

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

June 8, 1992

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Serial No. 92-086
NL&P/JYR:jmj
Docket Nos. 50-338
50-339
License Nos. NPF-4
NPF-7

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION UNITS 1 AND 2
ADMINISTRATIVE CORRECTION TO PROPOSED
TECHNICAL SPECIFICATION CHANGE

Our letters dated November 9, 1987 and March 31, 1988 proposed changes to the Technical Specifications for North Anna Units 1 and 2 associated with the intermediate range neutron flux trip setpoints. Specifically, the changes were to revise the method by which the nominal and allowable intermediate range neutron flux trip setpoints were specified, and reformat the operability and surveillance requirements to be consistent with Standard Technical Specifications. However, due to an administrative oversight, the submittals were incomplete. The proposed increase from 25% to 35% of rated thermal power for the intermediate range flux trip setpoints was not addressed in two specifications.

Technical Specifications 3.10.3 (Physics Tests) and 3.10.4 (Reactor Coolant Loops) address special test exceptions. In the current wording, the reactor trip setpoints for both the intermediate and power range changes are the same. The wording should have been revised to differentiate between power levels with respect to the intermediate and power range channels. Proposed revisions to those specifications were not included in the submittals, but were addressed within the scope of the original safety evaluation.

Specifically, Technical Specification 3.10.3 and 3.10.4 should be revised to state that the reactor trip setpoints for the intermediate range channels are set at $\leq 35\%$ of rated thermal power, and the power range channels are set at $\leq 25\%$ of rated thermal power. This will make the intermediate range trip setpoint in those two specifications consistent with the setpoint proposed for Table 2.2-1 (Reactor Trip System Instrumentation Setpoints) which was previously submitted.

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Attached are the revised Technical Specification 3.10.3 and 3.10.4 pages for North Anna Units 1 and 2. The safety evaluation providing the basis for our determination that this proposed change does not involve a significant hazard consideration previously submitted remains unchanged. The conclusion that the proposed changes do not involve an unreviewed safety question as defined in 10 CFR 50.59 or a significant hazards consideration as defined in 10 CFR 50.92 remains valid.

Should you have any questions or require additional information, please contact us.

Very truly yours,



W. L. Stewart
Senior Vice President - Nuclear

Attachments

cc: United States Nuclear Regulatory Commission
Region II
101 Marietta Street, N.W.
Suite 2900
Atlanta, GA 30323

Mr. M. S. Lesser
NRC Senior Resident Inspector
North Anna Power Station

Commissioner
Department of Health
Room 400
109 Governor Street
Richmond, Virginia 23219