AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272
Unit Name Salem # 1
Date Sept. 10,1984
Telephone 609-935-6000
Extension 4455

Completed by J. P. Ronafalvy

Month August 1984

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

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1/1

OPERATING DATA REPORT

Docket No. 50-272
Date Sept. 10, 1984
Telephone 935-6000
Extension 4455

Completed by J. P. Ronafalvy

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	August 1984	
3.	Licensed Thermal Power (MWt)	3338	
4.	Nameplate Rating (Gross MWe)	1135	
5.	Design Electrical Rating (Net MWe)	1090	
6.	Maximum Dependable Capacity (Gross MWe)	1124	
7.	Maximum Dependable Capacity (Net MWe)	1079	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	5855	62880
12. No. of Hrs. Reactor was Critical	0	1237.6	34388.8
13. Reactor Reserve Shutdown Hrs.	0	54.5	3088.4
14. Hours Generator On-Line	0	1197.8	32975.7
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	3800023	99619394
17. Gross Elec. Energy Generated (MWH)	0	1281380	32896480
18. Net Elec. Energy Generated (MWH)	(5029)	1201036	31172348
19. Unit Service Factor	0	20.5	52.4
20. Unit Availability Factor	0	20.5	52.4
21. Unit Capacity Factor (using MDC Net)	0	19.0	45.9
22. Unit Capacity Factor (using DER Net)	0	18.8	45.4
23. Unit Forced Outage Rate	100	65.8	31.8

24. Shutdowns scheduled over next 6 months (type, date and duration of each)
N/A

25. If shutdown at end of Report Period, Estimated Date of Startup:
9-15-84

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	9/30/76	12/11/76
Initial Electricity	11/1/76	12/25/76
Commercial Operation	12/20/76	6/30/77

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UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH August 1984

Completed by J.P. Ronafalvy

Docket No. 50-272
Unit Name Salem No.1
Date Sept. 10,1984
Telephone 609-935-6000
Extension 4455

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
84-176	7-9	F	744	A	4	-	HA	GENERA	Generator Liquid Cooling System

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4 Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Salem as Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH August 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: September 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-0884	Cable Trays	Reroute various cables from cable tray 1C003.
1EC-1437	Reactor Vessel Internals Lifting Rig	Revise design to incorporate use of an integral work/access platform and integral tools for engaging/disengaging the lift rig to reactor vessel internals.
1EC-1613	Reactor Vessel Level Instrumentation	Add capacitors to DT1842-4 board on the reactor vessel level microprocessor.
1EC-1649	Safety Injection	Modify the existing 1SJ4 and 1SJ5 valve leak-off connections to facilitate visual inspection of valve stem leakage.
1EC-1672	Steam Generator Drains and Blowdown	Replace valve Nos. 11/12/13/14 GB918 (D3) with new Mark F53 valves.
1EC-1686	Service Water	Remove caviatation control tube bundles from 11, 12, 13, 14, 15 SW57 and SW223 valves.
1EC-1791	Fire Protection	Install smoke detectors in areas designed PIC-1, PIC-3, PIC-4, PIC-5, P1F1, PIG-1.
1EC-1813	Condensate Polishing	Reline embedded floor drain piping using corrosion/erosion resistant material - provide necessary core bores in floor for access.
1EC-1822	Safety Injection	Provide an encapsulation around Weld No. RH-2246-148 for housing Furmanite.

MAJOR PLANT MODIFICATIONS
REPORT MONTH August 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: September 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1869	Steam Generator Feed Pump No. 11	Upgrade the material of the impeller to CA6NM utilizing the spare rotating assembly.
1EC-1882	Pressurizer Safety Valve Piping-RCS	Replace insulation on the 6" loop seal piping off pressurizer with cerablanket insulation.
1EC-1889	Steam Generator Hydraulic Snubbers	Replace existing four "Rexnord" hydraulic snubbers attached to steam generator with four new "Paul Monroe" hydraulic snubbers.
1SC-0313	Service Water	Modify service water pump motors to allow filling of oil reservoirs.
1SC-0363	Condensate	Modify #12 Heater Drain Pump motor to allow filling of oil reservoirs.
1SC-0650	Steam Generator Feed Water Pumps #11 & 12-Pump Journal Bearings	Modify new bearing retainers per Franklin Institute Research Lab Report F-A5477.
1SC-0896	Turbine Aux Cooling	Change material of tubing in Unit 1 Turbine Aux Heat Exchangers from type 90-10 copper nickel to a material more suited to our system.
1SC-0965A	Reactor Head	Design temporary shielding in accordance with Proto-Power Management Corp. proposal #7612000-A009. Shielding would be installed prior to stud detensioning and removed after tensioning completed.

MAJOR PLANT MODIFICATIONS
REPORT MONTH August 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: September 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1SC-1124	Containment Spray/ Refueling Cavity	Provide a hard pipe for the containment spray cavity fill line. Also designate a storage area for the pipe when not in use.
1SC-1231	Control Air	Install in the control air system two blocking valves and two vent valves.
1SC-1239	Stator Water Cooling	Install discharge isolation valves on 11 and 12 Stator Water Cooling Pumps.
1SC-1268	Hydrogen Monitoring	Install ladder and platform in Unit 1 Containment to gain access to 11 and 12 Hydrogen Monitor System sensor assembly to perform maintenance.
1SC-1285	Fresh Water	Replace temporary water supply line to QA toilet facilities.
1SC-1330	Structural-"B" Building	Install a wall and doorway in the old TSC to provide conference space and office space.

MAJOR PLANT MODIFICATIONS
REPORT MONTH AUGUST 1984

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: September 10, 1984
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-0884	This cable reroute does not affect the operation of any safety related equipment. No unreviewed safety or environmental questions are involved.
1EC-1437	This change affects only the Internals Lifting Rig which is not a safety related component. This change is to Westinghouse specifications. The modifications do not modify or alter any load carrying member of the lift rig nor do they alter in any way the function of the lift rig. No unreviewed safety or environmental questions are involved.
1EC-1613	This change modifies circuitry installed under the RVLIS package. No unreviewed safety or environmental questions are involved.
1EC-1649	This change does not alter any of the Safety Injection System original design functions nor of any related components or systems. This change does not increase the consequences of an event nor the likelihood of an occurrence. Therefore, no unreviewed safety or environmental questions are involved.
1EC-1672	This change upgrades the valves used. No unreviewed safety or environmental questions are involved.
1EC-1686	This change removes bundles allowing the valve to operate without interference. No unreviewed safety or environmental questions are involved.
1EC-1791	This change expands the fire protection capabilities of the plant. No unreviewed safety or environmental questions are involved.
1EC-1813	This change involves piping repair in a non-safety related building. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1822	This change does not alter the original design, function, or intent of the Safety Injection System. This change does not increase the likelihood of an occurrence, nor the consequences of an event. Therefore, no unreviewed safety or environmental questions are involved.
1EC-1869	This change in the material of the impeller of No. 11 Steam Generator Feed Pump does not affect any presently performed safety analysis nor does it create any new hazards. No unreviewed safety or environmental questions are invovled.
1EC-1882	The insulation being installed restores the original insulation on the piping. The insulation meets the requirement of Nonmetallic Thermal Insulation for Austenitic Stainless Steel as outlined in Reg. Guide 1.36. No unreviewed safety or environmental questions are involved.
1EC-1889	The new snubbers have the same design load capacity and perform the same function. However, the new snubbers are fitted with "tefzel" seals which has a life expectancy of forty years. No unreviewed safety or environmental questions are involved.
1SC-0313	The change in the upper bearing oil fill piping will not affect the operation of the motors. No unreviewed safety or environmental questions are involved.
1SC-0363	The change in the lower bearing oil fill piping will not affect the operation of the motors. No unreviewed safety or environmental questions are involved.
1SC-0650	The change is in accordance with the Franklin Institute Research Lab. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1SC-0896	This change does not alter the design, performance, or operation of the Turbine Aux. Control System. No unreviewed safety or environmental questions are involved.
1SC-0965A	The lead blankets are used only in Mode 5. The RV Head Lift rig design meets the guidelines of NUREG 0612, "Control of Heavy Loads at Nuclear Power Plants." The head lift rig is not safety related. This DCR introduces plastic material into the containment. Its use is acceptable because it is used with the plant in cold shutdown only, the material is not permanently installed and will be removed prior to startup, fire extinguishing materials are available in the area, and the area is patrolled by a roving fire watch. No unreviewed safety or environmental questions are involved.
1SC-1124	The installation of the support structure, angles, plates, anchor bolts and washers does not affect any presently applied safety feature. It also does not create any new safety/fire hazards. No unreviewed safety or environmental questions are involved.
1SC-1231	The new valves make the Type "C" and Type "A" leak tests simpler to perform on CA 330 valves. No unreviewed safety or environmental questions are involved.
1SC-1239	This modification does not alter any plant process or discharge and will not affect the existing plant impact. No unreviewed safety or environmental questions are involved.
1SC-1268	This modification does not alter any plant process or discharge and will not affect the existing plant impact. No unreviewed safety or environmental questions are involved. The telescoping ladders and platforms conform to ANSI Standard A92.3 - 1980 and seismic restraints are used to negate seismic events effects.
1SC-1285	This change involves the Potable Water System which is outside the limits of the Safety Related Structures. No unreviewed safety or environmental questions are involved.
1SC-1330	This change involves electrical work from non-safety related buses. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

PS[illegible]ALEM GENERATING STATION
[illegible]ELATED WORK ORDER LOG

SALEM UNIT 1

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-07-30-033-1			
	SMD	1	13 CHILLER RELIEF VALVE
			FAILURE DESCRIPTION: VALVE LEAKS
			CORRECTIVE ACTION: REPLACED VALVE
943754	NCS	1	VALVE 1FP147
			FAILURE DESCRIPTION: VALVE FAILED
			CORRECTIVE ACTION: REPACKED AND REPLACED BONNET GASKET
943629	NCS	1	VALVE 13GB4
			FAILURE DESCRIPTION: VALVE FAILED LEAK RATE TEST
			CORRECTIVE ACTION: REPLACED PLUG, SEAT, PACKING, AND BONNET GASKET
943628	NCS	1	VALVE 12GB4
			FAILURE DESCRIPTION: VALVE FAILED LEAK RATE TEST
			CORRECTIVE ACTION: REPLACED PLUG, SEAT, PACKING, AND BONNET GASKET
84-07-29-992-8			
	SMD	1	100' CONTAINMENT AIRLOCK
			FAILURE DESCRIPTION: EXTERIOR DOOR BALL VALVE LEAKING
			CORRECTIVE ACTION: RENEWED ALL SEALS ON VALVE

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-06-22-306-5	SMD	1	BREAKER 326 IN PANEL 1CCDC
			FAILURE DESCRIPTION: GROUND INDICATED ON NOS. 11 - 14 MAIN STEAM ISOLATION VALVES
			CORRECTIVE ACTION: WIRE 3 OF 1MS208CT GROUNDED; IT WAS CUT AND REPULLED
84-07-18-518-3	SMD	1	12 RCDT PUMP
			FAILURE DESCRIPTION: PUMP DOES NOT RUN
			CORRECTIVE ACTION: FOUND 2 PHASES SHORTED TO GROUND; REPLACED DAMAGED LEAD OF CABLE
9900129-2	SMD	1	100' ELEVATION AIRLOCK DOOR
			FAILURE DESCRIPTION: WELD REPAIR REQUIRED AS PER DR #MD 84-3235
			CORRECTIVE ACTION: WELD REPAIRED THREE (3) AREAS
009910291	SMD	1	1C SEC
			FAILURE DESCRIPTION: TEST 18 WILL NOT RESET
			CORRECTIVE ACTION: REPLACED CARD ASSEMBLY WITH SPARE ASSEMBLY AND RELAY XK6 REPLACED
84-06-02-804-1	MT	1	VALVE 1WL-96
			FAILURE DESCRIPTION: VALVE FAILED RETEST
			CORRECTIVE ACTION: REPLACED STEM, O-RINGS, COMPRESSOR, AND D1APHRGAM

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
84-05-26-399-3	MT	1	VALVE 12CA330
			FAILURE DESCRIPTION: VALVE FAILED LRT
			CORRECTIVE ACTION: MACHINED SEAT AND PLUG; REPLACED BONNET GASKET AND PACKING
943761	MT	1	1SA264
			FAILURE DESCRIPTION: VALVE FAILED LRT
			CORRECTIVE ACTION: REPACKED AND REPLACED THE BONNET SEAT
924477-8	RE	1	INCORE FLUX MAPPING SYSTEM
			FAILURE DESCRIPTION: FAILED INCORE DETECTOR ON DRIVE "E" AND DETECTOR "F" IS INTERMITTENT
			CORRECTIVE ACTION: REPLACED "E" DRIVE CABLE AND TIGHTENED LOOSE CONNECTION ON "F" DRIVE GLEASON REEL
84-06-24-340-6	SMD	1	12 BORIC ACID TRANSFER PUMP
			FAILURE DESCRIPTION: INBOARD SEAL LEAKING
			CORRECTIVE ACTION: REPLACED MECHANICAL SEAL
0099101254	SMD	1	11 CV PUMP AUX. OIL PUMP CHARGING
			FAILURE DESCRIPTION: RUNS INTERMITTENTLY IN AUTO WHEN 11CV PUMP IS IN SERVICE
			CORRECTIVE ACTION: INSTALLED A NEW RECALIBRATED PRESSURE SWITCH

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
9900139-0	SMD	1	12 CHARGING PUMP
			FAILURE DESCRIPTION: SMOKE FROM MOTOR, PUMP WON'T TURN
			CORRECTIVE ACTION: CHANGED OUT ROTATING ELEMENTS
009910271	SMD	1	12 CHARGING PUMP
			FAILURE DESCRIPTION: OUTBOARD BEARING LEAKS
			CORRECTIVE ACTION: REPAIRED SEAL HOUSING
0099101076	SMD	1	VALVE 1CV259
			FAILURE DESCRIPTION: BAD DIAPHRAGM LEAK
			CORRECTIVE ACTION: REPLACE DIAPHRAGM
0099028514	SMD	1	VALVE 11CV150
			FAILURE DESCRIPTION: BONNET LEAK
			CORRECTIVE ACTION: REPLACED BONNET
0099028506	SMD	1	VALVE 11CV151
			FAILURE DESCRIPTION: BAD DIAPHRAGM LEAK
			CORRECTIVE ACTION: REPLACE DIAPHRAGM
0099101114	SMD	1	VALVE 11CV156
			FAILURE DESCRIPTION: BLOWN DIAPHRAGM
			CORRECTIVE ACTION: REPLACED BONNET GASKET

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
0099001594	SMD	1	VALVE 1CV44
			FAILURE DESCRIPTION: VALVE LEAKING
			CORRECTIVE ACTION: ROTATED DISKS
84-08-16-880-1	SMD	1	SW STRN. BACKWSH.
			FAILURE DESCRIPTION: VALVE LEAKING
			CORRECTIVE ACTION: REPLACED VALVE BODY AND DIAPHRAGM
0099027755	SMD	1	VALVE 11SW99
			FAILURE DESCRIPTION: VALVE DID NOT STOP FLOW OF WATER AS REQUIRED DURING USE OF PROCEDURE 4.0.5-V-SW2
			CORRECTIVE ACTION: ADJUSTED SPRING AND CLEANED VALVE
84-08-02-208-3	SMD	1	NO. 13 SERVICE WATER PUMP
			FAILURE DESCRIPTION: THE PUMP STRAINER PACKING APPEARS TO HAVE BLOWN
			CORRECTIVE ACTION: REPLACE SHAFT AND REPACKED/REPLACED OIL SEAL ON GEAR BOX
948901	OD	1	SERVICE WATER HEADER ISOLATION VALVE (12SW23)
			FAILURE DESCRIPTION: VALVE LEAKS
			CORRECTIVE ACTION: REPAIRED RUBBER LINING

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
922458	OD	1	VALVE 11SW17
			FAILURE DESCRIPTION: VALVE LEAKS
			CORRECTIVE ACTION: REPAIRED RUBBER LINING
940707-3	OD	1	VALVE 14SW13
			FAILURE DESCRIPTION: CHECK VALVE DOES NOT HOLD
			CORRECTIVE ACTION: NEW CHECK VALVE INSTALLED
922459	OD	1	VALVE 12SW17
			FAILURE DESCRIPTION: VALVE LEAKS
			CORRECTIVE ACTION: REPAIRED RUBBER LINING
922463	OD	1	VALVE 12SW20
			FAILURE DESCRIPTION: VALVE LEAKS
			CORRECTIVE ACTION: REPAIRED RUBBER LINING
009900189-6	SMD	1	NO. 12SW20 VALVE MOTOR
			FAILURE DESCRIPTION: THE MOTOR MEGGERS BAD AND HAS WATER IN ITS BAY
			CORRECTIVE ACTION: REPLACED MOTOR
947253	MD	1	FC-1-32 SW EXPANSION JOINT
			FAILURE DESCRIPTION: EXPANSION JOINT LEAKS
			CORRECTIVE ACTION: REPLACED 30" EXPANSION JOINT

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 1
AUGUST 1984

Unit No. 1 remained shutdown as the fifth refueling outage continues. The Unit began the month in Mode 5. No. 1 generator reassembly work was completed on August 15, 1984. Work that was done in support of the generator reassembly included turbine bearing inspection, exciter reinstallation and placing the Main Turbine on its turning gear. The Reactor head insulation has been replaced. No. 11 Charging Pump has been inspected, repaired and flushed. No. 14 CFCU has been installed. In-core Thermocouple cable upgrade was completed. Limitorque valve modifications are in progress. Reactor Coolant System fill and vent has been completed. Preparations were completed for the Type A Containment Integrated Leakage Rate Test. The test commenced on August 11, 1984 and was successfully completed on August 12, 1984. The recirculation line from the Boron Injection Tank to the Boric Acid Tank was found to be plugged. Efforts have been successfull in clearing the line. Preparations began for Unit Mode change from Mode 5 to Mode 4. Surveillance testing for the Mode change is in progress. On August 23, 1984, a failure of a 30" expansion joint in Service Water Bay 1 resulted in minor flooding of the Bay area and subsequent equipment damage. The expansion joint has been repaired. In attempting to isolate the expansion joint, valve 13SW20 leaked excessively. The valve has been removed and is in the process of being repaired. A spacer (blank) has been installed replacing 13SW20 in order to complete surveillance testing of the Service Water System in support of Unit Mode change.

REFUELING INFORMATION

COMPLETED BY: J. Ronafalvy

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: September 10, 1984
TELEPHONE: 609/935-6000
EXTENSION: 4455

Month August 1984

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: February 22, 1986

3. Scheduled date for restart following refueling: May 4,1986

4. A) Will Technical Specification changes or other license amendments be required?
   YES _____ NO _____
   NOT DETERMINED TO DATE _____ X

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
   YES _____ NO X
   If no, when is it scheduled? January 1986

5. Scheduled date(s) for submitting proposed licensing action:
   January 1986 if required

6. Important licensing considerations associated with refueling:
   NONE

7. Number of Fuel Assemblies:
   A) Incore 193
   B) In Spent Fuel Storage 296

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4

PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

September 10, 1984

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of August 1984 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

LKM:sbh
cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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