OS				2-94-3437. SAT.
ASME		VALVE BOLTING					

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- Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanty, Georgia 30302
 Plant: Edwin I. Hatch Nuclear Plant. Route 1, Box 278, Buxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Data 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B7.70	A-6/06	2B21-F028A BOLTING	VT-H-710/03		S95H2V050	SAT	
B-G-2							
ASME		VALVE BOLTING					
B7.70	A-7/07	2B21-F028B BOLTING	VT-H-710/03		S95H2V051	SAT	
B-G-2							
ASME		VALVE BOLTING					
B7.70	A-8/07	2B21-F028C BOLTING	VT-H-710/03		S95H2V052	SAT	
B-G-2							
ASME		VALVE BOLTING					
B7.70	A-9/06	2B21-F028D BOLTING	VT-H-710/03		S95H2V053	SAT	
B-G-2							
ASME		VALVE BOLTING					
B7.70	A- 22	2E11-F015A BOLTING					
5-G-2	7. 22	ZETT- OTSA BOLTING	GPC		N/A		EXAMINED BY GPC QC. MWO 2-95-2847 SAT.
ASMF		VALVE BOLTING	45QC-INS-010-OS				Z-93-2047. 3A1.

Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 3030:
 Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, QA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B7.70 B-G-2 ASME	A- 23	2E11-F015B BOL7ING VALVE BOLTING	GPC 45QC-INS-010-OS		N/A		EXAMINE: 3Y GPC QC. MWO 2-95-2849. THREAD DAMAGE ON 3 STUDS FOUND DURING INSPECTION. THREAD DAMAGE EVALUATED AS PEING CAUSED BY DISASSEMBLY. NO REDUCTION IN BOLT INTEGRITY. BOLTS ACCEPTABLE FOR CONTINUED SERVICE PER GPC ENGINEERING EVALUATION.
B7.70 B-G-2 ASME	A- 23/05	2E11-F017B BOLTING VALVE BOLTING	VT-H-710/03		S95H2V067	SAT	
B7.70 B-G-2 ASME	A- 24	2E21-F005A BOLTING VALVE BOLTING	GPC 45QC-INS-010-OS		N/A		EXAMINED BY GPC QC. MWO - 2-95-2845. SAT
1,7.70 B-G-2 ASME	A- 25	2E21-F005B BOLTING VALVE BOLTING	GPC 45QC-INS-010-OS		N/A		EXAMINED BY GPC QC. MWO 2-95-2846. SAT.

VALVE BODIES

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 303
 Plant: Edwin I. Hatch Nuclear Plans, Route 1, Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 Class 1 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B12.50 B-M-2 ASME	A- 13	2821-F010B BODIES VALVE BODIES	GPC 45QC-INS-010-OS		N/A		EXAMINED BY GPC QC. MWO 2-95-3015. SAT. MINOR GOUGES FOUND IN THE BODY ABOVE THE SPACER RING AREA. EVALUATED BY ENGINEERING AS ACCEPTABLE.
B12.50 B-M-2 ASME	A- 5	2B21-F013D BODIES VALVE BODIES	GPC 45QC-INS-010-OS		N/A		EXAMINED BY GPC QC. MWO 2-94-3430. SAT.
B12.50 B-M-2 ASME	A- 5	2B21-F013H BODIES VALVE BODIES	GPC 45QC-INS-010-OS		N/A		EXAMINED BY GPC QC MWO 2-95-2204 SAT
B12.50 B-M-2 ASME	A- 5	2B21-F013L BODIES VALVE BODIES	GPC 45QC-INS-010-OS		N/A		EXAMINED BY GPC QC. MWO 2-94-3436. SAT.
B12.50 B-M-2 ASME	A- 5	2B21-F013M BODIES VALVE BODIES	GPC 45QC-INS-010-OS		N/A		EXAMINED BY GPC QC. MWO 2-94-3437. SAT.

Owner: Ocorgia Power Company, 333 Piedmont Ave., NE. P. J. Box 4545, Atlanta, Ocorgia 303
 Plant: Edwin L Hatch Nuclear Plant, Route 1, Box 278, Baxley, OA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

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E.I. Hatch Unit 2 Class 1 Components

	ASME ction XI	Exam Figure No.	Examination/Area	Examination Procedure Co		Exam/Cal Sheet No.	Results	Remarks
В	12 50 M-2	A- 22	2E11-F015A BODIES	GPC 45QC-INS-010-OS	,	N/A		EXAMINED BY GPC QC. MWO 2-95-2847 SAT. VALVE STEM HAS A GOUGE. STEM FOUND
A	SME		VALVE BODIES					TO BE ACCEPTABLE AFTER REPLACEMENT. SOME THREAD DAMAGE ON BONNET HOLES WHICH WAS REPAIRED/EVALUATED
B	12.50	A- 23	2E11-F015B BODIES	GPC		N/A		EXAMINED BY GPC QC. MWO
3-	M-2			45QC-INS-010-OS				2-95-2849. PITTING NOTED IN STUFFING BOX EVALUATED
AS	SME		VALVE BODIES					AS ACCEPTABLE BY GPC ENGINEERING.
D.	12 50	A- 24	2E21-F005A BODIES	GPC		N/A		EXAMINED BY GPC QC MWO
	M-2			45QC-INS-010-OS				2-95-2845 SAT
	SME		VALVE BODIES					
			2F21-F005B BODIES	GPC		√A		EXAMINED BY GPC QC. MWO
_	12.50	A- 25	ZEZ 1-PUUSB BODIES	45QC-INS-010-QS				2-95-2846. SAT.
_	M-2 SME		VALVE BODIES	1000 110 010 00				

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta. Georgia 3030/.
 Plant: Edwin I. Hatch Nuclear Plant. Route 1, Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if roq.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

- 1. Owner: Georgia Fower Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
- 2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Eaxley, GA 31513.
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VISUAL EXAMINATION OF CLASS 1 CRD BOLTING

ASME Section XI requires examination of the CRD components (bolts, studs, and nuts) whenever the housings are disassembled. Twenty (20) CRDs were replaced during the outage which facilitated visual examination (VT-1) of the bolts, studs, and nuts. The subject examinations were performed using GPC QC personnel in conjunction with the CRD maintenance/replacement activities. Listed below are the CRDs which were examined using GPC procedure 45QC-INS-010-0S. The majority of the bolts were found to have minor pitting generally localized in the area near the bolt head. Four (4) out of the one hundred sixty (160) bolts examined were found to have small cracks near the head of the bolt. These types of indications on CRD bolting are addressed in General Electric SIL No. 483. All bolts with pitting or cracking were replaced.

34-35	10-39
10-15	18-03
26-27	18-23
38-47	26-23
26-19	22-47
10-43	02-31
22-15	38-23
14-19	46-39
46-15	22-27
02-19	14-31

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SUMMARY

OF

CLASS 2 EXAMINATIONS

E.I. Hatch Unit 2 Class 2 Components

ASME E Section XI	xam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
co	NTROL ROD	DRIVE SYSTEM					
	B-83/04	2C11-2CRD-3-2FW-1611	UT-H-400/13	4-H	S95H2U107 S95H2U108	NRI NRI	ONE-SIDED EXAM DUE TO REDUCER.
NUREG-0619		PIPE TO REDUCER					
	B-83/04	2C11-2CRD-4-2FW-1611	UT-H-400/13	142-H	S95H2U109	NRI	LIMITED EXAM DUE TO CONFIGURATION 35%
- NUREG-0619		REDUCER TO TEE			S95H2U110	NRI	COVERAGE.
RESID	DUAL HEAT R	EMOVEL SYSTEM					
	B-97/03	2E11-2RHR-4-HS-11	MT-H-500/06		S95H2M059	NRI	
AUGMENTED		PIPE TO 45 DEGREE ELBOW					
	B-97A/03	2E11-2RHR-4-HS-14	MT-H-500/06		S95H2M056	NRI	
AUGMENTED		ELBOW TO PIPE					
	B-33/04	2E11-2RHR-4B-PD-D-1	MT-H-500/06		S95H2M014	NRI	
AUGMENTED		BC TO VALVE					
C5.21	B-67	2E11-2RHR-6B-SS-3A	MT-H-500/06	50-H	S95H2U180	NRI	SURFACE EXAMINATION PERFORMED BY GPC QC
C-F ASME	(DRAFT)	PIPE TO CAP	UT-H-400/13		S95H2U181 S95H2U182 S95H2U193	NRI NRI N/A T/C	MWO 2-95-2301 FW-004.

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E.I. Hatch Unit 2 Class 2 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
C5 21	B-20/04	2E11-2RHR-6B-SS-9	MT-H-500/06	50-H	S95H2M018	NRI	
C-F			UT-H-400/13		S95H2U163	NRI	
ASME		PIPE TO ELBOW			S95H2U164	NRI	
					S95H2U165	N/A T/C	
C5.11 C-F	B-23/05	2E11-2RHR-8-FPS-12	MT-H-500/06		S95H2M057	NRI	
ASME		PIPE TO 45-DEGREE ELBOW					
C5.11	B-23/05	2E11-2RHR-8-FPS-16	MT-H-500/06		S95H2M068	NRI	
ASME		PIPE TO ELBOW					
C5.11	B-24/03	2E11-2RHR-10A-SWDS-8	45QC-INS-006-OS		Q95H2M043	NRI	EXAMINED BY GPC QC.
-F							
ASME		VALVE TO ELBOW					
C5.11	B-25/03	2E11-2RHR-10B-SWDS-2	MT-H-500/06		S95H2M001	NRI	
ASME		PIPE TO VALVE					
5 11	B-14/08	2E11-2RHR-16A-DS-4	MT-H-500/06		S95H2M007	NRI	
-F							
SME		PIPE TO ELBOW					
05.11 C-F	B-26/04	2E11-2RHR-16A-HXI-5	45QC-INS-006-OS		Q95H2M044	NRI	EXAMINED BY GPC QC.
SME		PIPE TO ELBOW					

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 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
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E.I. Hatch Unit 2 Class 2 Components

ASME Exection XI	nam Figure	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
	B-27/03	2E11-2RHR-16A-HXO-2BC-1/2E11- 2RHR-4A-HXO	MT-H-500/06		S95H2M010	NRI	
UGMENTED		PIPE TO BC					
5.11 -F	B-26 (DRAFT)	2E11-2RHR-16A-SS-5A	45QC-INS-006-OS		Q95H2M056	N/A	EXAMINED BY GPC QC. MWO 2-95-2301 FW-001.
SME		PIPE TO CAP					
3 20 -C	B-18/06	2E11-2RHR-16B-DS-5PL-1 THRU 8	MT-H-500/06		S95H2M031	NRI	
SME		DEVICE 2E11-RHR-R129					
5.11 -F	B-21/07	2E11-2RHR-16B-SH-6	MT-H-500/06		S95H2M002	NRI	
SME		45-DEGREE ELBOW TO PIPE					
5.11 -F	B-30 (DRAFT)	2E11-2RHR-16B-SS-4A	45QC-INS-006-OS		Q95H2M057	N/A	EXAMINED BY GPC QC MWO 2-95-2300 FW-001
SME		PIPE TO CAP					
5.11 F	B-35/04	2E11-2RHR-20A-D-1	45QC-INS-006-OS		Q95H2M045	NRI	EXAMINED BY GPC QC.
SME		TEE TO PIPE					
5.11 F	B-35/04	2E11-2RHR-20A-D-2	45QC-INS-006-OS		Q95H2M046	NRI	EXAMINED BY GPC QC.
SME		PIPE TO 45-DEGREE ELBOW					

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E.I. Hatch Unit 2 Class 2 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results		Remarks
C5.11 C-F	B-35/04	2E11-2RHR-20A-D-11	45QC-INS-006-OS		Q95H2M047	NRI		EXAMINED BY GPC QC.
ASME		PIPE TO ELBOW						
C5 11 C-F	B-29/04	2E11-2RHR-20A-PD-C-8	45QC-INS-006-OS		Q95H2M048	NRI		EXAMINED BY GPC QC
ASME		VALVE TO PIPE						
C3.20 C-C	B-36/03	2E11-2RHR-20B-D-11PL-1 THRU 4	MT-H-500//16		S95H2M011	NRI		
ASME		DEVICE 2E11-RHR-H169						
C3.20 C-C	B- 36/03	2E11-2RHR-2u8-D-11PL-5 THRU 12	MT-H-500/06		S95H2M012	NRI		
ASME		DEVICE 2E11-RHR-R241						
C5.11 C-F	B-37/04	2E11 2RHR-20C-D-5	45QC-INS-006-OS		Q95H2M049	NRI		EXAMINED BY GPC QC.
NIME		VALVE TO PIPE						
C5 21	B-40/04	2E11-2RHR-24A-BP-20	45QC-INS-006-OS UT-H-400/13	139-H	Q95H2M050 S95H2U123	NRI N/A T/C		MT BY GPC QC. ONE-SIDED EXAM DUE TO TEE
C-F ASME		PIPE TO TEE			S95H2U128 S95H2U132	NRI RI	GEOMETRY	CONFIGURATION.
C5.21	B-43/03	2E11-2RHR-24A-TS-A-17	45QC-INS-006-OS UT-H-400/13	139-H	Q95H2M051 S95H2U122	NRI N/A T/C		MT BY GPC QC. ONE-SIDED EXAM DUE TO VALVE
C-F ASME		VALVE TO PIPE	01-11-400/13		S95H2U130 S95H2U131	NRI RI	GEOMETRY	CONFIGURATION.

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E.I. Hatch Unit 2 Class 2 Components

ASME E	xam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
	B-49/04	2E11-2RHR-24B-TS-B-17BC-1 /2E11-2RHR-4B-TS-B	MT-H-500/06		Q95H2M039	NRI	
					S95H2M009	NRI	
AUGMENTED		PIPE TO BC					
C1.10	B-1/03	2HX-A-2	UT-H-400/13	73-H	Q95H2M052	NRI	LIMITED EXAMINATION DUE
C-A				72-H	S95H2U124	N/A T/C	TO SUPPORT LUG INTERFERENCE 73%
ASME		UPPER SHELL RING TO LOWER			S95H2U125	NRI	COVERAGE
		SHELL RING			S95H2U126	RI GEOMETRY	
					S95H2U127	NRI	
					S95H2U129	NRI	
23.10	B-1/03	2HX-A-3S-2	45QC-INS-006-OS		Q95H2M053	NRI	EXAMINED BY GPC QC
C-C	6-1/03	2114-4-33-2	4340 110 000 00				
ASME		LOWER SHELL RING SUPPORT					
3.10	B-1/03	2HX-A-3S-3	45QC-INS-006-OS		Q95H2M054	NRI	EXAMINED BY GPC QC.
C-C							
ASME		LOWER SHELL RING SUPPORT					
3.10	B-1/03	2HX-A-3S-4	45QC-INS-006-OS		Q95H2M055	NRI	EXAMINED BY GPC QC.
2-C	6-1703	2114-4-33-4	4545 110 000 00				
SME		LOWER SHELL RING SUPPORT					
	CORE SPRA	VEVETEM					
	CORE SPINA	I SISIEM					
25.11	8-54/05	2E21-2CS-10B-11	MT-H-500/06		S95H2M063	NRI	
)-F							
ASME		PIPE TO VALVE					

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ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Exam/Cal Block Sheet No.		Remarks
C5.11	B-58/04	2E21-2CS-12A-34	MT-H-500/06	S95H2M0	028 NRI	
C-F ASME		PIPE TO ELBOW				
C3 20 C-C	B-61/05	2E21-2CS-20A-TS-12PS-1A AND 2A	45QC-INS-006-OS	Q95H2M0	036 NRI	EXAMINED BY GPC QC.
ASME		DEVICE 2E21-CS-R41				
C5.11 C-F	B-63/05	2E21-2CS-20B-TS-2	45QC-INS-006-OS	Q95H2M0	035 NRI	EXAMINED BY GPC QC.
ASME		PIPE TO 45-DEGREE ELBOW				
C6 10 C-G	B-2A/02	2E21-2CS-POP-B-1BC/	MT-H-500/06	S95H2M0	13 NRI	RR 3.1.3
ASME		SHAFT ELBOW TO BC CORE SPRAY PUMP B				
HIGH PRE	SSURE COOLA	NT INVECTION SYSTEM				
	B-94/02	2E41-2HPCI-4-MFL-1	45QC-INS-006-OS	Q95H2M0	40 NRI	EXAMINED BY GPC QC.
AUGMENTE	C	BRANCH CONNECTION TO PIPE				
	B-94/02	2E41-2HPCI-4-MFL-4	MT-H-500/06	S95H2M02	25 NRI	
AUGMENTE	D	PIPE TO ELBOW				

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 Plans: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, QA 31513.
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E.I. Hatch Unit 2 Class 2 Components

ASME I	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results		Remarks
C5 21	B-16/04	2E41-2HPCI-8A-SS-1	MT-H-500/06	52-H	S95H2M020	NRI		ONE-SIDED EXAM DUE TO
C-F			UT-H-400/13		S95H2U101	RI	GEOMETRY	REDUCER.
ASME		REDUCER TO PIPE			S95H2U102	RI	GEOMETRY	
C5 21	B-16	2E41-2HPCI-8A-SS-5A	MT-H-500/06		S95H2U169	NRI		MT PERFORMED BY GPC QC
C-F	(DRAFT)		UT-H-400/13	154-H	S95H2U170	NRI		MWO 2-95-2301 FW-003
ASME		PIPE TO CAP			S95H2U171	NRI		
KSME					S95H2U194	N/A T/C		
05.21	B-67/G3	2E41-2HPCI-10-D-10	MT-H-500/06	54-H	S95H2M006	NRI		
C-F			UT-H-400/13		S95H2U160	N/A T/C		
SME		TEE TO REDUCER			S95H2U161	NRI		
					S95H2U162	NRI		
5.21	B-68/05	2E41-2HPCI-10-D-29	MT-H-500/06	54-H	S95H2M023	NRI		
C-F			UT-H-400/13		S95H2U157	NRI	CECHETEV	
ASME		PIPE TO TEE			S95H2U158	RI	GEOMETRY	
					S95H2U159	N/A T/C		
C5 21	B-70/05	2E41-2HPCI-10-TL-1	45QC-INS-006-OS	137-H	Q95H2M041	NRI		MT PERFORMED BY GPC QC
25.21 2-F	B-10/03	224	UT-H-400/13		S95H2U183	NRI		ONE-SIDED EXAM DUE TO TEE CONFIGURATION.
SME		TEE TO PIPE			S95H2U184	NRI		TEE COM TOOLS WITHOUT
25.21	B-66/07	2E41-2HPCI-14-R-33	MT-H-500/06	116-H	S95H2M029	NRI		
C-F			UT-H-400/13		S95H2U207	NRI		
ASME		PIPE TO 45-DEGREE ELBOW			S95H2U208	NRI		
	B-73/05	2E41-2HPCI-16-TS-12	45QC-INS-006-OS		Q95H2M038	NRI		EXAMINED BY GPC QC.
AUGMENTE	D	VALVE TO PIPE						

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302
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 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
	B-73/05	2E41-2HPCI-16-TS-24	45QC-INS-006-OS		Q95H2M042	NRI	EXAMINED BY GPC QC.
AUGMENT	ED	PIPE TO REDUCER					
C5.11 C-F	B-74/03	2E41-2HPCI-18-TD-1	MT-H-500/06		S95H2M019	NRI	
ASME		FLANGE TO TEE					
C5.11 C-F	B-69/04	2E41-2HPCI-20-TD-14	MT-H-500/06		S95H2M003	NRI	
ASME		PIPE TO TEE					
C5.11 C-F	B-69/04	2E41-2HPCI-20-TD-23	MT-H-500/06		S95H2M004	NRI	
ASME		ELBOW TO PIPE					
REACTO	R CORE ISOLA	TION COOLING SYSTEM					
	B-88/02	2E51-2RCIC-4-PS-5	MT-H-500/06		S95H2M016	NRI	
AUGMENT	ED	PIPE TO ELBOW					
	B-95/03	2E51-2RCIC-4-SS-13	MT-H-500/06		S95H2M069	NRI	
AUGMENT	ED	PIPE TO ELBOW					

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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

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ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Ramarks
	B-89/02	2E51-2RCIC-6-CST-9	MT-H-500/06		S95H2M015	NRI	
AUGMENTE	ED	PIPE TO 45 DEGREE ELBOW					
	8-87/02	2E51-2RCIC-6-PS-4BC/ 2E51- 2RCIC-3-PS	MT-H-500/06		S95H2M017	NRI	
AUGMENTE	D	PIPE TO BRANCH CONNECTIONS					
	B-86/04	2E51-2RCIC-6-TS-10	MT-H-500/06		S95H2M021	NRI	
UGMENTE	D	ELBOW TO VALVE					
5 11 F	B-62/06	2E51-2RCIC-10-TD-17	MT-H-500/06		S95H2M005	NRI	
ASME		PIPE TO VALVE					
REAC	CTOR WATER	CLEAN-UP SYSTEM					
	B-83/04	2G31-2RWCU-4-2FW-33	UT-H-400/13	142-H	S95H2U111 S95H2U112	NRI NRI	ONE-SIDED EXAM DUE TO TEE CONFIGURATION.
IUREG-061	9	TEE TO PIPE					
MA	IN STEAM AUX	CILIARY SYSTEM					
5.21 C-F	B-6/04	2N11-2MSA-10C-SSR-2	MT-H-500/06 UT-H-400/13		S95H2M058 S95H2U166	NRI N/A T/C	ONE-SIDED EXAM DUE TO VALVE CONFIGURATION
SME		PIPE TO VALVE			S95H2U167 S95H2U168	RI GEOMETRY NRI	

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E.I. Hatch Unit 2 Class 2 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Sheet No.	Results		Remarks
C5 21	B-7/04	2N11-2MSA-16A-18	MT-H-500/06	53-H	S95H2M027	NRI		
C-F			UT-H-400/13		S95H2U115	RI	GEOMETRY	
ASME		45-DEGREE ELBOW TO PIPE			S95H2U116	RI	GEOMETRY	
					S95H2U117	N/A T/C		
C3.20	B-9/06	2N11-2MSA-24A-5PL-1 THRU 8	MT-H-500/06		S95H2M061	NRI		
C-C								
ASME		DEVICE 2N11-MS-R45						
C3.20	B-11/06	2N11-2MSA-2-C-1PS-1 THRU 2	MT-H-500/06		S95H2M026	NRI		
C-C								
ASME		PIPE SUPPORT						
	0.40/00	2N11-2MSA-24D-4	N* H-500/06	152-H	S95H2M067	NRI		
C5.21	B-12/06	ZN11-ZMSA-Z4U-4	UT-H-400/13		S95H2U204	N/A T/C		
C-F		PIPE TO ELBOW	011110010		S95H2U205	RI	GEOMETRY	
ASME		PIPE TO ELBOW			S95H2U206	RI	GEOMETRY	
C3.20	B-12/06	2N11-2MSA-24D-11PL-1 THRU 8	MT-H-500/06		S95H2M022	NRI		
C-C	0 12.00							
ASME		DEVICE 2N11-MS-A60						
	FEEDWATE	D CVCTEM						
	PEEUWATE	Walalew						ONE-SIDED EXAM DUE TO
B9.11	A-38A/02	2N21-2FW-18A-3	MT-H-500/06		S95H2M024	NRI	OFOMETRY	VALVE CONFIGURATION. RR
B-J			UT-H-400/13		S95H2U113	RI	GEOMETRY	2.1.4
ASME		PIPE TO VALVE			S95H2U114	RI	GEOMETRY	

CONTAINMENT PURGE AND INERTING SYSTEM

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 3030
 Plant: Edwin J. Hatch Nuclear Plant. Route 1, Box 278, Baxley, GA 31513.
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 Commercial Service Date 99/05/79 6. National Board No. for Unit N/A

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
C5.11	B-77/05	2T48-2CPI-6-SVD-6	MT-H-500/06		S95H2M008	NRI	
C-F ASME		PIPE TO ELBOW					
C3 20 C-C	B-78/04	2T48-2CPI-18-PID-2PL-1 THRU 8	MT-H-500/06		S95H2M062	NRI	
ASME		DEVICE 2T48-CPUR-R49					
C5.11	B-78/04	2T48-2CPI-18-PID-6	MT-H-500/06		Q95H2M037	NRI	EXAMINED BY GPC QC
C-F ASME		45-DEGREE ELBOW TO PIPE					
C5.11	B-79/06	2T48-2CPI-20-PIT-5	MT-H-500/06		S95H2M030	NRI	
C-F ASME		VALVE TO PIPE					

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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

SUMMARY

OF

CLASS 1 AND 2 PRESSURE TESTS

PRESSURE TESTING

- 1. Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
- Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
- Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
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CLASS 1 AND 2 PRESSURE TESTS

GENERAL

This section of the report provides a discussion of the pressure tests which were performed during the 1995 Plant E. I. Hatch Unit 2 Fall Refueling Outage. These pressure tests were performed for the purpose of inservice inspection of Class 1 components. No Class 2 pressure tests were performed this outage. The pressure tests and their boundaries are identified in the Inservice Inspection Plan Documents prepared by Southern Nuclear Operating Company.

All pressure tests were performed in accordance with ASME Section XI, 1980 Edition with Addenda through Winter 1981. All tests were witnessed and/or reviewed by the resident ANII. The completed test reports are available for review in the Records Management Department at Plant E. I. Hatch.

CLASS 1 PRESSURE TESTS

One (1) Class 1 Leakage Test was performed during the outage per ASME Section XI, paragraph IWA-5211(a). The test was performed per GPC procedure 42IT-TET-006-2S, ISI Pressure Test of the Class 1 System.

TEST RESULTS

One (1) fillet weld on a small bore reactor water sample line was found cracked during the VT-2 examination. This weld was repaired/replaced prior to startup. The VT-2 examination also found some minor leakage at mechanical joints. Any component which was disassembled prior to startup or to repair leakage, was re-examined during startup at normal operating pressure (1035 psig) per GPC procedure 42IT-TET-004-0S, Operating Pressure Testing of Piping and Components.

CLASS 1 PRESSURE TEST SUMMARY

TEST I.D.	PROCEDURE	MWO NUMBER
2B21-LT-1	42IT-TET-006-2S	2-95-0377

- Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
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SUMMARY OF VISUAL EXAMINATIONS CLASS 1 AND 2 COMPONENT SUPPORTS

1. Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302

2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.

 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for 6. National Board No. for Unit N/A

COMPONENT SUPPORT EXAMINATIONS

This section of the report provides a discussion of the visual examinations performed on selected component supports on Hatch Unit 2. Pipe Supports inspected are listed on separate tables as part of this section. The subject examinations were performed during the Refueling/Maintenance Outage. Examinations were performed using SNC Procedure VT-H-730 (VT-3 and VT-4). The procedure and all examination data sheets are available for review at the plant site.

Class 1

One (1) component support from the 2B11 system was visually examined. No unacceptable indication was detected.

Class 2

Twenty-eight (28) component supports from the 2E11, 2E21, 2E41,2E51, and 2T47 systems were visually examined. No unacceptable indications were detected.

E.I. Hatch Unit 2 **Support Components**

ASME Class	Exam Figure No.	Examination/Hanger Type	Location	Elevation	Exam/Cal Sheet No.	Results	Remarks
1	2B11-SK1	- 2B11-A001-S01 REACTOR VESSEL	DRYWELL	114	S95H2V054	SAT	
2	2E11-SK4	- 2E11-B001A-S01 HT EXCH	NE DIAG	112	S95H2V031	SAT	
2	2E11-SK5	- 2E11-C002A-S01 PUMP	NE DIAG	122	S95H2V030	SAT	
2	2E21-SK6	- 2E21-C001A-S01 PUMP	NE DIAG	87	S95H2V055	SAT	
2	2E41-SK7	- 2E41-C001-S01 PUMP	SE DIAG	112	S95H2V021	SAT	
2	2E51-SK8	- 2E51-C001-S01 PUMP	NW DIAG	87	S95H2V033	SAT	
2	2T47-SK9	- 2T47-B007A-S01 COOLING UNIT	DRYWELL	183	S95H2V045	SAT	
2	2T47-SK9	- 2T47-B010A-S01 COOLING UNIT	DRYWELL	114	S95H2V044	SAT	
2	B-101/02	2E11 - RHR-R138 RESTRAINT	SE DIAG	94	S95H2V013	SAT	

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302
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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 **Support Components**

ASME Class	Exam Figure No.	Examination/Hanger Type	Location	Elevation	Exam/Cal Sheet No.	Results	Remarks
2	B-101/02	2E11 - RHR-R139 RESTRAINT	SE DIAG	94	S95H2V014	SAT	
2	B-19/05	2E11 - RHR-H202 HANGER	TORUS	122	S95H2V025	SAT	
2	B-19/05	2E11 - RHR-H203 HANGER	TORUS	122	S95H2V026	SAT	
2	B-19/05	2E11 - RHR-H204 HANGER	TORUS	122	S95H2V027	SAT	
2	B-23/05	2E11 - RHR-R81 RESTRAINT	RWCU HX ROOM	178	S95H2V043	SAT	
2	B-23/05	2E11 - RHR-R82 RESTRAINT	RWCU HX ROOM	178	S95H2V042	SAT	
2	B-27/03	2E11 - RHR-H178 SPRING	NE DIAG	113	S95H2V037	SAT	
2	B-34/06	2E11 - RHR-A50 ANCHOR	TORUS	121	S95H2V034	SAT	
2	B-59/06	2E21 - CS-A33 ANCHOR	SE DIAG	123	S95H2V015	SAT	

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Owner: Georgia Power Company, 323 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30332
 Plant: Edwin I, Hatch Nuclear Plant, Route I, Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

E.I. Hatch Unit 2 **Support Components**

ASME Class	Exam Figure No.	Examination/Hanger Type	Location	Elevation	Exam/Cal Sheet No.	Results	Remarks
2	B-59/06	2E21 - CS-R70 RESTRAINT	SE DIAG	108	S95H2V016	SAT	
2	B-59/06	2E21 - CS-R71 RESTRAINT	SE DIAG	108	S95H2V017	SAT	
2	B-60/05	2E21 - CS-A32 ANCHOR	REACTOR BLDG	162	S95H2V023	SAT	
2	B-60B/03	2E21 - CS-H83 SPRING	SE DIAG	95	S95H2V035	SAT	
2	B-61/05	2E21 - CS-R41 RESTRAINT	TORUS	87	S95H2V029	SAT	
2	B-16/04	2E41 - HPCI-H85 HANGER	NW DIAG	120	S95H2V040	SAT	
2	B-67/03	2E41 - HPCI-A65 ANCHOR	TORUS	120	S95H2V041	SAT	
2	B-68/05	2E41 - HPCI-H70 SPRING	HPCI ROOM	120	S95H2V036	SAT	
2	B-68/05	2E41 - HPCI-R75 RESTRAINT	HPCI ROOM	120	S95H2V018	SAT	

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 3030
 Plant: Edwin I. Hatch Nuclear Plant, Route I., Box 278, Baxley, QA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

E.I. Hatch Unit 2 **Support Components**

ASME Class	Exam Figure No.	Examination/Hanger Type	Location	Elevation	Exam/Cal Sheet No.	Results	Remarks
2	B-68/05	2E41 - HPCI-R79 RESTRAINT	HPCI ROOM	120	S95H2V019	SAT	
2	B- 70A(DRAFT)	2E51 - RCIC-H709 RESTRAINT	TORUS	120	S95H2V056	SAT	

Owner: Ocorgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlenta, Georgia 30302
 Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, QA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

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- 1. Cwner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
- 2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
- Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for 6. National Board No. for Unit N/A

SNUBBER SUPPORT EXAMINATIONS

This section of the report provides a discussion of the visual examination (VT) performed on snubber supports. All safety related snubbers were VT examined by GPC QC personnel using GPC procedure 45QC-INS-012-0S (VT-3).

The subject examinations were performed prior to and during the Refueling/Maintenance Outage. The procedure and all examination data sheets are available for review at the plant site.

Examinations

Class 1

Sixteen (16) snubbers from the 2B21, 2B31, 2E11, and 2E41 systems were visually examined. No unacceptable indications were detected.

Class 2

Thirty-two (32) snubbers from the 2E11, 2E21, 2E51, 2N11, 2T46, and 2T48 systems were visually examined. No unacceptable indications were detected.

Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30°)2
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E. I. Hatch Unit 2 Snubbers

ASME Class	Exam Figure No.	Examination/Hanger Type	Location	Elevation	Results	Remarks
1	A-7/07	2B21-MS-R49A Mechanical Snubber	Drywell	167	Sat	
1	A-7/07	2B21-MS-R49B Mechanical Snubber	Drywell	167	Sat	
1	A-8/07	2B21-MS-R44A Mechanical Snubber	Drywell	150	Sat	
1	A-8/07	2B21-MS-R44B Mechanical Snubber	Drywell	150	Sat	
1	A-17/04	2B31-SS-A20 Hydraulic Snubber	Drywell	138	Sat	Replaced MWO 2-95-2582 Evaluated by GPC Engineering as degraded, not a failure.
1	A-18/05	2B31-SS-B1 Hydraulic Snubber	Drywell	117	Sat	
1	A-18/05	2B31-SS-B2 Hydraulic Snubber	Drywell	125	Sat	Replaced MWO 2-95-1118 Evaluated by GPC Engineering as degraded, not a failure.
1	A-18/05	2B31-SS-B3 Hydraulic Snubber	Drywell	125	Sat	

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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

E. I. Hatch Unit 2 Snubbers

ASME Class	Exam Figure No.	Examination/Hanger Type	Location	Elevation	Results	Remarks
1	A-18/05	2B31-SS-B6 Hydraulic Snubber	Drywell	137	Sat	
1		2B31-S40-H1 Mechanical Snubber	Drywell	120	Sat	
1	A-21/05	2E11-RHR-R342 Mechanical Snubber	Drywell	139	Sat	
1	A-22/05	2E11-RHR-R355 Mechanical Snubber	Drywell	150	Sat	Deleted MWO 2-95-2592 Evaluated by GPC Engineering as degraded, not a failure.
1	A-23/05	2E11-RHR-R350 Mechanical Snubber	Drywell	150	Sat	
1	A-23/05	2E11-RHR-R357A Mechanical Snubber	Drywell	150	Sat	
1	A-26/06	2E41-HPCI-R113B Mechanical Snubber	Drywell	145	Sat	
1	A-26/06	2E41-HPCI-R112 Mechanical Snubber	Drywell	145	Sat	Replaced MWO 2-95-2969 Evaluated by GPC Engineering as degraded, not a failure.

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E. I. Hatch Unit 2 Snubbers

ASME Class	Exam Figure No.	Examination/Hanger Type	Location	Elevation	Results	Remarks
2	B-15/05	2E11-RHR-R292A Mechanical Snubber	Torus	118	Sat	
2	B-17/07	2E11-RHR-R93B Hydraulic Snubber	Torus	118	Sat	
2	B-17/07	2E11-RHR-R705A Mechanical Snubber	Torus	118	Sat	
2	B-18/06	2E11-RHR-R129A Hydraulic Snubber	Torus	123	Sat	
2	B-18/06	2E11-RHR-R129B Hydraulic Snubber	Torus	123	Sat	
2	B-19/05	2E11-RHR-R297A Mechanical Snubber	Torus	118	Sat	
2	B-21/07	2E11-RHR-R100 Hydrautic Snubber	Torus	119	Sat	
2	B-21/07	2E11-RHR-R105A Mechanical Snubber	Torus	118	Sat	
2	B-21/07	2E11-RHR-R106 Mechanical Snubber	Torus	118	Sat	

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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

E. I. Hatch Unit 2 Snubbers

ASME Class	Exam Figure No.	Examination/Hanger Type	Location	Elevation	Results	Remarks
2	B-21/07	2E11-RHR-R708 Mechanical Snubber	Torus	119	Sat	
2	B-26/04	2E11-RHR-R251A Hydraulic Snubber	NE Diag	119	Sat	
2	B-32/04	2E11-RHR-R282 Hydraulic Snubber	SE Diag	103	Sat	
2	B-32/04	2E11-RHR-R285B Hydraulic Snubber	SE Diag	100	Sat	
2	B-40/04	2E11-RHR-R253 Hydraulic Snubber	NE Diag	118	Sat	
2	B-42/06	2E11-RHR-R225 Mechanical Snubber	Torus	90	Sat	
2	B-42/06	2E11-RHR-R230 Mechanical Snubber	Torus	90	Sat	
2	B-60B/03	2E21-CS-R82B Mechanical Snubber	SE Diag	97	Sat	Replaced MWO 2-95-2570 Evaluated by GPC Engineering as degraded, not a failure.
2	B-86/04	2E51-RCIC-R705 Mechanical Snubber	Torus	89	Sat	Replaced MWO 2-95-2571 Evaluated by GPC Engineering as degraded, not a failure.

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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E. I. Hatch Unit 2 Snubbers

ASME Class	Exam Figure No.	Examination/Hanger Type	Location	Elevation	Results	Remarks
C1433	140.	Chairmation Hanger Type	Location	Lievation	Results	Remarks
2	B-6/04	2N11-HPS-R67B Mechanical Snubber	Con Bay	156'10	Sat	
2	B-7/04	2N11-MS-R70 Mechanical Snubber	Con Bay	149	Sat	
2	B-7/04	2N11-MS-R72 Hydraulic Snubber	Con Bay	149	Sat	
2	B-9/05	2N11-MS-R44A Hydraulic Snubber	Con Bay	154	Sat	
2	B-10/07	2N11-MS-R38A Hydraulic Snubber	Con Bay	149	Sat	
2	B-12/06	2N11-MS-R50A Hydraulic Snubber	Con Bay	154	Sat	
2	B-12/06	2N11-MS-R50B Hydraulic Snubber	Con Bay	154	Sat	
2	-	2N11-MS-A60D Mechanical Snubber	Con Bay	153	Sat	
2		2N11-MS-R61B Mechanical Snubber	Con Bay	154	Sat	Replace ² MWO 2-95-3030 Evaluated by GPC Engineering as degraded, not a failure.

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 Plant Urit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

E. I. Hatch Unit 2 Snubbers

ASME Class	Exam Figure No.	Examination/Hanger Type	Location	Elevation	Results	Remarks
2	-	2N11-MS-R62A Mechanical Snubber	Con Bay	154	Sat	Replaced MWO 2-95-3030 Evaluated by GPC Engineering as degraded, not a failure.
2	-	2T46-SGTS-R65B Hydraulic Snubber	A/H Room	197	Sat	
2		2T48-CPUR-R12 Hydraulic Snubber	Rx Bldg.	185	Sat	
2	***	2T48-CPUR-R13 Hydraulic Snubber	Rx Bldg.	185	Sat	
2	-	2T48-CPUR-R55 Hydraulic Snubber	Rx Bldg.	130	Sat	

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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

SUMMARY OF

NON-SAFETY NUREG-0313, REV. 2 INSPECTIONS

E.I. Hatch Unit 2 Class 3 Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results		Remarks
REA	CTOR WATER	CLEAN-UP SYSTEM						
	C-128/02	2G31-3RWCU-4-BD-3	UT-H-400/13	145-H	S95H2U201	NRI		
					S95H2U202	NRI		
NUREG-031	135	ELBOW TO REDUCER			S95H2U203	N/A T/C		
	C-129/01	2G31-3RWCU-4-D-1	UT-H-400/13	145-H	S95H2U154	NRI		
					S95H2U155	NRI		
NUREG-031	138	REDUCER TO TEE			S95H2U156	N/A T/C		
	C-129/01	2G31-3RWCU-4-D-2	UT-H-400/13	145-H	S95H2U137	NRI		
					S95H2U138	NRI		
NUREG-031	135	TEE TO PIPE			S95H2U139	N/A T/C		
	C-126/01	2G31-3RWCU-5-BS-1	UT-H-400/13	153-H	S95H2U195	RI	GEOMETRY	ONE-SIDED EXAM DUE TO FLANGE CONFIGURATION
- 133					S95H2U196	NRI		FEANGE CONFIGURATION
NUREG-031	3S	REDUCER TO PIPE			S95H2U197	N/A T/C		
	C-126/01	2G31-3RWCU-5-BS-2	UT-H-400/13	153-H	S95H2U198	NRI		ONE-SIDED EXAM DUE TO FLANGE CONFIGURATION.
					S95H2U199	NRI		FLANGE CONFIGURATION
NUREG-031	35	PIPE TO FLANGE			S95H2U200	N/A T/C		
	C-125/01	2G31-3RWCU-6-D-1	UT-H-400/13	133-H	S95H2U143	N/A T/C		ONE-SIDED EXAM DUE TO
					S95H2U144	NRI		VALVE CONFIGURATION.
NUREG-031	35	VALVE TO ELBOW			S95H2U145	NRI		

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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
_	C-125/01	2G31-3RWCU-6-D-5	UT-H-400/13	133-H	S95H2U140	NRI	
					S95H2U141	NRI	
NUREG-0	313S	PIPE TO ELBOW			S95H2U142	N/A T/C	

Owner: Georgia Power Company, 3/3 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302
 Plant: Edwin I. Hatch Nuclear Plant, Route 1 Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authoriza, on (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

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- Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302
 Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

SUMMARY OF REACTOR PRESSURE VESSEL INTERNAL INSPECTIONS

E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cai Block	Sheet No.	Results	Remarks
11	N-VESSEL VISUA	AL INSPECTIONS					
B13.21 B-N-2	2-BN-8-1	D-13	VT-H-750/06		S95H2V100	NI	
ASME		TOP GUIDE HOLD DOWNS @ 187.5*					
313.21	2-BN-8-1	D-14	VT-H-750/06		S95H2V100	NI	
S-N-2 ISME		TOP GUIDE HOLD DOWNS @ 202.5"					
313.21 3-N-2	2-BN-8-1	D-15	VT-H-750/06		S95H2V100	NI	
SME		TOP GUIDE HOLD DOWNS @ 217.5°					
313.21	2-BN-8-1	D-16	VT-H-750/06		S95H2V100	NI	
SME		TOP GUIDE HOLD DOWNS @ 232.5°					
13.21 -N-2	2-BN-8-1	D-17	VT-H-750/06		S95H2V100	NI	
SME		TOP GUIDE HOLD DOWNS @ 247.5°					
13.21 -N-2	2-BN-8-1	D-18	VT-H-750/06		S95H2V100	NI	
SME		TOP GUIDE HOLD DOWNS @ 262.5°					

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Owner. Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 303
 Plant: Edwin I, Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13.21	2-BN-8-1	D-19	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		TOP GUIDE HOLD DOWNS @ 277.5*					
B13.21 B-N-2	2-BN-8-1	D-20	VT-H-750/06		S95H2V100	NI	
ASME		TOP GUIDE HOLD DOWNS @ 292.5*					
B13.21 B-N-2	2-BN-8-1	D-21	VT-H-750/06		S95H2V100	NI	
ASME		TOP GUIDE HOLD DOWNS @ 307.5°					
B13.21	2-BN-8-1	D-22	VT-H-750/06		S95H2V100	:#	
asme		TOP GUIDE HOLD DOWNS @ 322.5°					
813.21	2-BN-8-1	D-23	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		TOP GUIDE HOLD DOWNS @ 337.5°					
B13.21	2-BN-8-1	D-24	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		TOP GUIDE HOLD DOWNS @ 352.5°					

Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 39302
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 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

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E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Rema ks
	2-BN-17-1	DRYER	VT-H-750/06		S95H2V100	RI	RE-EXAMINATION OF PREVIOUSLY RECORDED INDICATIONS.
		TEAR ON TOP PLATE NEAR 180° AZIMUTH					INDICATIONS.
	2-BN-6-3	E-22	VT-H-750/06		S95H2V100	NI	
		SHROUD HEAD HOLD DWN BRCKTS ATTACH WELD AZIMUTH 215 DEG					
	2-BN-6-3	E-23	VT-H-750/06		S95H2V100	NRI	
		SHROUD HEAD HOLD DWN BRCKTS ATTACH WELD AZIMUTH 225 DEG					
	2-BN-6-3	E-24	VT-H-750/06		S95H2V100	NI	
		SHROUD HEAD HOLD DWN BRCKTS ATTACH WELD AZIMUTH 235 DEG					
	2-BN-6-3	E-25	VT-H-750/06		S95H2V100	NI	
		SHROUD HEAD HOLD DWN BRCKTS ATTACH WELD AZIMUTH 245 DEG					
	2-BN-6-3	E-26	VT-H-750/06		S95H2V100	NRI	
		SHROUD HEAD HOLD DWN BRCKTS ATTACH WELD AZIMUTH 255 DEG					

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E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
	2-BN-6-3	E-27	VT-H-750/06		S95H2V100	NI	
		CURCUR UEAR HOLD DAM					
		SHROUD HEAD HOLD DWN BRCKTS ATTACH WELD AZIMUTH 265 DEG					
	2-BN-6-3	E-28	VT-H-750/06		S95H2V100	Ni	
		SHROUD HEAD HOLD DWN BRCKTS ATTACH WELD AZIMUTH 275 DEG					
**	2-BN-6-3	E-29	VT-H-750/06		S95H2V100	NI	
		SHROUD HEAD HOLD DWN BRCKTS ATTACH WELD AZIMUTH 285 DEG					
	2-BN-6-3	E-30	VT-H-750/06		S95H2V100	NI	
		SHROUD HEAD HOLD DWN BRCKTS ATTACH WELD AZIMUTH 295 DEG					
B13.22	2-BN-8-2	F-25 42-43	VT-H-750/06		S95H2V100	NI	
B-N-2							
ASME		CORE TOP GUIDE CELLS					
B13.22	2-BN-8-2	F-26 42-47	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		CORE TOP GUIDE CELLS					

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1. Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302

2. Plant: Edwin I. Hatch Nuclear Plant, Route I, Box 278, Baxley, GA 31513.

3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.

5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13 22 B-N-2	2-BN-8-2	F-31 46-43	VT-H-750/06		S95H2V100	NI	
ASME		CORE TOP GUIDE CELLS					
B13.22	2-BN-8-2	F-58 42-07	VT-H-750/06		S95H2V100	NI .	
B-N-2 ASME		CORE 1/P GUIDE CELLS					
B13.22	2-BN-8-2	F-59 42-11	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		CORE TOP GUIDE CELLS			S95H2V100	NI	
B13.22	2-BN-8-2	F-63 46-11	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		CORE TOP GUIDE CELLS					
B13.22	2-BN-8-2	F-78 10-11	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		CORE TOP GUIDE CELLS					
B13.22	2-BN-8-2	F-108 06-43	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		CORE TOP GUIDE CELLS					

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E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13.22 B-N-2	2-BN-8-2	F-112 10-43	VT-H-750/06		S95H2V100	NI	
ASME		CORE TOP GUIDE CELLS					
B13 22	2-BN-8-2	F-113 10-47	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		CORE TOP GUIDE CELLS					
B13.10	2-BN-3-3	G-8	VT-H-750/06		S95H2V100	NI	
B-N-1 ASME		EXAMINATION OF VESSEL INTERIOR					
B13.22 B-N-2	2-BN-6-4	H-9 @ 135°	VT-H-750/06		S95H2V100	NI	
SIL-572		SHROUD SUPPORT LEDGE TO RPV WELD					
B13.22 B-N-2	2-BN-6-4	H-9 @ 225°	Vî-H-750/06		S95H2V100	NI	
SIL-572		SHROUD SUPPORT LEDGE TO RPV WELD					
B13.22 B-N-2	2-BN-6-4	H-9 @ 315*	VT-H-750/06		S95H2V100 S95H2V100	NI NI	
SIL-572		SHROUD SUPPORT LEDGE TO RPV WELD			S95H2V100	NI	

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 303
 Plant: Edwin I. Hatch Nuclear Plant. Route 1. Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13 22	2-BN-6-4	H-9 @ 45°	VT-H-750/06		S95H2V100	NI	
B-N-2							
SIL-572		SHROUD SUPPORT LEDGE TO RPV WELD					
B13.22	2-BN-6-4	H-10 ID	VT-H-750/06		S95H2V100	NI	
B-N-2 SIL-572		VERT WELD 230 DEG WELD V-2					
B13.22	2-BN-6-4	H-14 OD	VT-H-750/06		S95H2V100	NI	
B-N-2	2-014-0-4	11-14-00					
SIL-572		VERT WELD 230 DEG WELD V-2					
B13.22	2-BN-6-4	H-13 OD	VT-H-750/06		S95H2V100	NI	
3-N-2							
SIL-572		VERT WELD 230 DEG WELD V-2					
B13 22	2-BN-6-4	H-15 OD	VT-H-750/06		S95H2V100	NI	
3-N-2							
SIL-572		VERT WELD 230 DEG WELD V-2					
B13.22	2-BN-6-4	H-10 OD	VT-H-750/06		S95H2V100	NI	
B-N-2							
SIL-572		VERT WELD 230 DEG WELD V-2					

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1. Owner: Georgia Power Company, 333 Piedmont Ave., NF. P. O. Box 4545, Atlanta, Georgia 30302

2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxdey, GA 31513,

3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.

5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 **IVVI Components**

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13.22 B-N-2	2-BN-6-4	H-13 ID	VT-H-750/06		S95H2V100	NRI	
SIL-572		VERTICAL WELD 50 DEG WELD V- 5					
B13.22 B-N-2	2-BN-6-4	H-14 ID	VT-H-750/06		S95H2V100	NI	
SIL-572		VERT WELD 230 DEG WELD V-6					
B13.22 B-N-2	2-BN-6-4	H-15 ID	VT-H-750/06		S95H2V100	NI	
SIL-572		VERT WELD 140 DEG WELD V-7					
B13.22	2-BN-6-4	Q-1	VT-H-750/06		S95H2V101	NI	
B-N-2 SIL-572		SHROUD RESTRAINT NO.1 @ 45° SHROUD RESTRAINT ASSEMBLY					
B13.22 B-N-2	2-BN-6-4	Q- 2	VT-H-750/06		S95H2V101	NI	
SIL-572		SHROUD RESTRAINT NO.1 @ 45° CLEVIS PIN CONTACT AREA					
B13 22 B-N-2	2-BN-6-4	Q-3	VT-H-750/06		S95H2V101	NI	
SIL-572		SHROUD RESTRAINT NO.1 @ 45° LOW SPRNG CONTACT AREA TO SHROUD					

Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 3030
 Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 99/05/79 6. National Board No. for Unit N/A.

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E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13 22 B-N-2	2-BN-6-4	Q-4	VT-H-750/06		S95H2V101	NI	
SIL-572		SHROUD RESTRAINT NO.1 @ 45° LOWER SPRING CONTACT AREA TO RPV					
D42.22	2-BN-6-4	Q-5	VT-H-750/06		S95H2V101	NI	
B13 22 B-N-2	2-014-0-4						
SIL-572		SHROUD RESTRAINT NO.1 @ 45° MID SUPPORT CONTACT AREA TO RPV					
	2-BN-6-4	Q-6	VT-H-75		S95H2V101	NI	
B13.22 B-N-2	Z-BN-0-4	Q- 6					
SIL-572		SHROUD RESTRAINT NO 1 @ 45° UP STAB CONTACT AREA TO SHROUD					
B13.22	2-BN-6-4	Q-7	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.1 @ 45° UPPER STAB. CONTACT AREA TO RPV					
B13.22	2-BN-6-4	Q-8	VT-H-750/06		S95H2V101	NI	
B-N-2	201101						
SIL-572		SHROUD RESTRAINT NO.1 @ 45° CORE PLATE WEDGE ASSEMBLY					
B13.22	2-BN-6-4	Q-9	VT-H-750/06		S95H2V101	NI	
B-N-2	2-014-0-4						
SIL-572		SHROUD RESTRAINT NO.2 @ 135° SHROUD RESTRAINT ASSEMBLY					

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1. Owner: Qeorgia Power Company, 333 Piedmont Ave., NE. P. Q. Box 4545, Atlanta, Georgia 39302

2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.

3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.

5. Commercial Service Date 99/05/79 6. National Board No. for Unit N/A.

E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13.22	2-BN-6-4	Q-10	VT-H-750/06		S95H2V101	Ni	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.2 @ 135°_ CLEVIS PIN CONTACT AREA					
042.22	2-BN-6-4	0-11	VT-H-750/06		S95H2V101	NI	
B13.22 B-N-2	Z-BN-0-4	4-11					
SIL-572		SHROUD RESTRAINT NO.2 @ 135° SPRING CONTACT AREA TO SHROUD					
B13.22	2-BN-6-4	Q-12	VT-H-750/06		S95H2V101	NI	
B-N-2	201104						
SIL-572		SHROUD RESTRAINT NO.2 @ 135° LOW SPRING CONTACT AREA TO RPV					
		Q-13	VT-H-750/06		S95H2V101	NI	
B13.22	2-BN-6-4	Q-13					
B-N-2 SIL-572		SHROUD RESTRAINT NO 2 @ 135° MID SUPPORT CONTACT AREA TO RPV					
			VT-H-750/06		S95H2V101	NI	
B13.22	2-BN-6-4	Q-14	V 1-N-750/00				
B-N-2							
SIL-572		SHROUD RESTRAINT NO.2 @ 135° UP STAB CONTACT AREA TO SHROUD					
B13.22	2-BN-6-4	Q-15	VT-H-750/06		S95H2V101	NI	
B-N-2	201101						
SIL-572		SHROUD RESTRAINT NO.2 @ 135* UPPER STAB. CONTACT AREA TO RPV					

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1. Owner: Georgia Fower Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 3030;

2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.

3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.

5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Sheet No.	Results	Remarks
B13.22 B-N-2	2-BN-6-4	Q-16	VT-H-750/06		S95H2V101	NI	
SIL-572		SHROUD RESTRAINT NO.2 @ 135* CORE PLATE WEDGE ASSEMBLY					
B13.22	2-BN-6-4	Q-17	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.3 @ 225° SHROUD RESTRAINT ASSEMBLY					
B13 22	2-BN-6-4	Q-18	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.3 @ 225° CLEVIS PIN CONTACT AREA					
B13.22	2-BN-6-4	Q-19	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.3 @ 225° LOW SPRG CONTACT AREA TO SHROUD					
B13 22 B-N-2	2-BN-6-4	Q-20	VT-H-750/06		S95H2V101	NI	
SIL-572		SHROUD RESTRAINT NO.3 @ 225° LOW SPRING CONTACT AREA TO RPV					
B13.22	2-BN-6-4	Q-21	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.3 @ 225° MID SUPPORT CONTACT AREA TO RPV					

Page: 69
1. Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302
2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
5. Commercial Service Date 09/03/72 6. National Board No. for Unit N/A

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13.22 B-N-2	2-BN-6-4	Q-22	VT-H-750/06		S95H2V101	NI	
SIL-572		SHROUD RESTRAINT NO.3 @ 225° UP STAB CONTACT AREA TO SHROUD					
B13 22	2-BN-6-4	Q-23	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.3 @ 225° UPPER STAB. CONTACT AREA TO RPV					
B13.22	2-BN-6-4	Q-24	VT-H-750/06		S95H2V101	NI	
B-N-2	2-014-0-4	424					
SIL-572		SHROUD RESTRAINT NO.3 @ 225° CORE PLATE WEDGE ASSEMBLY					
042.22	2-BN-6-4	Q-25	VT-H-750/06		S95H2V101	NI	
B13.22 B-N-2	2-BN-0-4	G-25	***************************************				
SIL-572		SHROUD RESTRAINT NO.4 @ 315° SHROUD RESTRAINT ASSEMBLY					
		0.20	VT-H-750/06		S95H2V101	NI	
B13.22 B-N-2	2-BN-6-4	Q-26	V1-1-7-50700		0301127101		
SIL-572		SHROUD RESTRAINT NO.4 @ 315° CLEVIS PIN CONTACT AREA					
B13.22	2-BN-6-4	Q-27	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.4 @ 315° LOW SPRG CONTACT AREA TO SHROUD					

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1. Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302

2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.

3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.

5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13.22	2-BN-6-4	Q-28	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO 4 @ 315° LOW SPRING CONTACT AREA TO RPV					
B13.22	2-BN-6-4	Q-29	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.4 @ 315° MID SUPPORT CONTACT AREA TO RPV					
B13 22	2-BN-6-4	Q-30	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.4 @ 315° UP STAB CONTACT AREA TO SHROUD					
B13.22	2-BN-6-4	Q-31	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.4 @ 315° UPPER STAB. CONTACT AREA TO RPV					
B13.22	2-BN-6-4	Q-32	VT-H-750/06		S95H2V101	NI	
B-N-2							
SIL-572		SHROUD RESTRAINT NO.4 @ 315° CORE PLATE WEDGE ASSEMBLY					
	2-BN-4-8	J-1	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NI	
SIL-574		RESTRAINER FOR JET PUMPS 1 & 2 AZIMUTH 30 DEG					

Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. Q. Box 4545, Atlanta, Georgia 3030;
 Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.

Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
	2-BN-4-8	J-17	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NI	
SIL-574		RESTRA:NER FOR JET PUMPS 3 AND 4 AZIMUTH 60 DEG					
			VT-H-750/06		S95H2V100	NI	
**	2-BN-4-8	J-33	711113333		S95H2V100	NI	
SIL-574		RESTRAINER FOR JET PUMPS 5 AND 6 AZIMUTH 90 DEG					
			VT-H-750/06		S95H2V100	NI	
	2-BN-4-8	J-49	¥1-11-730/00		S95H2V100	NI	
SIL-574		RESTRAINER FOR JET PUMPS 7 AND 8 AZIMUTH 120 DEG					
	2-BN-4-8	J-57	VT-H-750/06		S95H2V100	NI	
SIL-420		JET PUMP 7 SENSING LINE AZIMUTH 120 DEG					
	2-BN-4-8	J-58	VT-H-750/06		S95H2V100	NI	
SIL-420		JET PUMP 7 SENSING LINE UP SPRT BRKT AZIMUTH 120 DEG					
	2-BN-4-8	J-59	VT-H-750/06		S95H2V100	NI NI	
SIL-420		JET PUMP 7 SENSING LINE LOW SPRT BRKT AZIMUTH 120 DEG					

^{1.} Owner: Georgia Power Comusery, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 3030

2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.

3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.

5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cai Block	Sheet No.	Results	Remarks
	2-BN-4-8	J-65	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NI	
SIL-574		RESTRAINER FOR JET PUMPS 9 AND 10 AZIMUTH 150 DEG					
	2-BN-4-8	J-81	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NI	
SIL-574		RESTRAINER FOR JET PUMPS 11 & 12 AZIMUTH 210 DEG					
313.20	2-BN-4-2	J-81A	VT-H-750/06		S95H2V100	NI	
3-N-2	2-014-4-2	J-01A			S95H2V100	NI	
ASME		RISER BRACE SUPPORT PADS TO VESSEL WLD FOR JET PUMPS 11 & 12					
	2-BN-4-8	J-81B	VT-H-750/06		S95H2V100	NI	
	Z-BN-4-0	3-616			S95H2V100	NI	
SIL-551		RISER BRACE ARM FOR JET PUMPS 11 & 12					
	2-BN-4-8	J-97	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NRI	
SIL-574		RESTRAINER FOR JET PUMPS 13 & 14 AZIMUTH 240 DEG					
313.20	2-BN-4-2	J-97A	VT-H-750/06		S95H2V100	NI	
5-N-2	201112				S95H2V100	NI	
ASME		RISER BRACE SUPPORT PADS TO VESSEL WLD FOR JET PUMPS 13 & 14					

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1. Owner: Georgia Fower Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302

2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.

3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.

5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
	2-BN-4-8	J-97B	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NI	
SIL-551		RISER BRACE ARM FOR JET PUMPS 13 & 14					
	2-BN-4-8	J-113	VT-H-750/06		S95H2V100	NI	
- 1					S95H2V100	NI	
SIL-574		RESTRAINER FOR JET PUMPS 15 & 16 AZIMUTH 270 DEG					
113.20	2-BN-4-2	J-113A	VT-H-750/06		S95H2V100	NI	
-N-2	Z-DI4-4-Z	0.1100			S95H2V100	NI	
ASME		RISER BRACE SUPPORT PADS TO VESSEL WLD FOR JET PUMPS 15 & 16					
	2-BN-4-8	J-113B	VT-H-750/06		S95H2V100	NI	
	2-011-1-0				S95H2V100	Ni	
IL-551		RISER BRACE ARM FOR JET PUMPS 15 & 16					
	2-BN-4-8	J-129	VT-H-750/06		S95H2V100	NI	
	Z-DI4-4-0	0.123			S95H2V100	NI	
IL-574		RESTRAINER FOR JET PUMPS 17 AND 18 AZIMUTH 300 DEG					
313.20	2-BN-4-2	J-129A	VT-H-750/06		S95H2V100	NI	
-N-2					S95H2V100	NI	
SME		RISER BRACE SUPPORT PADS TO VESSEL WLD FOR JET PUMPS 17 & 18					

Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 3030
 Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A Page: 74

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
		LAGIIIIIGUVIDAIVG					
**	2-BN-4-8	J-129B	VT-H-750/06		S95H2V100	NI	
444					S95H2V100	NI	
SIL-551		RISER BRACE ARM FOR JET PUMPS 17 & 18					
	2-BN-4-8	J-137	VT-H-750/06		S95H2V100	NI	
**	Z-DIV-4-0	3-131			S95H2V100	N;	
SIL-420		JET PUMP 17 SENSING LINE AZIMUTH 300 DEG					
			VIT 11 750/00		S95H2V100	NI	
	2-BN-4-8	J-138	VT-H-750/06			NI	
					S95H2V100	NI	
SIL-420		JET PUMP 17 SENSING LINE UP SPRT BRKT AZIMUTH 300 DEG					
	2.001.4.5	J-139	VT-H-750/06		S95H2V100	NI	
	2-BN-4-8	3-139	V1-11-130100		S95H2V100	NI	
SIL-420		JET PUMP 17 SENSING LINE LOW SPRT BRKT AZIMUTH 300 DEG					
	2-BN-4-8	J-145	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NI	
SIL-574		RESTRAINER FOR JET PUMPS 19 & 20 AZIMUTH 330 DEG					
B13.20	2-BN-4-2	J-145A	VT-H-750/06		S95H2V100	NI	
B-N-2	2011-12				S95H2V100	NI	
ASME		RISER BRACE SUPPORT PADS TO VESSEL WLD FOR JET PUMPS 19 & 20					

Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 3030.
 Plant: Edwin I, Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513,
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Data 09/05/79 6. National Board No. for Unit N/A.

E.I. Hatch Unit 2 **IVVI Components**

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
	2-BN-4-8	J-1458	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NI	
SIL-551		RISER BRACE ARM FOR JET PUMPS 19 & 20					
	2-BN-17-1	K-1	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NI	
		STEAM DRYER LIFTING EYE AND ROD AZIMUTH 35 DEG					
	2-BN-17-1	K-2	VT-H-750/06		S95H2V100	RI	RE-EXAMINATION OF
	Z-DI4-11-1	N-2			S95H2V100	NI	PREVIOUSLY RECORDED
		STEAM DRYER LIFTING EYE AND ROD AZIMUTH 145 DEG					INDICATIONS.
	2-BN-17-1	K-3	VT-H-750/06		S95H2V100	RI	RE-EXAMINATION OF
	2-014-17-1	7.5			S95H2V100	NI	PREVIOUSLY RECORDED
		STEAM DRYER LIFTING EYE AND ROD AZIMUTH 215 DEG					INDICATIONS.
	2-BN-17-1	K-4	VT-H-750/06		S95H2V100	RI	RE-EXAMINATION OF
	Z-BN-17-1	K-4	***************************************		S95H2V100	NI	PREVIOUSLY RECORDED
		STEAM DRYER LIFTING EYE AND ROD AZIMUTH 325 DEG					INDICATIONS.
	2-BN-17-1	K-5	VT-H-750/06		S95H2V100	RI	RE-EXAMINATION OF
					S95H2V100	NI	PREVIOUSLY RECORDED INDICATIONS
		STEAM DRYER SUPPORT BRACKET AZIMUTH 34 DEG			S95H2V100	NI	INDICATIONS.

^{1.} Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302

2. Plant: Edwin I. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.

3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.

5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
	2-BN-17-1	K-6	VT-H-750/06		S95H2V100	RI	RE-EXAMINATION OF
**					S95H2V100	RI	PREVIOUSLY RECORDED INDICATIONS
		STEAM DRYER SUPPORT BRACKET AZIMUTH 146 DEG					INDICATIONS.
	2-BN-17-1	K-7	VT-H-750/06		S95H2V100	RI	RE-EXAMINATION OF PREVIOUSLY RECORDED
					S95H2V100	NRI	INDICATIONS.
		STEAM DRYER SUPPORT BRACKET AZIMUTH 214 DEG			S95H2V100	NRI	
	2-BN-17-1	K-8	VT-H-750/06		S95H2V100	RI	RE-EXAMINATION OF
					S95H2V100	NI	PREVIOUSLY RECORDED
		STEAM DRYER SUPPORT BRACKET AZIMUTH 326 DEG					INDICATIONS.
	2-BN-18-3	L-11	VT-H-750/06		S95H2V100	NI	
		MOISTURE SEPARATOR LOWER SUPPORT BLOCK O° THRU 360°					
	2-BN-18-3	L-12	VT-H-750/06		S95H2V100	NI	
**		MOIST SEP LWR SPRT BLOCK GUSSETS & ATTACHMENT WELDS 1 - 36					
**	2-BN-7-5	M-1A	VT-H-750/06		S95H2V100 S95H2V100	NRI NI	
NUREG CR	-4523	UPPER CORE SPRAY A PPG, NOZ & BRCKTS - 270 TO 90 DG					

Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 3030
 Plant: Edwin I. Hatch Nuclear Plant, Route 1. Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
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E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
**	2-BN-7-5	M-1B	VT-H-750/06		S95F ?V100	RI	SEE INF # IS5H2003. LINEAR
					S95H2V100	RI	INDICATIONS ON THE
NUREG CF	R-4523	LOWER CORE SPRAY B PPG, NOZ			S95H2V100	RI	BRACKET TO SHROUD ATTACHMENT WELDS
		& BRCKTS - 270 TO 90 DG			S95H2V100	NI	ACCEPTABLE AS IS PER GE
					S95H2V100	RI	EVALUATION.
	2-BN-7-5	M-1C	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NI	
NUREG CR	R-4523	UPPER CORE SPRAY C PPG, NOZ & BRCKTS - 90 TO 270 DG			S95H2V100	NI	
	2-BN-7-5	M-1D	VT-H-750/06		S95H2V100	NI	SEE INF # 195H2003 LINEAR
					S95H2V100	NI	INDICATIONS ON THE
NUREG CR	1-4523	LOWER CORE SPRAY D PPG,			S95H2V100	NI	BRACKET TO SHROUD ATTACHMENT WELDS.
		NOZ & BRCKTS - 90 TO 270 DG			S95H2V100	RI	ACCEPTABLE AS IS PER GE
					S95H2V100	NI	EVALUATION.
					S95H2V100	RI	
					S95H2V100	NI	
					S95H2V100	RI	
					S95H2V100	RI	
					S95H2V100	NI	
					S95H2V100	RI	
					S95H2V100	NI	
					S95H2V100	RI	
					S95H2V100	RI	
	2-BN-7-5	M-2A	VT-H-750/06		S95H2V100	NI	
NUREG CR	-4523	UPPER CORE SPRAY INLET T- BOX AT 10 DEGREES TO SHROUD					

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2. Plant: Edwin I. Hatch Nuclea, Plant, Route 1, Box 278, Baxley, GA 31513.

3. Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.

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E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
	2-BN-7-5	M-2B	VT-H-750/06		S95H2V100	NI	
NUREG CR	R-4523	UPPER CORE SPRAY INLET T- BOX AT 170 DEGREES TO SHROUD					
	2-BN-7-5	M-2C	VT-H-750/06		S95H2V100	NI	
NUREG CR	₹-4523	LOWER CORE SPRAY INLET T- BOX AT 190 DEGREES TO SHROUD					
	2-BN-7-5	M-2D	VT-H-750/06		S95H2V100	NI	
NUREG CR	4523	LOWER CORE SPRAY INLET T- BOX AT 350 DEGREES TO SHROUD					
	2-BN-7-1	N-1A	VT-H-750/06		S95H2V100	NI	
NUREG CR	4523	CORE SPRY INTERN PPG FROM INLET @ 90 (2N5B) TO JUNC BOX					
	2-BN-7-1	N-1B	VT-H-750/06		S95H2V100	NI	
NUREG CR	-4523	CORE SPRY INTERN PPG FROM INLET @ 270 (2N5A) TO JUNC BOX					

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E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
	2-BN-7-1	N-2A	VT-H-750/06		S95H2V100	NI	SEE INF # 195H2002 LINEAR
					S95H2V100	NI	INDICATION 1/2 INCH LONG. ACCEPTABLE AS IS PER GE
NUREG CE	R-4523	CORE SPRY INTERN PPG FROM			S95H2V100	NI	LETTER G-GPC-5-118 DATED
		JUNC BOX @ 90 TO SHROUD @ 10			S95H2V100	NRI	OCTOBER 6, 1995
					S95H2V100	NI	
					S35H2V100	RI	
	2-BN-7-1	N-2B	VT-H-750/06		S95H2V100	NI	
					S95H2V100	NI	
NUREG CF	R-4523	CORE SPRY INTERN PPG FROM JUNC BOX @ 90 TO SHROUD @ 170			S95H2V100	NI	
	2-BN-7-1	N-3A	VT-H-750/06		S95H2V100	NI	
	2 0				S95H2V100	NI	
NUREG CF	R-4523	CORE SPRY INTERN PPG FROM JUNC BOX @ 270 TO SHROUD @ 190					
	2-BN-7-1	N-3B	VT-H-750/06		S95H2V100	NI	
**	Z-DN-7-1	14-30			S95H2V100	NI	
NUREG CF	R-4523	CORE SPRY INTERN PPG FROM JUNC BOX @ 270 TO SHROUD @ 350					
B13.20	2-BN-7-2	N-4A	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		CORE SPRAY SUPPORT BRCKT ATTACH WELDS TO VESSEL @ 30					

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ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13 21	2-BN-7-2	N-4B	VT-H-750/06		S95H2V100	NI	
B-N-2							
ASME		CORE SPRAY SUPPORT BRCKT ATTACH WELDS TO VESSEL @ 150					
B13.20	2-BN-7-3	N-4C	VT-H-750/06		S95H2V100	NI	
B-N-2							
ASME		CORE SPRAY SUPPORT BRCKT ATTACH WELDS TO VESSEL @ 210					
B13.21	2-BN-7-3	N-4D	VT-H-750/06		S95H2V100	NI	
B-N-2							
ASME		CORE SPRAY SUPPORT BRCKT ATTACH WELDS TO VESSEL @ 330					
	A-1/05	N-5A	VT-H-750/06		S95H2V100	NI	
4							
NUREG CR	1-4523	A LOOP CORE SPRAY INLET NOZZLE N5A					
	A-1/05	N-5B	VT-H-750/06		S95H2V100	NI	
NUREG CR	2-4523	B LOOP CORE SPRAY INLET NOZZLE N5B					
	2-BA-1	0-3E	VT-H-750/06		S95H2V100	NI	
NUREG-06	19	FEEDWATER NOZZLE INNER RADIUS @ 225 DEG (2N4C)					

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E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	No.	Examination/Area	Examination Procedure	Cal Block	Sheet No.	Results	Remarks
	2-BA-1	0-4E	VT-H-753/06		S95H2V100	NI	
NUREG-061	1.3	FEEDWATER NOZZLE INNER RADIUS @ 315 DEG (2N4D)					
B 1.22	2-BN-14-4	P-4 46-35	VT-H-750/06		S95H2V100	NI	
B-N-2							
ASME		FUEL SUPPORT CASTINGS					
B13.22	2-BN-14-4	P-4 49-16	VT-H-750/96		S95H2V100	NI	
3-N-2							
ISME		FUEL SUPPORT CASTINGS					
313.22	2-BN-14-4	P-4 15-04	VT-H-750/06		S95H2V100	NI	
I-N-2							
SME		FUEL SUPPORT CASTINGS					
313.22	2-BN-14-4	P-4 07-46	VT-H-750/06		S95H2V100	NI	
I-N-2							
SME		FUEL SUPPORT CASTINGS					
313.22	2-BN-14-4	P-4 07-08	VT-H-750/06		S95H2V100	NI	
3-N-2							
SME		FUEL SUPPORT CASTINGS					

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5. Commercial Service Date 99/05/79 6. National No. for Unit N/A.

E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13 22 B-N-2	2-BN-14-4	P-4 03-38	VT-H-750/06		S95H2V100	NI	
ASME		FUEL SUPPORT CASTINGS					
B13 22	2-BN-14-4	P-4 03-16	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		FUEL SUPPORT CASTINGS					
B13.22	2-BN-14-4	P-4 37-04	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		FUEL SUPPORT CASTINGS					
B13.22	2-BN-14-4	P-4 37-50	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		FUEL SUPPORT CASTINGS					
B13.22	2-BN-14-4	P-4 50-19	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		FUEL SUPPORT CASTINGS					
B13.22	2-BN-14-4	P-4 45-46	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		FUEL SUPPORT CASTINGS					

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Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 30302
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E.I. Hatch Unit 2 **IVVI** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
313.22	2-BN-14-4	P-4 50-35	VT-H-750/06		S95H2V100	NI	
3-N-2							
SME		FUEL SUPPORT CASTINGS					
13.22	2-BN-14-4	P-4 49-38	VT-H-750/06		S95H2V100	NI	
I-N-2							
SME		FUEL SUPPORT CASTINGS					
113.22	2-BN-14-4	P-4 38-07	VT-H-750/06		S95H2V100	NI	
-N-2							
SME		FUEL SUPPORT CASTINGS					
313.22	2-BN-14-4	P-4 38-47	VT-H-750/06		S95H2V100	NI	
-N-2							
SME		FUEL SUPPORT CASTINGS					
13.22	2-BN-14-4	P-4 46-15	VT-H-750/06		S95H2V100	NI	
-N-2							
SME		FUEL SUPPORT CASTINGS					
13.22	2-BN-14-4	P-4 46-19	VT-H-750/06		S95H2V100	NI	
N-2							
SME		FUEL SUPPORT CASTINGS					

Owner: Georgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta, Georgia 3030.
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E.I. Hatch Unit 2 **IVV!** Components

ASME Section XI	Exam Figure No.	Examination/Area	Examination Procedure	Cal Block	Exam/Cal Sheet No.	Results	Remarks
B13 22	2-BN-14-4	P-4 15-50	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		FUEL SUPPORT CASTINGS					
B13.22	2-BN-14-4	P-4 46-39	VT-H-750/06		S95H2V100	NI	
S-N-2 ASME		FUEL SUPPORT CASTINGS					
B13.22	2-BN-14-4	P-4 50-23	VT-H-750/06		S95H2V100	NI	
S-N-2 ASME		FUEL SUPPORT CASTINGS					
313.22	2-BN-14-4	P-4 50-27	VT-H-750/06		S95H2V100	NI	
3-N-2 ASME		FUEL SUPPORT CASTINGS					
B13 22	2-BN-14-4	P-4 45-08	VT-H-750/06		S95H2V100	NI	
B-N-2 ASME		FUEL SUPPORT CASTINGS					

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Owner: Qeorgia Power Company, 333 Piedmont Ave., NE. P. O. Box 4545, Atlanta. Georgia 30,07.
 Plant: Edwin I. Hatch Nuclear Plant. Route I. Box 278, Baxley, GA 31513.
 Plant Unit 2. 4. Owner Certificate of Authorization (if req.) N/A.
 Commercial Service Date 09/05/79 6. National Board No. for Unit N/A.

- Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
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SUMMARY OF CLASS 1 AND 2

REPAIRS AND REPLACEMENTS

- 1. Owner: Georgia Power Company, 333 Piedmont Ave., NE, P. O. Box 4545, Atlanta, Georgia 30302
- 2. Plant: Edwin J. Hatch Nuclear Plant, Route 1, Box 278, Baxley, GA 31513.
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REPAIRS AND REPLACEMENTS

GPC procedure 42EN-ENG-014-0S provides guidelines for determining the ASME Section XI, repair/replacement requirements at E. I. Hatch Nuclear Plant. The site Repair/Replacement Coordinator maintains an itemized listing of Class 1 and 2 repair/replacement activities for each cycle. The following tables provide an itemized list of those components which were included in the ASME Section XI Repair and Replacement Program. Copies of the individual Repair/Replacement evaluation sheets are filed with the MWO Packages and are available for review at the site.

1. Owner: Georgia Power Company, 333 Piedmont Ave., NE, P.O. Box 4545, Atlanta, Ga 30302 2. Plant: Edwin I Hatch Nuclear Plant, Rte. 1 Box 278, Baxley, Ga 31513 3. Plant Unit 2 4. Owner Certificate of Authorization (if req.) N/A 5. Commercial Service Date 09/05/79 6. National Board No. for Unit N/A

MPL Number	ASME/ Section XI Class	Description	UNIT 2 FALL 1995 Evaluation	MW/O/DCR	DC Number	Pressure Test
2B11	1/1	Install shroud support system consisting of 4 tie rod assemblies and wedges to ensure shroud structural integrity if welds H1 through H8 fail. Welds H1 through H8 are subject to cracking due to IGSCC.	The core shroud stabilizer design has been submitted to the NRC per letter dated 7-3-95. The design will ensure shroud integrity upon weld failure. Note. in NRC letter dated 9-25-95, NRC approved this repair.	2-95-02853/94-052	N/A	VT-1, VT-3
2B21-F010B	1/1	Install new disc to facilitate proper allignment of hinge pins. Also change pressure seal (bonnet) from old metal style to new grafoil style.	Exact replacement disc will be used, failure is attributed to service system wear. The new replacement seal is an equivalent replacement, and the new cover to be installed is also. This is an upgrade of parts for improved service.	2-95-03015	9503996	VT-1, VT-3, Operating Pressure (N-416-1)
2B21-F010B	1/1	Replace valve components including hinge pins, disc, pressure seals, spring guides, and bushing bearing inserts.	The components used for the replacement of the disc, hinge pins, pressure seals, spring guides, and bushing bearing inserts are exact replacement parts from an approved vendor. This failure is attributed to normal system wear. This is a LLRT valve thus it is evaluated for wear on an established cycle designed to predict unacceptable result prior to valve failure. Therefore this is a suitable repair.	2-95-03257	95-04419	VT-1, VT-3, Operating Pressure Test, N-416-1
2B21-F013H	1/1	Remove valve and send to Wyle Labs for testing and repair as necessary.	Valve is to be repaired under appreoved maintenance program at Wyle Labs. The removal, testing and repair of this valve is a standard preventative measure, and is an acceptable method for ensuring proper valve operation.	2-95-00177	2-94-02476	VT-3, VT-1, Operating Pressure Test
2B21-MSRV-R62	1/1	Remove snubber R62 and reinforce wide beam.	This work is being performed under an approved DCR. No failure has occurred. This is being done as part of	2-95-00141/94-016	N/A	None

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	ASME/		UNIT 2 FALL 1995			
MPL Number	Section XI Class	Description	Evaluation	MWO/DCR	DC Number	Pressure Test
			the reevaluation of snubbers.			
2B31	1/1	Install 2 new 4 inch decon ports	This modification is evaluated and approved under the DCR.	2-95-1654/2H91-03 0	N/A	Volumetric, Surface, VT, VT-1, VT-3, Operating Pressure Test (N-416-1)
2B31-F052A	1/1	Valve had seat and disc damage which is attributed to service induced wear.	Replace with like kind valve meeting specifications of orginal valve.	2-95-03469 R2	95-04701	Surface, VT, Operating Pressure Test, N-416-1
2C41-F029A/B	2/2	Replace flanged relief valves as specified by MDC 93-5041.	This replacement is being performed under an approved MDC for improvement of system reliability. Suitability evaluation is covered by MDC 93-5041. No service induced failure involved. Valves are being replaced due to poor reliability, maintenance history and availability of replacement parts and valves of existing relief valves.	2-94-00633/93-5041	N/A	Surface, Visual, Hydrostatic Pressure Test.
2C51	1/1	Install new ECP/LPRM in the vessel to monitor the effectiveness of the Hydrogen Injection System for protecting the core of the reactor vessel. Assembly is flanged to the bottom of vessel.	No failure occurred. The LPRM assembly is fabricated in accordance with class 1 material.	2-95-02111/93-059	N/A	Operating Pressure Test
2E11	2/2	Cut and cap piping to remove RHR steam condensing mode. This is a eight inch valve.	Due to valve leakage RHR steam condensing mode will be removed. All aspects of this work has been evaluated and approved by DCR 2H94-032.	2-95-02301/2H94-0 32	N/A	VT, Volumetric, Surface, Operating Pressure Test, N-416-1
2E11	2/2	Cut and cap piping to remove RHR steam condensing mode.	Due to valve leakage RHR steam condensing mode will be removed. All aspects of this work has been evaluated and approved by DCR 2H94-032.	2-95-02300/2H94-0 32	N/A	VT, Volumetric, Surface, Operating Pressure Test, N-416-1

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MPL Number	ASME/ Section XI Class	Description	UNIT 2 FALL 1995 Evaluation	MWO/DCR	DC Number	Pressure Test
2E11-C001B	2/2	Replace pump assembly with rebuilt assembly from warehouse. This assembly includes bolting.	Work is being performed under a 60 month PM program. No inservice failure involved. R/R evaluation written to document new pump assembly installation.	2-93-03154	2-94-01605	System Operability Surveillance
2E11-F005B	3/3	Replace valve disc and hanger	The disc is being replaced in order to install an anti-rotation spud on the disc. The hanger is being replaced as a preventative measure. To be replaced with like kind components. No failures have occurred.	2-95-00312	N/A	Operating Pressure Test
2E11-F007B	2/2	Replace valve per DCR 2H95-056	Due to excessive wear on the 2E11-F007B valve internals, the valve will be replaced with a new anchor darling valve. This is a 4 inch valve.	2-95-03163/2H95-0 56	N/A	Volumetric, VT, Operating Pressure (N-416-1)
2E11-F015B	1/1	Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to eliminate potential valve pressure locking.	NRC Generic Letter 89-10 recommends that safety related motor operated valves be able to function either opening or closing when subjected to the maximum differential pressure across the valve during normal operation and abnormal events within the design basis of the plant. The modification to the valve will meet the requirements of Generic Letter 89-10 and will provide increased margin to operate during a design basis event. This modification will increase MOV capability and improve system reliability by precluding the possibility of valve pressure locking. A failure has not occurred, thus a failure evaluation is not required.	2-95-02849/2H94-0 34	N/A	VT-3, VT-1, Operating Pressure Test, N-416-1

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MPL	ASME/ Section XI		UNIT 2 FALL 1995		DC	
Number	Class	Description	Evaluation	MWO/DCR	Number	Pressure Test
2E11-F018C	2/2	Replace and relocate valve	Replace valve with a like kind. Failure cause is attributed to service in a harsh environment. Valve was originally installed close to an elbow which resulted in turbulent flow. This MDC relocates the valve to a more favorable location.	2-95-01390 R3/95-5040	95-4091	Volumetric, Surface, Operating Pressure Test (N-416-1)
2E11-F028A	2/2	Replace stem, disc, disc guides, and yoke on valve 2E11-F028A.	NRC Generic Letter 89-10 recommends that safety related motor operated valves (MOV) be able to function either opening or closing when subjected to the maximum differential pressure across the valve during normal operation and abnormal events within the design basis of the plant. The modification to the valve will meet the requirements of Generic Letter 89-10 and will provide increased margin to operate during a design basis event. The new components will meet the design, material, and construction standards applicable to the RHR system. This modification will increase MOV capability and improve system reliability.	2-95-01972/2H94-0 34	N/A	Operating Pressure Test
2E11-F028B	2/2	Replace stem, disc, disc guides, and yoke on valve 2E11-F028B.	NRC Generic Letter 89-10 recommends that safety related motor operated valves (MOV) be able to function either opening or closing when subjected to the maximum differential pressure across the valve during normal operation and abnornevents within the design basis of the plant. The modification to the valve	2-95-01971/2H94-0 34	N/A	Operating Pressure Test

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	ASME/		UNIT 2 PALL 1993			
MPL	Section XI				DC	
Number	Class	Description	Evaluation	MWO/DCR	Number	Pressure Test
			will meet the requirements of Generic Letter 89-10 and will provide increased margin to operate during a design basis event. The new components will meet the design, material, and construction standards applicable to the RHR system. This modification will increase MOV capability and improve system reliability.			
2E11-F029	2/?	Nozzle of relief valve is being machine cut. Replace disc.	Machine cut is estimated to require removal of no more that 0.005", therefore no weld buildup is required. Machine cut of this nozzle is to remove a small defect and is an acceptable method of repair. Due to defect in the disc it will be replaced with an exact replacement. The seat leakage was found during seat leakage test, this test was repeated after repair of valve to verify defect removal.	2-94-03597	95-04101	Operating Pressure Test, Seat Leakage Test
2E11-F030D	2/2	Machine nozzle of valve and replace disc	Repair is to remove scratches on disc which were a result of leakage. Found during 42SV-SUV-004-0S. During repair welding of a new nipple for valve reiestallation was necessary.	2-94-03601 RI	9504187	Surface, VT, Operating Pressure Test (N-416-1)
2E11-F045	2/2	Grind out flaw and blend in with valve body. Do not exceed 1/4" depth without further evaluation.	Per evaluation by Structural Integrity Associates Inc. this is not a serviced induced flaw. Flaw evaluation determined this to be orginal construction flaw. This evaluation is to document that metal removal at these locations, no greater than 1/4" depth is acceptable. MT must be performed	2-95-02863	9503563	MT

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MPL Number	ASME/ Section XI Class	Description	Evaluation	MWO/DCR	DC Number	Pressure Test
			after metal removal to document flaw removal. No pressure testing required.			
2E11-F046B	2/2	Replace hinge pin, Bonnet studs, pipe caps, and polish seat.	The above replacement parts are exact equivalent components. The valve was operational and functional, however replacements parts will enhance continued performance of the valve.	2-92-03614 R1	2-92-03505	Operating Pressure Test
2E11-F111	2/2	Valve disc is worn, needs to be replaced.	This valve disc is attributed to normal system wear. The disc is to be replaced using like kind disc. This will not be a life of plant repair. This is a LLRT valve this it is evaluated for wear on an established cycle designed to predict unacceptable results prior to valve failure. Therefore this is a suitable repair. This is a 2 inch valve.	2-95-02990 R2	9503938	Operating pressure test
2E11-F123A	2/2	Replace valve per weld iso. Valve bonnet was replaced with Class 3 Bonnet and error was caught prior to returning valve to service. Craft damaged valve while correcting this error, thus the need for the replacing of entire valve.	As stated in description failure is due to craft. Valve is to be replaced with a Class 2 valve per approved welding procedure.	2-95-01427 R3	9503963	Surface, Operating Pressure (Code Case N-416-1)
2E11-F3078A	2/2	Remove blank flange and replace with blank flange specified in MDC 94-5035. Note: relief valve removed last outage under a TMM and a blank flange installed.	This blank flange is being installed under MDC 94-5035 and it's suitability was evaluated by that MDC.	2-94-02569/94-5035	N/A	Visual, Operating Pressure Test.
2E11-F3078B	2/2	Remove blank flange and replace with specified blank flange in MDC 94-5035. Note: Relief valve was removed last outage under a TMM.	This blank flange is being installed under MDC 94-5035 and it's suitability was evaluated by that MDC.	2-94-2570/94-5035	N/A	Visual, Operating Pressure Test.
2E11-RHR-224	2/2	Replace snubber with Lisega strut.	This replacement replaces a snubber 93	2-95-2212/94-5071	N/A	VT-3

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MPL Number	ASME/ Section XI Class	Description	Evaluation	MWO/DCR	DC Number	Pressure Test
		No inservice failure has occurred.	with a Lisega strut. It is part of the snubber reduction program. Reference MDC 94-5071 for evaluation.			
2E11-RHR-H700	2/2	Modify existing snubber	In order to meet the requirements of Generic Letter 89-10, DCR 94-034 replaces the yoke with one with shorter dimensions. Since the new yoke has shorter dimensions, the snubber will be lowered according to the new height. The new components and modification to the snubber will meet the design, material, and construction standards applicable to the RHR system. No failure has occured at this time.	2-95-1962/2H94-03 4	N/A	VT-3
2E11-RHR-H707	2/2	Modify existing snubber.	In order to meet the requirements of Generic Letter 89-10, DCR 94-034 replaces the yoke with one with shorter dimensions. Since the new yoke has shorter dimensions, snubber 1E11-RHR-H707 will be lowered according to the new height. The new components and modification to the snubber will meet the design, material and construction standards applicable to the RHR system.	2-95-01963/2H94-0 34	N/A	VT-3
2E11-RHR-H714A/	2/2	Remove existing PSA-100 snubber 2E11-RHR-H714 and replace with rigid strut.	This work is being performed under an approved MDC. Suitability is addressed via the MDC 94-5071. No service induced failure is involved. This is a snubber reduction program activity.	2-95-00515/94-5071	N/A	VT-3
2E11-RHR-H715A/	2/2	Remove existing PSA-100 snubber and replace with rigid strut.	This work is being performed under an approved MDC. Suitability is addressed via the MDC 94-5071. No	2-95-00513/94-5071	N/A	VT-3

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	ASMF/		UNIT 2 FALL 1995			
MPL Number	Section XI Class	Description	Evaluation	MWO/DCR	DC Number	Pressure Test
2E11-RHR-R228	2/2	Replace snubber with Lisega strut. This is part of the snubber replacement program. No actual failure has occurred at this time on the snubber.	This replacement is being done as part of the snubber reduction program. No failure has occurred at this time. Reference MDC 94-5071 for evaluation of replacement.	2-95-02211/94-5071	N/A	VT-3
2E11-RHR-R82	2/2	Torque wedge anchor bolts an base plate of support.	Torque of anchors is normal maintenance activity. Note: This work was in conjunction with hanger replacement work previously documented)	2-94-01261 R2	2-94-01163	VT-3
2E11-RHR-R82	B31.1/ 2	Replace strut hanger assembly due to spherical bearing failure.	Replacement of hangers are normal maintenance activities. Cause of failure is not known.	2-94-01261 R2	2-94-01148	VT-3
2E21-CS-R82B	2/2	Snubber had high drag during testing due to normal wear. This snubber will be replaced.	The snubber will be replaced with an identical unit, replacement is per approved plant procedures.	2-95-2570	N/A	VT-3
2E21-F005A	1/1	Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to eliminate potential valve pressure locking.	NRC Generic Letter 89-10 recommends that safety related motor operated valves be able to function either opening or closing when subjected to the maximum differential pressure across the valve during normal operation and abnormal events within the design basis of the plant. The modification to the valve will meet the requirements of Generic Letter 89-10 and will provide increased margin to operate during a design basis event. This modification will increase MOV capability and improve system reliability by precluding the possibility of valve	2-95-02845/2H94-0 34	N/A	VT-3, VT-1, Operating Pressure Test
	Number 2E11-RHR-R228 2E11-RHR-R82 2E11-RHR-R82	Number Class 2E11-RHR-R228 2/2 2E11-RHR-R82 2/2 2E11-RHR-R82 B31.1/ 2 2E21-CS-R82B 2/2	Number Section XI Class Description 2E11-RHR-R228 2/2 Replace snubber with Lisega strut. This is part of the snubber replacement program. No actual failure has occurred at this time on the snubber. 2E11-RHR-R82 2/2 Torque wedge anchor bolts an base plate of support. 2E11-RHR-R82 B31.1/ Replace strut hanger assembly due to spherical bearing failure. 2E21-CS-R82B 2/2 Snubber had high drag during testing due to normal wear. This snubber will be replaced. 2E21-F005A 1/1 Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to eliminate potential valve pressure	NIPL Section XI Number Class Description Evaluation service induced failure involved. This replacement is being done as part of the snubber replacement program. No actual failure has occurred at this time on the snubber. ZE11-RHR-R82 Z/2 Torque wedge anchor bolts an base plate of support. Torque of anchors is normal maintenance activity. Note: This work was in conjunction with hanger replacement work previously documented) ZE21-RHR-R82 B31.1/ ZE21-CS-R82B Z/2 Snubber had high drag during testing due to normal wear. This snubber will be replaced. ZE21-F005A JDiil hole in the upstream side of the flex wedge disc of ZE11-F005A to eliminate potential valve pressure locking. NRC Generic Letter 89-10 recommends that safety related motor operated valves be able to function either opening or closing when subjected to the maximum differential pressure across the valve during normal operation and abnormal events within the design basis of the plant. The modification to the valve will meet the requirements of Generic Letter 89-10 and will provide increased margin to operated during a design basis revent. This modification will increase MOV capability and improve system reliability by	MPL Section XI Number Class Description Evaluation service induced failure involved 2E11-RHR-R228 2/2 Replace snubber with Lisega strut. This is part of the snubber replacement program. No actual failure has occurred at this time on the snubber. 2E11-RHR-R82 2/2 Torque wedge anchor bolts an base plate of support. 2E11-RHR-R82 B31.1/ Replace strut hanger assembly due to spherical bearing failure. 2E11-RHR-R82 B31.1/ Particular bearing failure. 2E21-CS-R82B 2/2 Snubber had high drag during testing due to normal wear. This snubber will be replaced. 2E21-F005A 1/1 Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to climinate potential valve pressure locking. AMWO/DCR Evaluation service induced failure involved. This replacement is being done as part of the snubber reduction program. No failure has occurred at this time. Reference MDC 94-5071 for evaluation of replacement. Torque of anchors is normal maintenance activity. Note: This work was in conjunction with hanger replacement work previously documented) 2E21-CS-R82B 2/2 Snubber had high drag during testing due to normal wear. This snubber will be replaced. 2E21-F005A 1/1 Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to climinate potential valve pressure locking. ANG Generic Letter 89-10 2-95-02845/2H94-0 recommends that safety related motor operation and abnormal events within the design basis of the plant. The modification to the valve will meet the requirements of Generic Letter 89-10 and will provide increase daragin to operate during a design basis event. This modification will increase formarin to operate during a design basis event. This modification will increase formarin to operate during a design basis event. This modification will increase formarin to operate during a design basis event. This modification will increase formarin to operate during a design basis event. This modification will increase formarin to operate during a design basis event. This modification will increase formarine to the su	MPL Number Section XI Class Description Evaluation service induced failure involved. 2E11-RHR-R228 2/2 Replace snubber with Lisega strut. This is part of the snubber replacement program. No actual failure has occurred at this time on the snubber. 2E11-RHR-R82 2/2 Torque wedge anchor bolts an base plate of support. 2E11-RHR-R82 2/2 Torque wedge anchor bolts an base plate of support. 2E11-RHR-R82 2/2 Torque wedge anchor bolts an base plate of support. 2E11-RHR-R82 2/2 Torque wedge anchor bolts an base plate of support. 2E11-RHR-R82 3/2 Snubber had high drag during testing due to normal wear. This snubber will be replaced. 2E21-CS-R82B 2/2 Snubber had high drag during testing due to normal wear. This snubber will be replaced. 2E21-F005A 1/1 Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to climinate potential valve pressure locking. 2E21-F005A 1/1 Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to climinate potential valve pressure locking. 2E21-F005A 1/1 Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to climinate potential valve pressure locking. 2E21-F005A 1/1 Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to climinate potential valve pressure locking. 2E21-F005A 1/1 Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to climinate potential valve pressure within the design basis of the plant. The modification to the valve will mere the requirements of Generic Letter 89-10 and will provide increased margin to operate during a design basis event. This modification will increase MOV capability and improve system reliability by

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	ASN	ME/		UNIT 2 FALL 1995			
MPI Numb	L Section	on XI	Description	Evaluation	MWO/DCR	DC Number	Pressure Test
				pressure locking. A failure has not occurred, thus a failure evaluation is not required.			
2E21 F0	05B 1/1		Drill hole in the upstream side of the flex wedge disc of 2E21-F005B to eliminate valve pressure locking.	NRC Generic Letter 89-10 recommends that safety related motor operated valves (MOV) be able to function either opening or closing when subjected to the maximum differential pressure across the valve during normal operation and abnormal events within the design basis of the plant. The modification to the valve will meet the requirements of Generic Letter 89-10 and will provide increased margin to operate during a design basis event. The new components will meet the design, material, and construction standards applicable to the RHR system. This modification will increase MOV capability and improve system reliability.	2-95-02846/2H94-0 34	N/A	VT-3, VT-1, Operating Pressure Test
2E21-F0	112A 2/2		Valve to be replaced. Valve is bolted in.	This valve is an identical replacement. The original valve could not have its set pressure adjusted.	2-94-3611	2-95-2491	Operating Pressure Test
2E21-F0	12B 2/2		Replace Valve	Existing valve failed leakage test. Replacement valve is identical to original valve. Cause of failure does not appear to be service induced, but rather valve being degraded over time due to maintenance activities. Specifically in the seat area. Valve replacement will be in accordance with applicable code and plant approved	2-93-02393 R1	2-94-00825	Surface, Hydrostatic Pressure Test

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MPL	ASME/ Section XI		UNIT 2 FALL 1995		DC	
Number	Class	Description	Evaluation	MWO/DCR	Number	Pressure Test
			procedures.			
2E21-F015A	1/1	Drill hole in the upstream side of the flex wedge disc of 2E21-F015A to eliminate valve pressure locking. Also replace bolts for bonnet.	NRC Generic Letter 89-10 recommends that safety related motor operated valves (MOV) be able to function either opening or closing when subjected to the maximum differential pressure across the valve during normal operation and abnormal events within the design basis of the plant. The modification to the valve will meet the requirements of Generic Letter 89-10 and will provide increased margin to operate during a design basis event. The new components will meet the design, material, and construction standards applicable to the RHR system. This modification will increase MOV capability and improve system reliability. In addition bonnet bolts were found to be galled and corroded. Reason for this could not be determined.	2-95-02847/2H94-0 34	N/A	VT-1, VT-3, Operating Pressure Test
2E21-F019A	2/2	Replace disc and o-ring gland.	Disc and o-ring gland will be replaced with exact replacements.	2-94-01108	2-94-00772	Operating Pressure Test
2E21-F036A	2/2	Replace and Relocate check valve.	Check valve was originally installed with the discharge of the valve butted up to an elbow which causes the valve to wear due to turbulent flow conditions.	2-94-02522 R1/95-5038	2-94-00797	Surface, Volumetric, Operating Pressure Test (N-416-1)
2E21-F036A	2/2	Replace all valve internals including the disc.	Internals are being replaced due to wear. Wear is of normal operating nature.	2-93-4176 RI	2-94-00797	Operating Pressure Test

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	ASME/		UNIT 2 FALL 1995			
MPL Number	Section XI Class	Description	Evaluation	MWO/DCR	DC Number	Pressure Test
2E32	1&2/1 &2	Remove piping and supports associated with MSIV LCS in the Steam Chase as specified by DCR 88-106.	Piping is being removed as specified by DCR. Suitability evaluation will be performed during approval of DCR 88-106. No service induced failure involved. MSIV leakage control system removal is to eliminate a design inconsistency.	2-94-00661/88-106	N/A	Suface, Hydrostatic Pressure Test
2E41	2/2	Replace restricting orifices D001 and D002 and associated piping.	Evaluation for suitability has been evaluated during the DCR process. No inservice failure involved. Replacement is for pressure reduction past isolation valve.	2-94-699/2H93-036	N/A	Surface, Operating Pressure Test.
2E41	2/2	Replace and fabricate a new HPC1 turbine exhaust drain pot stilling well, as specified by DCR 93-036.	This replacement is being performed under an approved DCR for improvement of system operation. Suitability evaluation is covered by DCR 93-036. No service induced failure involved.	2-94-700/2H93-036	N/A	Surface, Visual, Hydrostatic Pressure Test.
2E41-F001	2/2	Replace valve disc due to normal wear. Also replace stem, studs/nuts, inner bonnet stem outer bonnet pressure seal.	The replacement disc is being supplied by the manufacturer with a stellite 6 facing. This is expected to improve the reliability of the valve. The studs and nuts are being replaced as a good practice, they have not failed.	2-95-00130	2-94-02299	Operating Pressure Test
2E41-F006	2/2	Relocate 2E41-F006 valve upstream of existing location. Replace old valve location with pipe spool.	This work is being performed by DCR 92-164. Material suitability has been evaluated by the DCR.	2-93-04997/2H92-1 64	N/A	VT, RT, UT, Surface, and Hydrostatic Pressure Test.
2E41-r006	2/2	Drill hole in the upstream side of the flex wedge disc of 2E41-F006 to eliminate potential valve pressure locking.	NRC Generic Letter 89-10 recommends that safety related motor operated valves (MOV) be able to function either opening or closing when subjected to the maximum	2-95-02844/2H94-0 34	N/A	Operating Pressure Test

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	ASME/		UNIT 2 FALL 1995			
MPL	Section XI			MINORCE	DC	P Tout
Number	Class	Description	Evaluation	MWO/DCR	Number	Pressure Test
			differential pressure across the valve during normal operation and abnormal events within the design basis of the plant. The modification to the valve will meet the requirements of Generic Letter 89-10 and will provide increased margin to operate during a design basis event. This modification will increase MOV capability and improve system reliability by precluding the possibility of valve pressure locking. A failure has not occured.			
2E41-HPCI-H99	2/2	Relocate strut support and replace strut per DCR 2H92-194	This work is being performed by DCR 92-164. Material suitability has been evaluated by the DCR.	2-93-04997/9H92-1 64	N/A	VT-3
2E51-F013	2/2	Drill hole in the upstream side of the flex wedge disc of 2E11-F005A to eliminate potential valve pressure acking.	NRC Generic Letter 89-10 recommends that safety related motor operated valves be able to function either opening or closing when subjected to the maximum differential pressure across the valve during normal operation and abnormal events within the design basis of the plant. The modification to the valve will meet the requirements of Generic Letter 89-10 and will provide increased margin to operate during a design basis event. This modification will increase MOV capability and improve system reliability by precluding the possibility of valve pressure locking. A failure has not occurred, thus a failure evaluation is not required.	2-95-02843/2H94-0 34	N/A	VT-3, VT-1, Operating Pressure Test, N-416-1

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MPL Number	ASME/ Section XI Class	Description	Evaluation	MWO/DCR	DC Number	Pressure Test
2E51-F021	2/2	valve plug is worn and flats rounded	This failure is attributed to system wear. The plug will be replaced with like kind parts.	2-95-00370	N/A	Surface, VT, Operating Pressure Test
2E51-F045	2/2	Replace the existing Rockwell valve with a Anchor/Darling special contour plug valve per DCR 94-034.	NRC Generic Letter 89-10 recommends that safety related MOV's be able to function either opening or closing when subjected to the maximum differential pressure across the valve during normal operation and abnormal events within the design basis of the plant. The modification to the valve will meet the requirements of GL 89-10 and will provide increased margin to operate during a design basis event. The new components will meet the design, material and construction standards applicable to the RCIC system. This modification will increase MOV capability and improve system reliability.	2-95-2067/2H94-03 4	N/A	RT, VT-3, PT, Operating Pressure Test (N-416-1)
2G11-F004	2/2	Replace valve. This valve wear is attributed to normal system wear.	The replacement of this valve will be with a like kind valve.	2-95-03085 R1	95-04130	Volumetric, Surface, VT, Operating Pressure Test, N-416-1
2G31	1/1	Prefab bottom head drain assembly for incore monitoring	No failure has occurred. Piping material being installed is per pipe class DCA and will be welded and inspected per 42EN-EME-012-0S.	2-95-02595	N/A	Surface, Volumtric, Operating Pressure (Code Case N-416-1), VT
2G31	1/1	Install new bottom head drain line assembly line assembly to monitor the effectiveness of the Hydrogen Injection System for protecting the bottom region of the reactor vessel. This assembly is 2 inch and under.	No failure occurred. The piping material being installed is per pipe class DCA and will be welded and inspected per 42EN-EME-012-0S.	2-95-02122/93-059	N/A	VT, Surface, Operating Pressure Test (N-416-1)

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MPL Number	ASME/ Section XI Class	Description	UNIT 2 FALL 1995 Evaluation	MWO/DCR	DC Number	Pressure Test
2N11-F003	2/2	Per MWO valve had a bad packing leak, upon investigation bonnet stuffing box was severely pitted due to normal inservice wear.	Replacement parts are identical parts obtained from the warehouse. All work is done per approved plant procedures.	2-93-02169 R1	2-93-01753	Operating Pressure Test
2P11-A001	2/2	Replace bolting in flanges on piping specified on MWO.	Bolting being used is an approved replacement for bolts being removed. Cause of failure appears to be corrosion and possible stress.	2-93-4766	N/A	Visual leakage check
2P42-F083	B31.1/ 2	Replace relief valve 2P42-F083. Valve failed leakage and set pressure test. Valve is bolted in.	It was determined by procurement review services (PRS) that existing relief valve 2P42-F083 was a Non-ASME code valve. IST inspection plan boundary diagram drawings identify the valve as being ASME section XI class 2. Per PRS existing valve is being replaced with an identical valve. The new valve was determined to be an acceptable replacement and will be upgraded by commercial grade dedication plan number CGDP No. 94-0621. Existing valve was degraded due to maintenance attempting to repair so as to past leakage test. Replacement will be performed in accordance with applicable approved plant procedures.	2-93-03018	N/A	Operating Pressure Test.
2T52	MC/2	Install new penetration per DCR 94-013	No failure has occurred. New cables need to be pulled and there was no more room in existing penetrations.	2-95-01348/2H94-0 13	N/A	Volumetric, Surface, Pnematic pressure test.

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Code Case and Alternative Testing Summary for Hatch Unit 2, Fall 1995 Outage

Code Case No.	<u>Title</u>
Case N-98	Ultrasonic Examination-Calibration Block Tolerances, Section XI.
Case N-416-1	Alternative Pressure Test Requirement for Welded Repairs or Installation of Replacement Items by Welding, Class 1, 2 and 3, Section XI, Division 1.
Case N-460	Alternative Examination Coverage for Class 1 and Class 2 Welds, Section XI, Division 1.
Case N-461	Alternative Rules for Piping Calibration Block Thickness, Section XI, Division 1.
Case N-491	Alternative Rules for Examination of Class 1, 2, 3, and MC Component Supports of Light-Water Cooled Power Plants, Section XI, Division 1.