NRC Form 366 (9-63)	LIC	ENSEE EVENT	REPORT	(LER)	A	CLEAR REGULATO PPROVED OMB NO XPIRES 8/31/85	
FACILITY NAME (1)					CKET NUMBER		PAGE (3)
Sequoyah, Unit	1			0	15 0 0	0 3 2 7	1 OF 012
Inoperability of	f Control Room Emer	gency Venti	lation Sy	stem			
EVENT DATE (6)	LER NUMBER (6)	REPORT DATE (7)		OTHER F	ACILITIES INVO		
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OPERATING THIS	REPORT IS SUBMITTED PURSUANT		OF 10 CFR 8: /C		the following! (1		
POWER LEVEL (19) 1.1.0.10	20.402(b) 20.405(a)(1)(i) 20.405(a)(1)(ii) 20.405(a)(1)(iii) 20.406(a)(1)(iv) 20.405(a)(1)(iv) 20.405(a)(1)(v)	20.405(c) 50.36(c)(1) 50.36(c)(2) 50.73(a)(2)(i) 50.73(a)(2)(ii) 50.73(a)(2)(iii)		50,73(a)(2)(iv) 50,73(a)(2)(v) 50,73(a)(2)(vii) 50,73(a)(2)(viii)(A) 50,73(a)(2)(viii)(B) 50,73(a)(2)(x)			kcify in Abstract Text, NRC Form
NAME	L	ICENSEE CONTACT FOR	THIS LER (12)			TELEPHONE NUME	ED
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Glenn B. Kirk, (Compliance Engineer		1993		61115	8 7 0 1 -	1611416
	COMPLETE ONE LINE FOR	EACH COMPONENT FA	LURE DESCRIBE	D IN THIS REPORT	(13)	T P	
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During a review Test", it was di trains of the co	e, epproximately fifteen single space type of test data for S iscovered that an i ontrol room emergen rates for approxim	I-144.2, "Co ncorrect cal cy ventilati	lculation ion syste	m being 1	area res eft with	ulted in	both
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NRC Form 366 (9-83)

		LICENSEE	EVENT	REPORT	(LER)	TEXT	CONTINUATION	
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U.S. NUCLEAR REGULATORY COMMISSION

APPEOVED OMB NO. 3150-0104 EXPIRES 8/31/85

	LER NUMBER (6)					PAGE (3)		
Converse light 1	R 51	NUMBER	L	REVISION	1	T	T	
Sequoyah, Unit 1 0 5 0 0 0 3 2 7 8 4	4 - 1	01512	-	0 10	012	or	0	12

SI-144.2, "Control Room Emergency Ventilation System Test", was completed on both trains of the control room emergency ventilation system at 0100C on 08/17/84. Field calculations performed during the testing indicated both trains had passed the flow rate acceptance criteria of 4000 CFM \pm 10%. Subsequent engineering review of the test data discovered that the field calculation of the duct area was in error. The duct area was incorrectly calculated in the field to be 3.67 ft.² when the actual area was 3.33 ft.². Recalculation of the flow rates using the correct duct area determined that the flow rates had actually been lower than acceptable. The recalculated flow rates were 3580 CFM for train 'A' and 3290 CFM for train 'B'. Both trains of the control room emergency ventilation system were declared inoperable at 03000 on 08/17/84 requiring entry into LCO 3.0.3.

The control damper was adjusted to obtain an acceptable flow rate for train 'A'. Then train 'B' was checked and the flow rate found acceptable without further adjustment of the common damper. Both trains were declared operable at 0430C on 08/17/84.

Investigation of this event determined that the proper duct size was used during the previous performance of the 18-month surveillance requirement, but it also was discovered that the Technical Instruction TI-50, "Air Flow Measurement Methods", used during the test contained an incorrect duct area. This incorrect duct size in TI-50 had not been used in either the present calculation or previous calculation of flow rates.

The cause of the event is attributed to a personnel error by engineering test personnel incorrectly calculating the duct area.

The following long term corrective actions have been taken:

- a. All computations for SI-144.2 will be performed by two independent persons.
- b. The control room emergency ventilation system common control damper will be marked to ensure that it is not moved between surveillance tests.

c. TI-50 has been revised to indicate the proper duct area.

Both unit 1 and unit 2 were in mode 1 (2235 psig, 578° F) at 100% reactor power at the time of the event. There was no effect on public health or safety. For 1984, there have been no previous occurrences.

NRC Form 366A

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant Post Office Box 2000 Soddy Daisy, Tennessee 37379

September 14, 1984

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO. 50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT SQR0-50-327/84052

The enclosed licensee event report provides details concerning both trains of the control room emergency ventilation system being inoperable. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.i, a.2.v, and a.2.vii.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

P.R. Walla

P. R. Wallace Plant Manager

Enclosure cc (Enclosure):

> James P. O'Reilly, Director U.S. Nuclear Regulatory Commission Suite 2900 101 Marietta Street, NW Atlanta, Georgia 30323

Records Center Institute of Nuclear Power Operations Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339

NRC Inspector, NUC PR, Sequoyah

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