

ENCLOSURE 1

Inservice Inspector Program  
for Sequoyah Nuclear Plant, Unit 1

Revisions 2 through 5 to SI-114.1

8409180297 840913  
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## REVISION HISTORY

### REVISION 2

Pages 50-65D, 88A, 88B, 89-91, 191A1-191U1

### REVISION 3

Pages 61-64, 65A

Affected pages 65B-65D of 65

### REVISION 4

Pages: Punchlist, Table of Contents, 46-65, 65A-65D, 88, 91, 107,  
116, 177, 178, 190, 191V1, 191W1, 191X1

REVISION 2

by Section XI of the ASME Code. This augmented examination does not require a special report. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

20.3 Reactor Coolant Pump Flywheel

The augmented examination requirements of the reactor coolant pump flywheel are included in Regulatory Position C.4.b of Regulatory Guide 1.14; (1) an inplace ultrasonic examination of the areas of higher stress concentration at the bore and keyway at approximately three-year intervals during the refueling or maintenance shutdown coinciding with the inservice inspection schedule as required by Section XI of the ASME Code, and (2) a surface examination of all exposed surfaces and complete ultrasonic examination at approximately 10-year intervals during the plant shutdown coinciding with the inservice inspection schedule as required by Section XI of the ASME Code.

This augmented examination does not require a special report unless the examination reveals a flaw. If the examination and evaluation indicate an increase in flaw size or growth rate greater than predicted for the service life of the flywheel, the results of the examination and evaluation should be submitted to the NRC for evaluation. Refer to Regulatory Guide 1.14 for information to be included. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

20.4 Pressurizer Relief Line

The augmented examination requirements of the pressurizer relief line are included in the Technical Specifications 4.0.5 and 4.4.3.2.4. The pressurizer relief line and repair welds shall be examined using improved ultrasonic detection and evaluation procedures prior to entering Mode 4 whenever the plant has been in cold shutdown for 72 hours or more if the examination has not been performed in the previous six months.

In the event these six-month period examinations find the piping free of unacceptable indications for three successive inspections, the inspection interval shall be extended to 36 month intervals ( $\pm 12$  months to coincide with a scheduled refueling outage). The report shall be submitted with the Final Inservice Inspection Report.

20.5 Steam Generator Tubing

The augmented examination requirements of the steam generator tubing are included in Technical Specifications 4.0.5, 4.4.5.0 and section 7.3.8 of this program. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the NRC within 15 days.

Plant management shall report this information to the Nuclear Regulatory Commission Region II Office within the time period specified. See Plant Instruction AI-18 for reporting instructions.

The complete results of the steam generator tube inservice inspection shall be submitted to the Nuclear Regulatory Commission in a special report pursuant to Technical Specification 6.9.2 within 12 months following completion of the inspection. The Chemical, Metallurgy, and Standards Group Staff Specialist shall prepare this special report and submit the report of the Nuclear Regulatory Commission Region II Office within the stated time period. This special report shall include:

1. Number and extent of tubes inspected.
2. Location and percent of wall-thickness penetration for each indication of an imperfection.
3. Identification of tubes plugged.

Results of steam generator tube inspections which fall into Technical Specification Category C-3 require prompt notification of the Nuclear Regulatory Commission pursuant to Technical Specification 6.9.1 prior to resumption of plant operation.

The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures to prevent recurrence.

Refer to Technical Specifications 4.0.5 and 4.4.5.0 and section 7.3.8 of this program for information to be included.

20.6 RPV Nozzle Cladding

All vessel nozzles cladding shall be ultrasonically examined at the end of each 10-year inspection interval, using techniques at least as sensitive as those used to conduct the supplemental examinations performed prior to fuel loading. The results of this examination shall be reported to NRC. This examination is done in accordance with Sequoyah technical specifications and satisfies Surveillance Requirement 4.4.10. This augmented examination does not require a special report. This examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

Nozzle forging material and cladding is identified in section 7.1.2 of this program.

20.7 RPV Closure Head Circumferential Weld (W09-10)

See subsection 7.1.1.3 of this program for augmented examination requirements. This augmented examination does not require a special report. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

21.0 REFERENCES

- 21.1 ASME Boiler and Pressure Vessel Code - Section XI 1974 Edition Summer 1975 addenda, Summer 1976 addenda; 1977 Edition, Summer 1978 addenda.
- 21.2 ASME Boiler and Pressure Vessel Code - Section V.
- 21.3 Instruction Manual - 173-inch I.D. Reactor Pressure Vessel - Rotterdam Dockyard Company, Contract No. 68C60-91934, N2M-2-3.
- 21.4 Sequoyah Nuclear Plant Standard Practice SQA41.
- 21.5 Sequoyah Nuclear Plant Final Safety Analysis Report.
- 21.6 Westinghouse Technical Manual - Pressurizer, TM 1440-C225, Contract No. 68C60-91934, N2M-2-6.
- 21.7 Westinghouse Technical Manual - Vertical Steam Generators, TM 1440-C224, Contract No. 68C31-91934, N2M-2-4.
- 21.8 Westinghouse Instruction Manual - Auxiliary Heat Exchangers, Contract No. 68C60-91934, N2M-2-25.

- 21.9 Westinghouse Instruction Book - Reactor Coolant Pump, Contract No. 68C60-91934, N2M-2-5.
- 21.10 Ingersoll-Rand Instruction Manual - Residual Heat Removal Pumps, Contract No. 68C60-91934, N2M-2-30.
- 21.11 Sequoyah Nuclear Plant Operational Quality Assurance Manual, Part II, Sections 2.3, 3.0, 5.1, and 6.3.
- 21.12 Sequoyah Nuclear Plant Technical Instruction TI-51.
- 21.13 Sequoyah Nuclear Plant Technical Specifications Unit 1.
- 21.14 Division Procedure Manual: N75C01, N76A10, N80E3, N83A3, SEQ80M7, SEQ82E1, and SQ82M1.

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APPENDIX A  
TABLE C  
SEQUOYAH IN-SERVICE INSPECTION PROGRAM  
CLASS C COMPONENTS

Component	Program Reference Section	Method of Insp.	Section XI Exam Category	40 Yr. Sample No. Welds/of Weld	1st Insp. Interval No. Length Welds/of Weld	Inspection Periods			Reference Dwg. No.
						3 Yrs.	7 Yrs.	10 Yrs.	
A. <u>Piping</u>									
1. All Classes C and D	9.0, 9.1	VT-2	D-A, D-B, D-C	N/A	N/A	100%	100%	100%	N/A
B. <u>Component Supports</u>									
1. Auxiliary Feedwater System	9.0.1.1	VT-3, VT-4	D-A	31	31	31	31	31	ISI-0113-C
2. Chemical and Volume Control System	9.0.1.2	VT-3, VT-4	D-A	7	7	7	7	7	ISI-0114-C
3. Component Cooling System	9.0.1.3	VT-3, VT-4	D-A, D-B	268	268	268	268	268	ISI-0126-C
4. Containment Spray System	9.0.1.4	VT-3, VT-4	D-B	25	25	25	25	25	ISI-0125-C
5. Essential Raw Cooling Water System	9.0.1.5	VT-3, VT-4	D-A, D-B	342	342	342	342	342	ISI-0123-C
6. Fuel Pool Cooling System	9.0.1.6	VT-3, VT-4	D-C	47	47	47	47	47	ISI-0127-C



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SEQUOYAH IN-SERVICE INSPECTION PROGRAM  
CLASS C COMPONENTS

Component	Program Reference Section	Method of Insp.	Section XI Exam Category	40 Yr. Sample No. Length Welds/of Weld	1st Insp. Interval No. Length Welds/of Weld	Inspection Periods			Reference Dwg. No.
						3 Yrs.	7 Yrs.	10 Yrs.	
B. <u>Component Supports</u> (Continued)									
7. Residual Heat Removal System	9.0.1.7	VT-3, VT-4	D-B	7	7	7	7	7	ISI-0115-C
8. Safety Injection System	9.0.1.8	VT-3, VT-4	D-B	24	24	24	24	24	ISI-0116-C

APPENDIX A  
 TABLE D  
LIST OF DRAWINGS - UNIT 1

Reactor Vessel

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
CH-M-2333-B	Reactor Coolant Piping	119-120
CH-M-2337-C	Upper Head Injection System	141-143
CH-M-2341-B	Reactor Vessel Stud Locations and Details	92-93
CH-M-2343-B	Reactor Vessel Seam Welds	94
CH-M-2358-A	Reactor Vessel Closure Head	95
CH-M-2359-A	Control Rod Drive Housing	96-97
CH-M-2360-A	Reactor Vessel Inlet Nozzles	98
CH-M-2651-C	CRD, UPI and Vent Pipe Penetrations	99-100
CH-M-2361-A	Reactor Vessel Outlet Nozzles	101
ISI-0014-A	Auxiliary Head Adapter	102
ISI-0016-A	Reactor Vessel Clad Patches (PSI only)	103
MSG-0001-B	Closure Head Cladding Patches (PSI only)	104
MSG-0004-C	Reactor Vessel Bottom Head Penetrations	105

Pressurizer

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
CH-M-2362-A	Pressurizer Support Skirt Weld	106
CH-M-2663-A	Pressurizer Seam Welds	107
MSG-0002-B	Pressurizer and Steam Generator Cladding Patches	108
MSG-0006-A	Pressurizer Heater Penetrations	109

Steam Generators

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
CH-M-2333-B	Reactor Coolant Piping	119-120
CH-M-2345-B	Steam Generator	110
76M1	Vertical Steam Generators (Tube Sheet Arrangement)	111
MSG-0002-B	Pressurizer and Steam Generators Cladding Patches	
MSG-0005-A	Steam Generator/Feedwater Transition Spool Piece	112

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Heat Exchangers

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
CH-M-2404-A	Residual Heat Removal Heat Exchanger Channel Welds	113
ISI-0066-A	Regenerative Heat Exchanger	114-116
ISI-0067-A	Excess Letdown Heat Exchanger	117
ISI-0068-A	Letdown Heat Exchanger	118

Piping and Valve Weld Maps

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
CH-M-2333-B	Reactor Coolant Piping (Main Loops)	119-120
CH-M-2333-C	Safety Injection System	121-127
CH-M-2334-C	Reactor Coolant System	128-134
CH-M-2335-C	Chemical and Volume Control System	135-136
CH-M-2336-C	Residual Heat Removal System	137-140
CH-M-2337-C	Upper Head Injection System	141-143
CH-M-2338-C	Seal Water Injection (Chemical and Volume Control System)	144-147
CH-M-2339-C	Feedwater System	148-149
CH-M-2340-C	Main Steam System	150-151
CH-M-2422-C	Containment Spray System	152

Class A and B Piping and Valve  
Support Drawings

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
CH-M-2432-C	Reactor Coolant System	153-159
CH-M-2433-C	Chemical and Volume Control System	160-161
CH-M-2434-C	Seal Water Injection (Chemical and Volume Control)	162-165
CH-M-2435-C	Residual Heat Removal System	166-169
CH-M-2436-C	Safety Injection System	168-176
CH-M-2437-C	Upper Head Injection System	177-179
CH-M-2438-C	Main Steam Line System	180-181
CH-M-2439-C	Feedwater System	182-183
CH-M-2440-C	Containment Spray System	184

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LIST OF DRAWINGS - UNIT 1

Class C and D Piping and Valve  
 Support Drawings

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
ISI-0113-C	Auxiliary Feedwater System	191A-191B
ISI-0114-C	Chemical and Volume Control System	191C
ISI-0115-C	Residual Heat Removal System	191D
ISI-0116-C	Safety Injection System	191E-191F
ISI-0123-C	Essential Raw Cooling Water System	191G-191Z
ISI-0125-C	Containment Spray System	191A1-191B1
ISI-0126-C	Component Cooling Water System	191C1-191R1
ISI-0127-C	Fuel Pool Cooling System	191S1-191U1

Pumps

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
CH-M-2675-B	Reactor Coolant Pump Main Flange and Lower Seal House Bolt Pattern	185
MSG-0003-B	Reactor Coolant Pump Casing	186
ISI-0099-A	Residual Heat Removal Pump Supports	187

Tanks

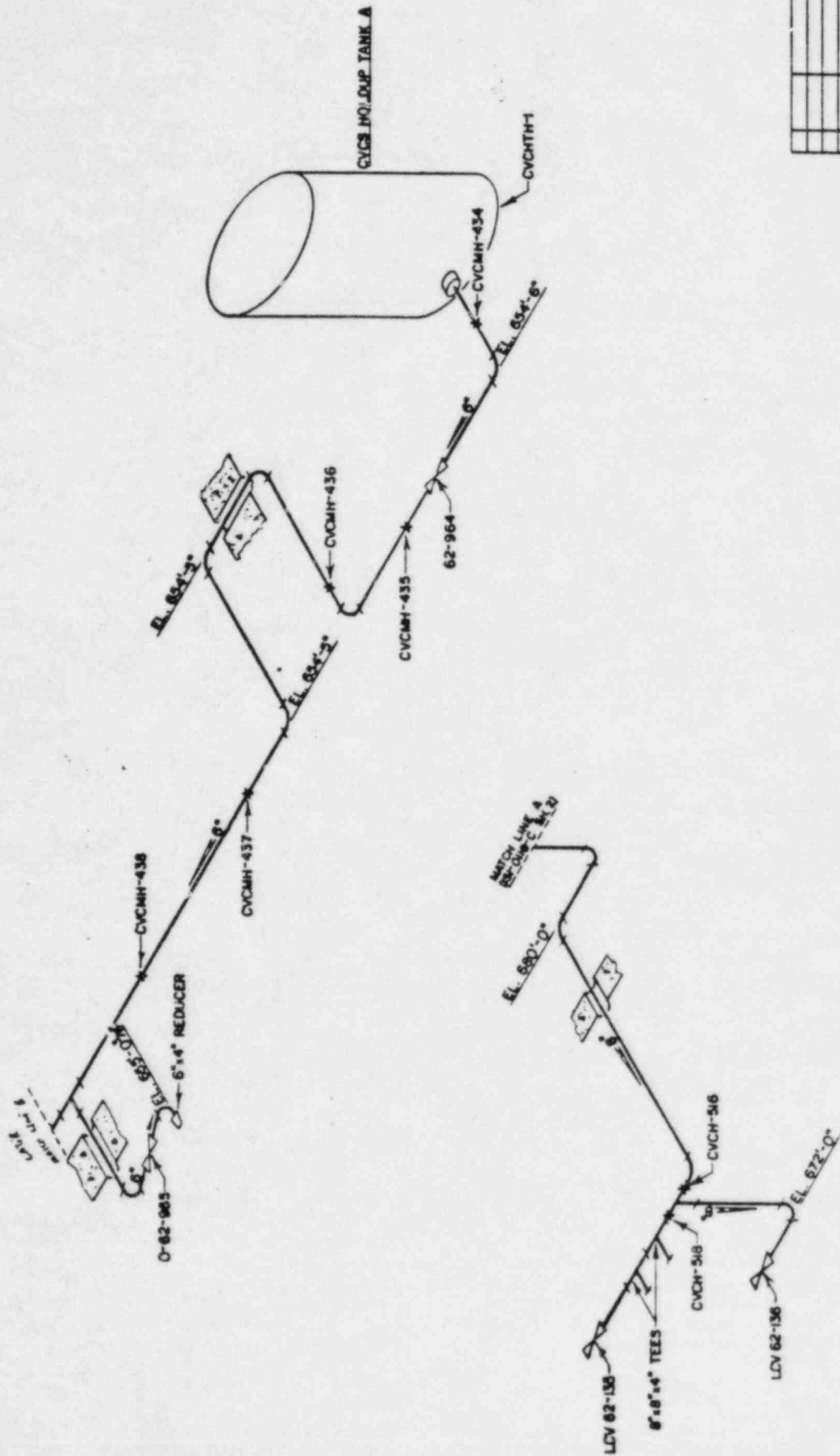
<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
ISI-0069-A	Boron Injection Tank (Unit 1)	188
ISI-0070-A	Upperhead Injection Water Accumulator	189-190
ISI-0071-A	Accumulator Surge Tank	191





REFERENCE DRAWINGS  
 47K203-39  
 47K208-39  
 TWA SAFETY CLASS C

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TRANSMISSION VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
 UNIT #1  
 CHEMICAL AND VOLUME CONTROL SYSTEM  
 SUPPORT LOCATIONS

DESIGNED BY: [Signature]  
 CHECKED BY: [Signature]  
 DRAWN BY: [Signature]  
 APPROVED BY: [Signature]

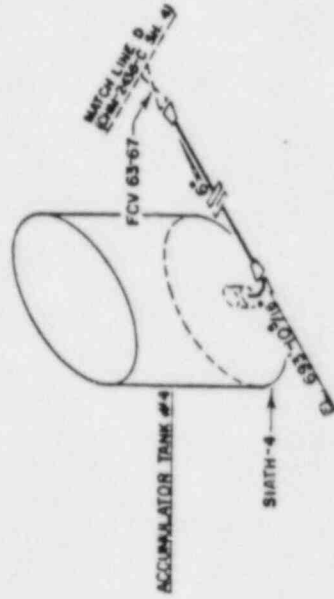
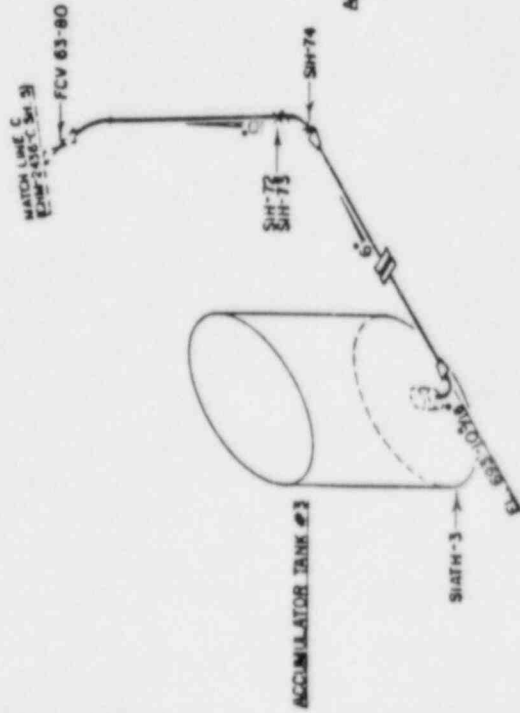
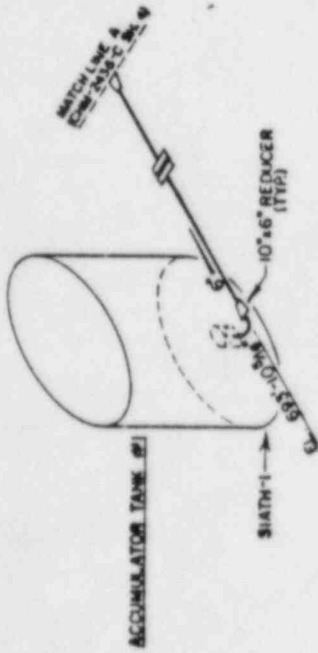
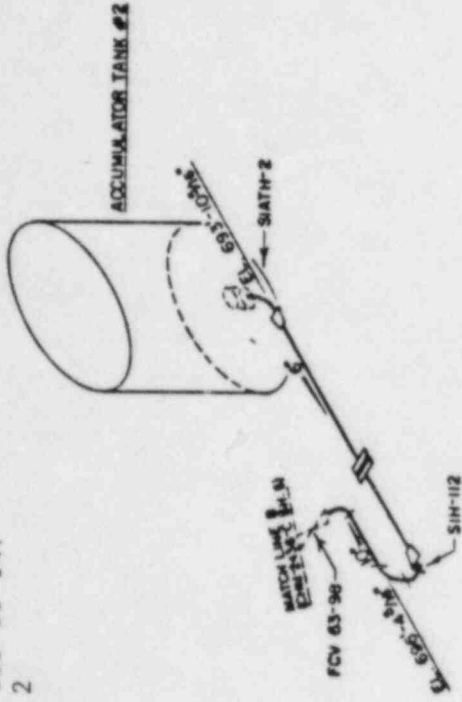
SI-0114-C R0





REFERENCE DRAWINGS  
 0600102-09-0  
 0600102-09-02  
 TWA SAFETY CLASS C

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NO.	DATE	REVISIONS	BY	DATE

DESIGNED BY: *BBB*  
 DRAWN BY: *BBB*  
 CHECKED BY: *BBB*  
 APPROVED BY: *BBB*

SI-114.1  
 SI-0116-C



































REFERENCE DRAWINGS

- 47K450-50
- 47K450-56
- 47K450-67
- 47K450-68

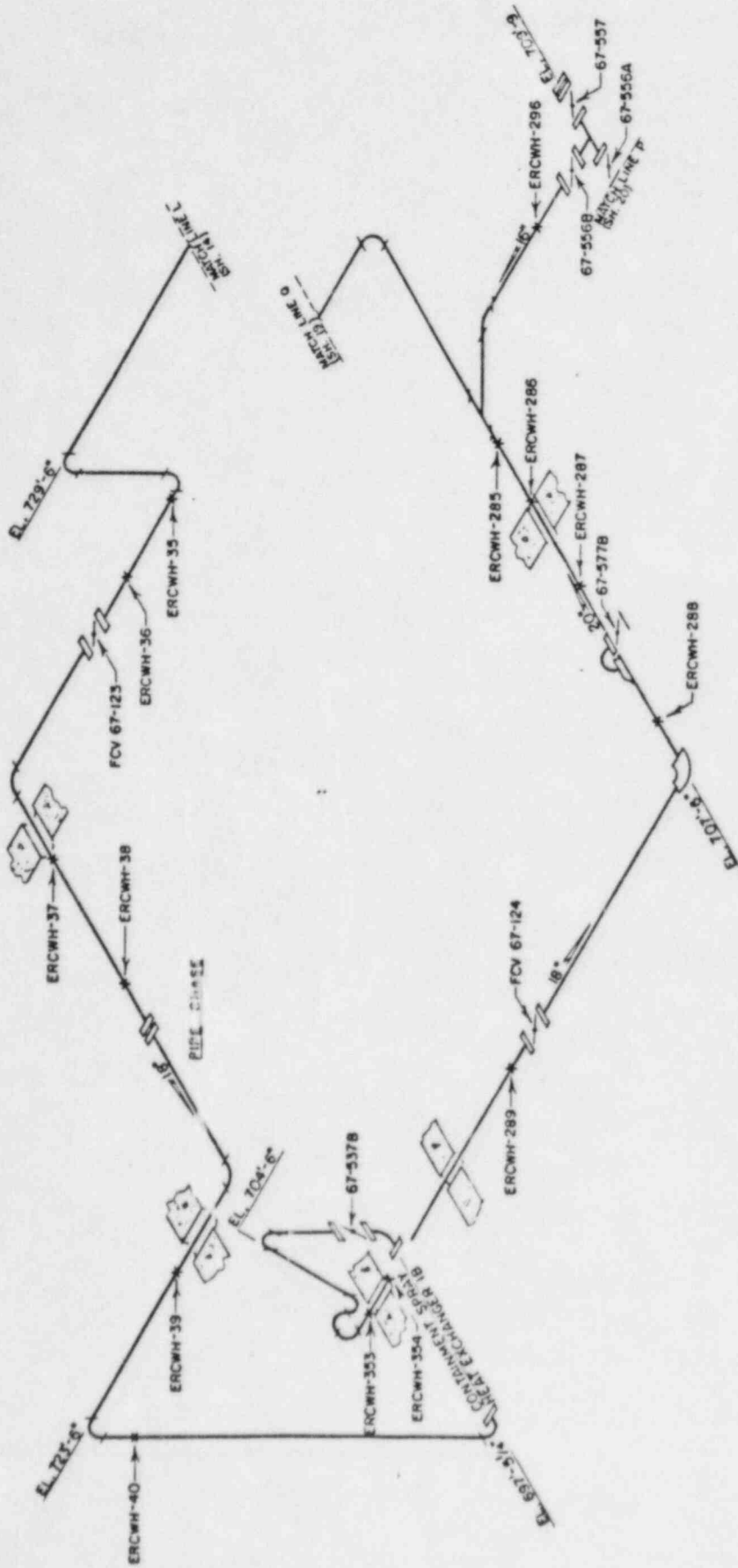
TVA SAFETY CLASS C

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REVISIONS

SEQUIOAH NUCLEAR PLANT

UNIT #1

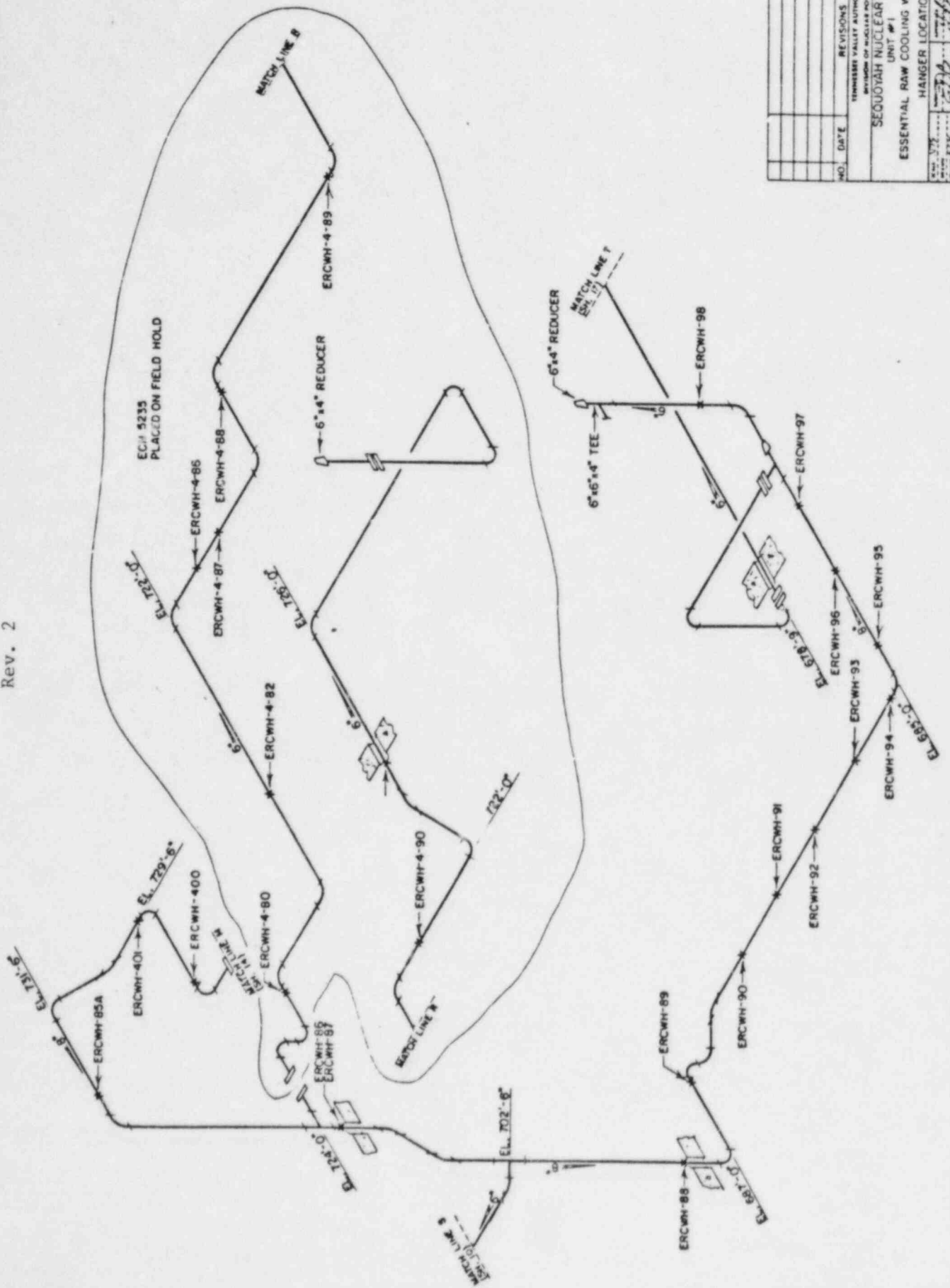
ESSENTIAL RAW COOLING WATER SYSTEM

HANGER LOCATIONS

SI-0123-C

REFERENCE DRAWINGS  
 47K450-51  
 47K450-231  
 47W450-SERIE9  
 TVA SAFETY CLASS C

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TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOYAH NUCLEAR PLANT  
 UNIT # 1  
 ESSENTIAL RAW COOLING WATER SYSTEM  
 HANGER LOCATIONS  
 DRAWN BY: [Signature]  
 CHECKED BY: [Signature]  
 DATE: 11/18/88  
 PROJECT NO.: SI-0123-C







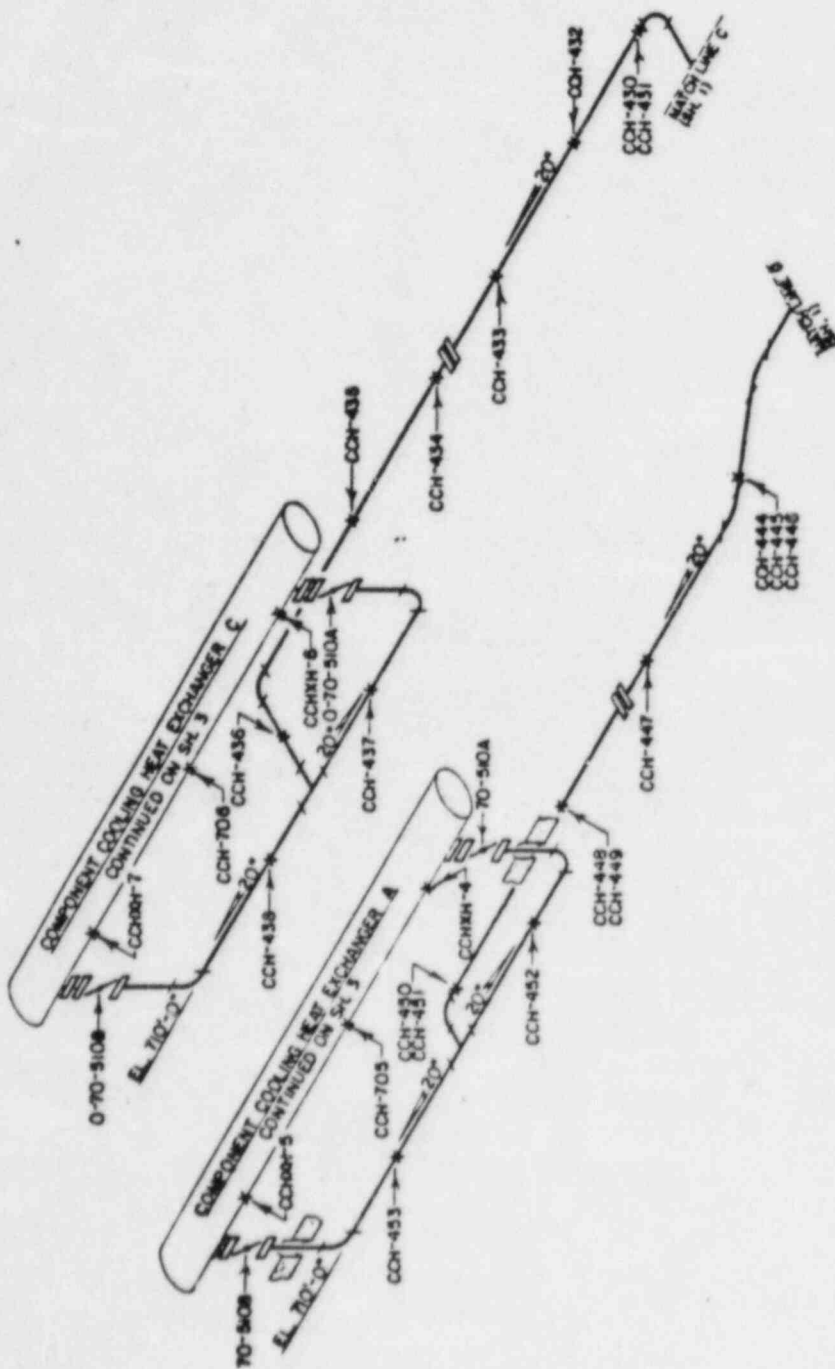












NO	DATE	REVISIONS

MEMORANDUM VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOYAH NUCLEAR PLANT  
 UNIT #1  
 COMPONENT COOLING SYSTEM  
 SUPPORT LOCATIONS

SI-0126-C #0











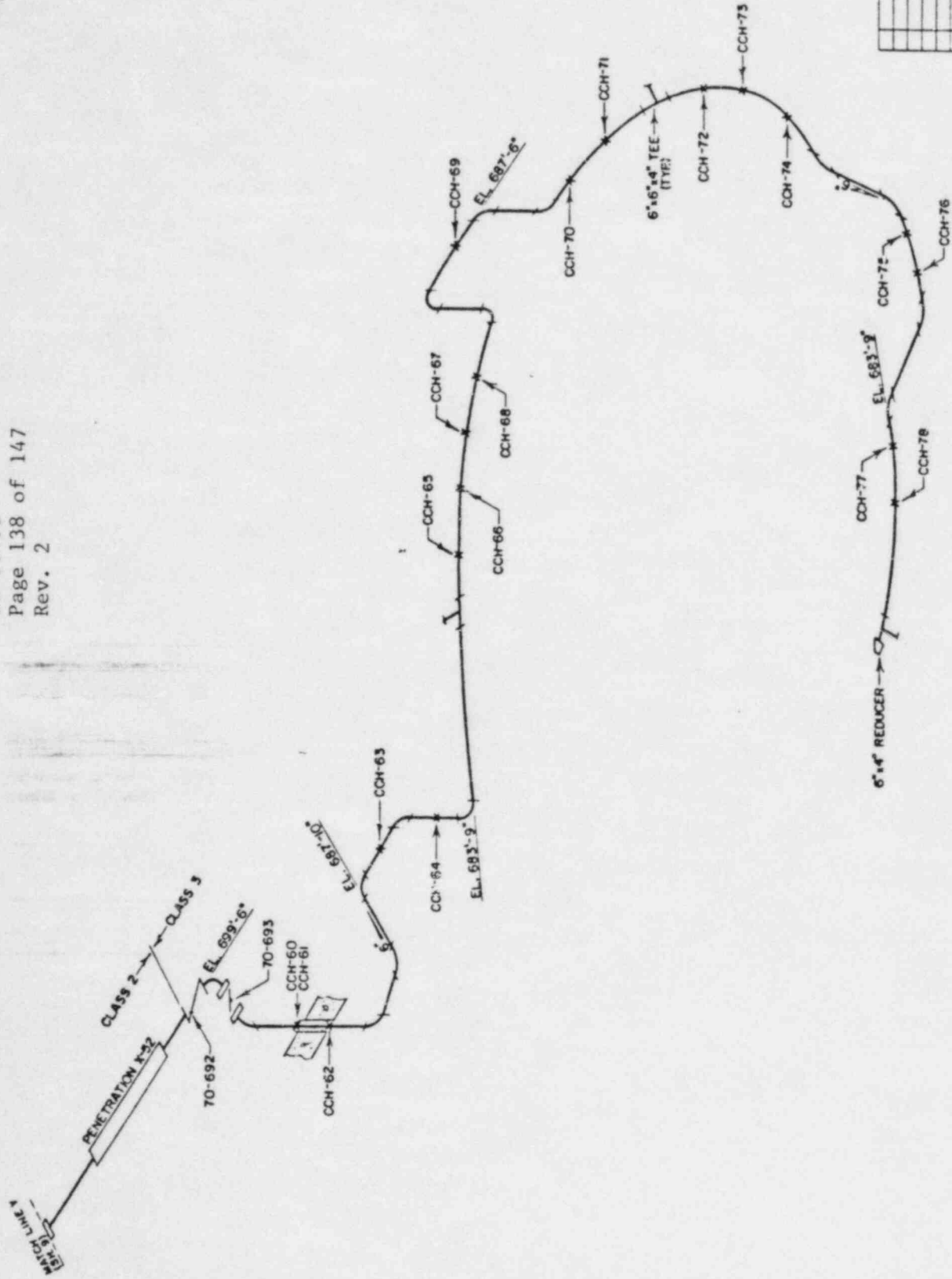






REFERENCE DRAWINGS  
 47K464-112  
 TVA SAFETY CLASS C

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TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOYAH NUCLEAR PLANT  
 UNIT #1  
 COMPONENT COOLING SYSTEM  
 HANGER LOCATIONS

DESIGNED BY: [Signature]  
 CHECKED BY: [Signature]  
 DATE: [Date]  
 SCALE: [Scale]  
 SHEET NO.: [Number]  
 TOTAL SHEETS: [Number]  
 SI-0126-C  
 80

















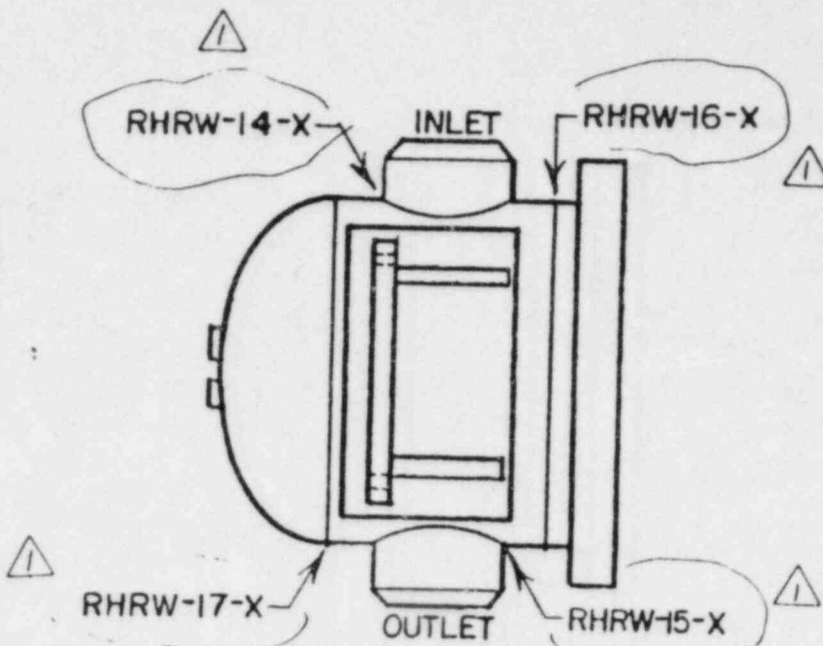
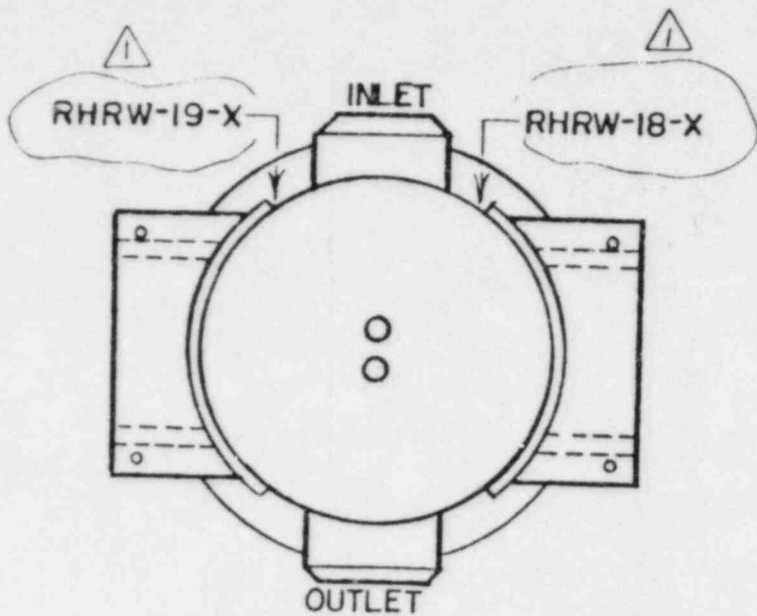




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NOTES

- REPLACE THE X IN THE WELD NUMBER WITH THE APPROPRIATE HEAT EXCHANGER NUMBER.



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1	1-9-84	REVISE WELD NOS. & ADD NOTES KEV	<i>EDC</i>

TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
 UNIT #1 & #2  
 RESIDUAL HEAT REMOVAL HEAT EXCHANGER  
 CHANNEL WELDS

SCALE: <i>NTS</i>	SUBMITTED: <i>EDC</i>	APPROVED: <i>HYB</i>	DATE: 4/19/77
DRAWN: <i>KEV</i>	TRACED: <i>EDC</i>	CHECKED: <i>EDC</i>	SHEET 1 OF 1 SHEETS
			CHM-2404-A

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The ISI Report shall have a cover sheet providing the following information:

- (1) Date of document completion.
- (2) Name and address of owner.
- (3) Name and address generating plant.
- (4) Name or number assigned to the nuclear power unit by TVA.
- (5) Commercial operation date for unit.

All reports shall provide the following information as a minimum.

- (a.) Numbers assigned to the components by the state.
- (b.) National Board Number assigned to the components by the manufacturer.
- (c.) Names of the components and descriptions including size, capacity, material, location, and drawings to aid identification.
- (d.) Name and address of principal manufacturer (e.g., Westinghouse, GE, etc.) and the principal contract number which will identify the subcontractors.
- (e.) Manufacturer's component identification numbers.
- (f.) Date of completion of the examination, (test, replacement, or repair).
- (g.) Name of ANII who witnessed or otherwise verified the examinations, (tests, replacements, or repairs), and his employer and business address, when required.
- (h.) Abstract of examinations, (tests, replacements, or repairs performed); conditions recorded; and corrective measures recommended or taken.
- (i.) Signature of ANII, when required.
- (j.) Owner's Data Report for inservice inspection, Form NIS-1, as shown in Appendix II of ASME Section XI.

- (k.) Completed examination reports.
- (l.) Completed calibration reports.
- (m.) List of component drawings.
- (n.) List of TVA NDE personnel and/or copies of contractor personnel certifications.
- (o.) List of TVA NDE equipment and/or copies of contractors equipment certifications.
- (p.) List of TVA NDE procedures used and/or copies if contractor NDE procedures.

The NDE Section shall submit the inservice examination report to the plant manager for retention as a quality assurance record in accordance with N-OQAM, Part III, Section 4.1. The NDE Section shall maintain a copy of the report for information, and shall submit applicable summaries of the report with a cover sheet as described above to the NRC via the Manager, Nuclear Licensing Branch, Office of Power.

16.3 Records for Classes A, B, C, and D Components

The following records shall be available for review:

- (1) Examination Plans
- (2) Examination Results and Reports
- (3) Examination Methods and Procedures
- (4) Evaluation of Results
- (5) Corrective Actions and Repairs

16.4 Records of System Pressure Tests

Records of the visual examinations conducted during system leakage or hydrostatic tests shall consist of an itemization of the number and location of leaks found in a system and the corrective actions taken.

16.5 Augmented Examination Reports

Augmented examination special reports shall be submitted to the Nuclear Regulatory Commission Region II Office within the time period specified for each report. For specific details on records, reports and reporting see Section 20.0, Augmented Inspections, of this program.

17.0 NOTIFICATION OF INDICATION

Plant management shall be formally notified of the presence of unacceptable indications detected during the performance of nondestructive examinations (excluding VT-2, visual examinations performed during system pressure tests). Unacceptable indications are defined by the applicable NDE procedure. Formal notification shall consist of completing and submitting to the Plant Superintendent the "Notification of Indication" form in Appendix D of this program.

Part I of the "Notification of Indication" shall be completed and signed by the NDE Level II or III examiner detecting the indication. The NDE Section Representative of the Quality Engineering Branch shall assign a sequential number review and sign the form. If the indication is detected by an outside contractor, the contractor's field supervisor shall review and sign the form. The original shall be sent to the plant superintendent and a copy to the ISI Programs Section.

The plant manager or his assistant shall be responsible for determining which organization (Outage Modification and Major Maintenance, Plant Maintenance) shall be responsible for preparing a disposition in Part II of the form and performing the associated corrective action. If the organization assigned responsibility for disposition is unable to determine a satisfactory disposition then the form should be sent to the Mechanical Branch for disposition. The individual responsible for preparation of the disposition shall sign and date Part II of the form. The cognizant supervisor of the appropriate organization shall review and approve the disposition and sign and date Part II of the form. Copies of the form shall be distributed to the plant manager and the ISI Programs Section. The original shall be returned to the NDE Section Representative. One copy shall be filed with the examination report.

If the organization assigned responsibility for repair is within NUC PR, they shall determine if the unacceptable condition is significant and potentially reportable in accordance with the requirements of N-OQAM, Part III, Section 7.2. Dispositions to correct the condition under the plant maintenance program shall



be processed in accordance with N-OQAM, Part II, Section 2.1. Dispositions other than restoring to original requirements shall be processed as modifications in accordance with N-OQAM, Part II, Section 3.0. Repair and replacement, activities, including coordination with the Authorized Inspection Agency (AIA), shall be performed in accordance with the requirements of N-OQAM, Part II, Section 2.3. Dispositions to accept the condition as-is shall include in Part II of the form the basis for the disposition. In addition for dispositions to accept the condition as-is, a USQD shall be prepared by the appropriate organization in accordance with established procedures. The organization responsible for corrective action shall include preservice examination requirements in the repair or replacement work instruction described in N-OQAM, Part II, Section 2.3.

Upon completion of corrective action the NDE Section Representative shall verify completion of corrective action, enter the work instruction and/or DCR numbers on the Notification of Indication form, enter the examination report number if re-examination was performed, and sign and date the form, Part III. The signed form shall remain with the examination report for use as a quality assurance record. If re-examination was performed, a copy of the signed form shall also remain with the re-examination report. Copies of the form shall also be distributed to those listed in the second paragraph above.

#### 18.0 CALIBRATION BLOCKS

Calibration blocks will be used for ultrasonic examinations (a calibration tube will be used for eddy current examination of steam generator tubing). The blocks will be fabricated to the general requirements of ASME Section V and ASME Section XI. The blocks shall be fabricated of the material to be examined or equivalent P numbers. Mill test reports shall be obtained and retained by the Quality Engineering Branch for all calibration blocks. The blocks shall employ drilled holes and/or notches for calibration reflectors (See Request for Relief ISI-2).

The NDE Section shall maintain as-built calibration block drawings. Copies of the original drawings and any revisions shall be submitted to the ISI Programs Section. The calibration blocks shall be stored at the plant site and maintained by the plant QA organization.

## 19.0 REQUESTS FOR RELIEF

Where TVA has determined that certain code requirements or examinations are impractical, TVA will submit written requests for relief to NRC with information to support the determinations and any proposed alternate examinations. The impractical code requirements or examinations shall be identified in this program, and references to particular requests for relief shall be included.

When impractical examination requirements are identified in the field, the NDE Section shall notify the ISI Programs Section such that the information may be included in this program and requests for relief may be prepared if necessary.

The Requests for Relief are listed in Appendix E.

## 20.0 AUGMENTED INSPECTIONS

Augmented inspections are performed in addition to ASME Section XI code requirements. The augmented inspections may be required by the NRC or self-imposed by TVA.

### 20.1 Feedwater System Piping and Supports

In order to satisfy the requirements of NRC-IE Bulletin 79-13, the following augmented examinations shall be performed.

The augmented examination of the feedwater nozzle-to-pipe welds (includes nozzle-to-transition piece welds and transition piece-to-pipe welds) and of adjacent pipe and nozzle areas are included in DPM No. SEQ80M7. The requirements are a radiographic examination supplemented by ultrasonic examination on completion of the hot functional testing and before fuel loading. During the first refueling outage, perform a volumetric examination of the feedwater nozzle-to-pipe welds, the feedwater piping welds to the first support, the feedwater line-to-containment penetration welds and an area of at least one pipe diameter of the main feedwater line downstream at the auxiliary feedwater to main feedwater connection. Also perform a visual inspection of all feedwater system piping supports and snubbers in containment to verify operability and conformance to design.

In the event cracking is identified during examination of the nozzle-to-pipe welds, all feedwater lines up to the first piping support or snubber outboard of the nozzle shall be volumetrically examined as stated in the first paragraph. All unacceptable code discontinuities shall be subject to repair unless justification for continued operation is provided.

Any cracking or other unacceptable code discontinuities identified shall be reported to the Director of the NRC Regional Office within 24 hours of identification. Plant management shall report this information to the Nuclear Regulatory Commission Region II Office per Plant Instruction AI-18. A written report of the results of examination shall be submitted by the QEB NDE Section within 30 days of completion of the examination. Refer to DPM No. SEQ80M7 for information to be included in this special report.

The requirements of NRC-IE Bulletin 79-13 were satisfied during the unit 1, cycle 1 outage. This information has been retained as historical information and is no longer applicable to Sequoyah Nuclear Plant, unit 1.

#### 20.2 RPV Nozzle Safe Ends

The augmented examination requirements of the RPV nozzle-to-safe end welds are included in the final report - Sequoyah Nuclear Plant - Evaluation of cracking in reactor vessel nozzle stainless steel buttering. The examinations for unit 1 will be monitored at the normal inservice inspection intervals for dissimilar metal welds as required by Section XI of the ASME Code. This augmented examination does not require a special report. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

#### 20.3 Reactor Coolant Pump Flywheel

The augmented examination requirements of the reactor coolant pump flywheel are included in Regulatory Position C.4.b of Regulatory Guide 1.14; (1) an in-place ultrasonic examination of the areas of higher stress concentration at the bore and keyway at approximately three-year intervals during the refueling or maintenance shutdown coinciding with the inservice inspection schedule as required by Section XI of the ASME Code, and (2) a surface examination of all exposed surfaces and complete ultrasonic examination at approximately 10-year intervals during the plant shutdown coinciding with the inservice inspection schedule as required by Section XI of the ASME Code.

This augmented examination does not require a special report unless the examination reveals a flaw. If the examination and evaluation indicate an increase in flaw size or growth rate greater than predicted for the service life of the flywheel, the results of the examination and evaluation should be submitted to the NRC for evaluation. Refer to Regulatory Guide 1.14 for information to be included. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

20.4 Pressurizer Relief Line

The augmented examination requirements of the pressurizer relief line are included in the Technical Specifications 4.0.5 and 4.4.3.2.4. The pressurizer relief line and repair welds shall be examined using improved ultrasonic detection and evaluation procedures prior to entering Mode 4 whenever the plant has been in cold shutdown for 72 hours or more if the examination has not been performed in the previous six months.

In the event these six-month period examinations find the piping free of unacceptable indications for three successive inspections, the inspection interval shall be extended to 36 month intervals ( $\pm 12$  months to coincide with a scheduled refueling outage). The report shall be submitted with the Final Inservice Inspection Report.

20.5 Steam Generator Tubing

The augmented examination requirements of the steam generator tubing are included in Technical Specifications 4.0.5, 4.4.5.0 and section 7.3.8 of this program. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the NRC within 15 days.

Plant management shall report this information to the Nuclear Regulatory Commission Region II Office within the time period specified. See Plant Instruction AI-18 for reporting instructions.

The complete results of the steam generator tube inservice inspection shall be submitted to the Nuclear Regulatory Commission in a special report pursuant to Technical Specification 6.9.2 within 12 months following completion of the inspection. The Chemical, Metallurgy, and Standards

Group Staff Specialist shall prepare this special report and submit the report of the Nuclear Regulatory Commission Region II Office within the stated time period. This special report shall include:

1. Number and extent of tubes inspected.
2. Location and percent of wall-thickness penetration for each indication of an imperfection.
3. Identification of tubes plugged.

Results of steam generator tube inspections which fall into Technical Specification Category C-3 require prompt notification of the Nuclear Regulatory Commission pursuant to Technical Specification 6.9.1 prior to resumption of plant operation.

The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures to prevent recurrence.

Refer to Technical Specifications 4.0.5 and 4.4.5.0 and section 7.3.8 of this program for information to be included.

#### 20.6 RPV Nozzle Cladding

All vessel nozzles cladding shall be ultrasonically examined at the end of each 10-year inspection interval, using techniques at least as sensitive as those used to conduct the supplemental examinations performed prior to fuel loading. The results of this examination shall be reported to NRC. This examination is done in accordance with Sequoyah technical specifications and satisfies Surveillance Requirement 4.4.10. This augmented examination does not require a special report. This examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

Nozzle forging material and cladding is identified in section 7.1.2 of this program.

#### 20.7 RPV Closure Head Circumferential Weld (W09-10)

See subsection 7.1.1.3 of this program for augmented examination requirements. This augmented examination does not require a special report. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

21.0 REFERENCES

- 21.1 ASME Boiler and Pressure Vessel Code - Section XI 1974 Edition Summer 1975 addenda, Summer 1976 addenda; 1977 Edition, Summer 1978 addenda.
- 21.2 ASME Boiler and Pressure Vessel Code - Section V.
- 21.3 Instruction Manual - 173-inch I.D. Reactor Pressure Vessel - Rotterdam Dockyard Company, Contract No. 68C60-91934, N2M-2-3.
- 21.4 Sequoyah Nuclear Plant Standard Practice SQA41.
- 21.5 Sequoyah Nuclear Plant Final Safety Analysis Report.
- 21.6 Westinghouse Technical Manual - Pressurizer, TM 1440-C225, Contract No. 68C60-91934, N2M-2-6.
- 21.7 Westinghouse Technical Manual - Vertical Steam Generators, TM 1440-C224, Contract No. 68C31-91934, N2M-2-4.
- 21.8 Westinghouse Instruction Manual - Auxiliary Heat Exchangers, Contract No. 68C60-91934, N2M-2-25.
- 21.9 Westinghouse Instruction Book - Reactor Coolant Pump, Contract No. 68C60-91934, N2M-2-5.
- 21.10 Ingersoll-Rand Instruction Manual - Residual Heat Removal Pumps, Contract No. 68C60-91934, N2M-2-30.
- 21.11 Sequoyah Nuclear Plant Operational Quality Assurance Manual, Part II, Sections 2.3, 3.7, 5.1, and 6.3.
- 21.12 Sequoyah Nuclear Plant Technical Instruction TI-51.
- 21.13 Sequoyah Nuclear Plant Technical Specifications Unit 1.
- 21.14 Division Procedure Manual: N75C01, N76A10, N80E3, N83A3, SEQ80M7, SEQ82E1, and SQ82M1.

REVISION 4

PUNCHLIST

Reactor Vessel Repair Welds Beltline Region  
Class B Systems Functional Leakage SI's



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8.10.1.2 RHRP Support Components

Each RHRP includes three support components bolted to the pump feet which are integrally forged with the pump. All component supports from each pump shall be visually examined during the inspection interval in accordance with visual examination method VT-3 (see Section 11.1). Support components extend from the RHRP to and including the attachment to the supporting structure.

Two of the supports from one RHRP shall be examined during the first inspection period. The final (3rd) support from the same RHRP shall be examined during the second inspection period. All of the supports from the remaining RHRP shall be examined during the third inspection period.

8.10.1.3 RHRP Supports - Mechanical and Hydraulic

There are no mechanical and hydraulic supports associated with the RHRP.

8.10.1.4 RHRP Pressure Retaining Bolting

The stuffing box extension to pump casing connection bolting is not greater than two inches in diameter.

The connection includes 24 studs at 1-1/4 inches in diameter with nuts and washers. The studs are fabricated to SA-193, GR. B7, and the nuts to SA-194, GR. 2H.

8.10.1.5 RHRP Casing Welds

The RHRP does not include any casing welds. The casing is a one piece forging fabricated to SA-182 F304.

\*8.10.2 CVCS Centrifugal Charging Pumps (2) CCP

8.10.2.1 CCP Integrally-Welded Supports

Each CCP includes 4 integrally-welded pump feet. These integrally-welded pump feet whose design base is  $>3/4$  of an inch shall be surface examined during the inspection interval in accordance with Table B of Appendix A.

8.10.2.2 CCP Component Supports

Each CCP includes 4 integrally-welded feet bolted to the support system. All component supports from each pump shall be visually examined during the inspection interval in accordance with visual examination method VT-3 (see section 11.1). The examinations shall be distributed during the inspection interval in accordance with Table B of Appendix A. Support components extend from the CCP to and including the attachment to the supporting structure.

8.10.2.3 CCP Supports - Mechanical and Hydraulic

There are no mechanical and hydraulic supports associated with the CCP.

8.10.2.4 CCP Pressure Retaining Bolting

The CCP case head studs are not greater than 2 inches in diameter. The connection includes 16 studs at 1-3/4 inches in diameter with nuts and washers. The studs are fabricated to SA-193, GRB7, nuts to SA-194 GR2H and washers to SA-322, GR4140.

8.10.2.5 CCP Casing Welds

Each CCP casing includes a pressure retaining casing weld. The carbon steel casing is a barrel-type forging fabricated to SA-266, T CL 1 and clad with SS 18-8. The casing weld shall be surface examined during the inspection interval in accordance with Table B of Appendix A.

\*8.10.3 Safety Injection Pumps (2) SIP

8.10.3.1 STP Integrally-Welded Supports

There are no integrally-welded supports associated with the SIP.

8.10.3.2 SIP Component Supports

Each SIP includes a support component bolted to the pump casing. All component supports from each pump shall be visually examined during the inspection interval in accordance with visual examination method VT-3 (see Section 11.1). The examinations shall

be distributed during the inspection interval in accordance with Table B of Appendix A. Support components extend from the SIP to and including the attachment to the supporting structure.

8.10.3.3 SIP Supports - Mechanical and Hydraulic

There are no mechanical and hydraulic supports associated with the SIP.

8.10.3.4 SIP Pressure Retaining Bolting

The SIP case capnuts are not greater than 2 inches in diameter. The casing bolting includes 32 studs at 2 inches in diameter with nuts and washers and 10 studs at 1-1/2 inches in diameter with nuts and washers. The studs are fabricated to SA-193, GRB7 and the nuts to SA-194, GR7.

8.10.3.5 SIP Casing Welds

The SIP casing does not include any casing welds. The casing consists of an upper and lower housing fabricated to A351 CF8.

8.11 Valves

8.11.1 Valve Integrally-Welded Supports

Examination of valve integrally-welded support members is included in Section 8.9.1.

8.11.2 Valve Component Supports

Examination of valve component supports is included in Section 8.9.2.

8.11.3 Valve Pressure-Retaining Bolting

There is no Class B pressure-retaining bolting greater than 2 inches in diameter.

8.11.4 Valve Body Welds

There are no Class B valves with body welds.

8.12 Pressure Retaining Components

All Class B pressure retaining components in systems or portions of systems which are not required to operate during normal reactor operation but for which periodic system or component functional tests are required (excluding open ended portions of systems), shall be subjected to a system functional leakage test with each inspection period in accordance with IWC-5221 of ASME Section XI. This is performed in accordance with SI's to be developed later. Those Class B pressure retaining components in systems or portions of systems not subject to the functional leakage tests (excluding open-ended portions of systems), shall be subjected to a system hydrostatic test with each inspection interval in accordance with IWC-5222 of ASME Section XI. This is performed in accordance with SI-265.3. The components shall be visually examined during the pressure tests in accordance with visual examination method VT-2 (see Section 11.1).

8.13 Exempted Components

8.13.1 Exempted Components (Except Piping Welds)

Components exempted from examination include:

- (1) Components in systems where both the design pressure and temperature are equal to or less than 275 psig and 200°F, respectively;
- (2) components in systems or portions of systems, other than emergency core cooling systems, which do not function during normal reactor operation;
- (3) components which perform an emergency core cooling function for which the control of the chemistry of the contained fluid is verified by periodic sampling and test, and
- (4) component connections, piping, and associated valves, and vessels (and their supports), that are 4-inch nominal pipe size and smaller.

8.13.2 Exempted Components (Piping Welds Only)

Piping exempted from examination include:

- (a) Piping systems where both the design pressure and temperature are equal to or less than 275 psig and 200°F, respectively;
- (b) piping systems or portions of systems other than emergency core cooling systems which do not function during normal reactor operation;
- (c) piping that is 4-inch nominal pipe size and smaller;
- (d) components which perform an emergency core cooling function provided the control of the chemistry of the contained fluid is verified by periodic sampling and test.

9.0 COMPONENTS SUBJECT TO EXAMINATION - TVA SAFETY CLASSES C AND D

Classes C and D (ASME Class 3) pressure retaining components identified in sections 9.1.1 and 9.1.2 shall be visually examined, VT-2, in conjunction with the system pressure test in accordance with IWA-5000, IWD-2000, and IWD-5000.

The component supports and restraints within the boundaries identified in sections 9.1.1 and 9.1.2 for components exceeding 4-inch nominal pipe size shall be visually examined, VT-3, during each inspection period.

Mechanical and hydraulic snubbers, spring loaded and constant weight supports within the boundaries identified in sections 9.1.1 and 9.1.2 for components exceeding 4-inch nominal pipe size shall be visually examined, VT-4 during each inspection period.

9.0.1 Piping and Valve Component Supports

All piping and valve component supports shall be visually examined during each inspection period in accordance with visual examination methods VT-3 and VT-4 (see Section 11.1). This examination includes integrally-welded and nonintegrally-welded component supports. Component supports extend from the piping and valves to and including the attachment to the supporting structure. The setting of snubbers, shock absorbers and spring-type hangers shall be verified. The examinations (100%) shall be conducted during each inspection period during the inspection interval in accordance with Table C of Appendix A.

9.0.1.1 Auxiliary Feedwater System Piping and Valve Component Supports

The auxiliary feedwater system piping includes 31 component supports. All of these shall be examined during each inspection period.

9.0.1.2 Chemical and Volume Control System Piping and Valve Component Supports

The chemical and volume control system piping includes seven component supports. All of these shall be examined during each inspection period.

9.0.1.3 Component Cooling System Piping and Valve  
Component Supports

The component cooling system piping includes 268 component supports. All of these shall be examined during each inspection period.

9.0.1.4 Containment Spray System Piping and Valve  
Component Supports

The containment spray system piping includes 25 component supports. All of these shall be examined during each inspection period.

9.0.1.5 Essential Raw Cooling Water System Piping and  
Valve Component Supports

The essential raw cooling water system piping includes 342 component supports. All of these shall be examined during each inspection period.

9.0.1.6 Fuel Pool Cooling System Piping and Valve  
Component Supports

The fuel pool cooling system piping includes 47 component supports. All of these shall be examined during each inspection period.

9.0.1.7 Residual Heat Removal System Piping and Valve  
Component Supports

The residual heat removal system piping includes seven component supports. All of these shall be examined during each inspection period.

9.0.1.8 Safety Injection System Piping and Valve Component  
Supports

The safety injection system piping includes 24 component supports. All of these shall be examined during each inspection period.

9.1 System Pressure Tests

9.1.1 System Inservice Tests

Pressure retaining components within the boundary of systems or portions of systems required to operate in



support of normal plant safety functions of shutting down and maintaining the reactor in the cold shutdown condition, and pressure retaining piping, pumps, and valves within the boundary or systems or portions of systems required to operate in support of residual heat removal from spent fuel storage pool, shall be visually examined in accordance with visual examination method VT-2 (see section 11.1) while the applicable systems are in service under operating pressure in accordance with IWD-5221 of ASME Section XI.

These examinations and tests shall be performed during each inspection period.

9.1.2 System Functional Tests

Pressure retaining components within the boundary of systems or portions of systems required to operate in support of the postaccident safety functions of emergency core cooling, containment heat removal and atmospheric cleanup, and long-term residual heat removal from the reactor vessel shall be visually examined in accordance with visual examination method VT-2 (see Section 11.1) during a system functional test conducted to verify operability in systems in accordance with IWD-5222 of ASME Section XI.

9.1.3 System Hydrostatic Tests

Pressure retaining components identified in Sections 9.1.1 and 9.1.2 shall be visually examined in accordance with visual examination method VT-2 (see Section 11.1) during a system hydrostatic test conducted in accordance with IWD-5223 of ASME Section XI. These examinations and tests shall be performed over each inspection interval.

9.2 Component Supports and Restraints

Component supports and restraints within the boundaries of the systems identified in Sections 9.1.1 and 9.1.2 for components exceeding 4-inch nominal pipe size shall be visually examined each inspection period in accordance with visual examination method VT-3 (see Section 11.1).

### 9.3 Snubbers and Hangers

Mechanical and hydraulic snubbers, spring loaded, and constant weight support hangers for components exceeding 4-inch nominal pipe size shall be visually examined each inspection period in accordance with visual examination method VT-4 (see Section 11.1.)

## 10.0 AUTHORIZED INSPECTOR

TVA shall employ an Authorized Inspector(s) in accordance with ASME Section XI for inservice examinations, repairs, and replacements of TVA Class A, B, C, and D components at Sequoyah Nuclear Plant. The inspector shall verify, assure, or witness that code requirements have been met. He shall have the prerogative and authorization to require requalification of any operator or procedure when he has reason to believe the requirements are not being met. TVA shall provide access for the AI in accordance with IWA-2140 of ASME Section XI.

TVA's interface with the Authorized Inspector for inservice inspection, repairs, and replacements is defined in OQAM Part II, Section 2.3 and Part II, Section 5.1

## 11.0 EXAMINATION METHODS

### 11.1 Visual Examination

Visual examinations that require clean surfaces or decontamination for valid interpretation of results shall be preceded by appropriate cleaning processes.

#### 11.1.1 Visual Examination VT-1

The VT-1 visual examination shall be conducted to determine the condition of the part, component, or surface examined, including such conditions as cracks, wear, corrosion, erosion, or physical damage on the surfaces of the part or components.

##### 11.1.1.1 Direct Visual Examination VT-1

Direct VT-1 visual examination may be conducted when access is sufficient to place the eye 24 inches (610 mm) of the surface to be examined and at an angle not less than 30 degrees to the surface. Mirrors may be used to improve the angle

of vision. Lighting, natural or artificial shall be sufficient to resolve a 1/32 inch (0.8 mm) black line on an 18% neutral gray card.

11.1.1.2 Remote Visual Examination VT-1

Remote VT-1 visual examination may be substituted for direct examination. Remote examination may use aids such as telescopes, borescopes, fiber optics, cameras, or other suitable instruments provided such systems have a resolution capability at least equivalent to that attainable by direct visual examination.

11.1.2 Visual Examination VT-2

The VT-2 visual examination shall be conducted to locate evidence of leakage from pressure retaining components, or abnormal leakage from components with or without leakage collection systems as required during the conduct of system pressure or functional test.

The visual examination, VT-2; may be conducted without the removal of insulation by examining the accessible and exposed surfaces and joints of the insulation. Essentially vertical surfaces of insulation need only be examined at the lowest elevation where leakage may be detectable. Essentially horizontal surfaces of insulation shall be examined at each insulation joint.

For components whose external insulation surfaces are inaccessible for direct examination, only the examination of surrounding area, including floor areas or equipment surfaces located underneath the components, for evidence of leakage, or other areas to which such leakage may be channeled, shall be required.

Discoloration or residue on surfaces examined shall be given particular attention to detect evidence of boric acid accumulations from borated reactor coolant leakage.

The visual examination shall be conducted during the system leakage tests conducted after refueling outages and prior to startup.

VT-2 visual examination shall be conducted in accordance with ASME Section XI, IWA-5240.

11.1.3 Visual Examination VT-3

The VT-3 visual examination shall be conducted to determine the general mechanical and structural conditions of components and their supports such as the presence of loose parts, debris, or abnormal corrosion products, wear, erosion, corrosion, and the loss of integrity at bolted or welded connections.

VT-3 may require, as applicable to determine structural integrity, the measurement of clearances, detection of physical displacement, structural adequacy of supporting elements, connections between load carrying structural members, and tightness of bolting.

For component supports and component interiors, the visual examination may be performed remotely with or without optical aids to verify the structural integrity of the component.

11.1.4 Visual Examination VT-4

The VT-4 visual examination shall be conducted to determine conditions relating to the operability of components or devices such as mechanical and hydraulic snubbers, component supports, pumps, valves, and spring loaded and constant weight hangers.

VT-4 shall confirm functional adequacy, verification of the settings, or freedom of motion. This examination may require (1) disassembly of components or devices and (2) operability test.

11.2 Surface Examination

11.2.1 Magnetic Particle Examination

Magnetic particle examination (MT) shall be conducted in accordance with Article 7, Section V of the ASME Code.

11.2.2 Liquid Penetrant Examination

Liquid penetrant examination (PT) shall be conducted in accordance with Article 6, Section V of the ASME Code.

11.3 Volumetric Examination

11.3.1 Radiographic Examination

Radiographic techniques, employing penetrating radiation such as X-rays, gamma rays, or thermalized neutrons, may be utilized with appropriate image recording devices such as photographic film or papers, electrostatic systems, direct-image orthicons, or image converters. For radiographic examinations employing either X-ray equipment or radioactive isotopes and photographic films, the procedure shall be as specified in Article 2, Section V, of the ASME Code.

11.3.2 Ultrasonic Examination

- (a) Ultrasonic examination of Class 1 and Class 2 vessel welds in ferritic material greater than 2 inch (51 mm) in thickness shall be conducted in accordance with Article 4 of Section V of the ASME Code.
- (b) Ultrasonic examination of Class 1 and Class 2 ferritic steel piping systems shall be conducted in accordance with Appendix III, amended as follows:
- (1) For examination of welds, reflectors that produce a response greater than 50% of the reference level shall be recorded.
- (c) If the requirements of (a) and (b) above are not applicable, the ultrasonic examination shall be conducted in accordance with the applicable requirements of Article 5 of Section V amended as follows:
- (1) For examination on welds, reflectors that produce a response greater than 50% of the reference level shall be recorded.

- (2) For examination of welds, all reflectors which produce a response greater than 100% of the reference level shall be investigated to the extent that the operator can determine the shape, identity, and location of all such reflectors in terms of the acceptance-rejection standards of IWA-3100(b).
- (3) The size of reflectors shall be measured between points which give amplitudes equal to 100% of the reference level.

### 11.3.3 Eddy Current Examination

Eddy current examination of heat exchanger tubing shall be conducted in accordance with the provisions of Appendix IV of Section XI of the ASME Boiler and Pressure Vessel Code.

## 12.0 QUALIFICATIONS OF NONDESTRUCTIVE EXAMINATION PERSONNEL

Personnel performing nondestructive examination operations shall be qualified in accordance with IWA-2300 of ASME Section XI (DPM N75C01).

## 13.0 ACCEPTANCE CRITERIA

All acceptance standards for Class A, B, C, and D components shall be in accordance with IWA-3000, IWB-3000, IWC-3000, or IWD-3000 of ASME Section XI, except where ASME Section III examinations are employed to satisfy ASME Section XI requirements.

## 14.0 REPAIRS

All ASME Section XI components and their supports (ASME Classes 1, 2, and 3) shall be repaired in accordance with the Repair and Replacement Program included in N-OQAM Part II, Section 2.3 and implemented by DPM SQ82M1.

## 15.0 REPLACEMENTS

Replacement of ASME Section XI components (ASME Classes 1, 2, and 3) shall be in accordance with the Repair and Replacement Program in N-OQAM Part II, Section 2.3 and implemented by DPM SQ82M1.

ASME Section XI repairs and replacements may be coordinated as necessary with the Chemical, Metallurgy and Standards Group of the Mechanical Branch.

16.0 RECORDS AND REPORTS

16.1 Recording of and Report of Examinations

A detailed report of all examinations shall be prepared by the performing or responsible organization and should contain but not be limited to the following information:

Title Page

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- I. Introduction - The introduction should include the following information: Plant, unit number, preservice or inservice inspection and cycle number, systems, components and vessels examinations were performed on, organization examinations were performed by, dates examinations were performed, ASME Section XI Code of Record.
- II. Summary - The summary should include a brief description of the overall inspection: Program, performance, personnel, equipment, procedures, evaluations, and results.
- III. Discussion - The discussion should discuss the governing documents (ASME Code, Technical Specifications, etc.), inspection schedule, materials, calibration standards, calibration performance, reporting, recording, interpretation, and brief evaluation.
- IV. Evaluation - Evaluation is based on the indication's location, metal path, general shape, and any tests that could be applied, such as damping. The evaluation section also should contain a listing of each examination performed and the evaluated results.
- V. Summary of Notifications - The summary of notifications shall give a short summary of each notification report along with the indication discrepancy and its location. It should also contain the final disposition and the date of completion.

- VI. Scan Plan - The Scan Plan shall give a detailed description of all areas subject to examination during the inspection. It shall contain the following information: Examination Area, Code Category, Weld Size and/or Number, Reference Drawing, Examination Method, Procedure, Calibration Block, and any reference details pertaining to the exam area, such as the weld number, meridional welds, pump studs, etc.
- VII. Weld and Hanger Maps - The Weld and Hanger Maps are the reference drawings for the inspection. The weld maps are isometric drawings showing the location of both field and shop welds on each vessel, component, and piping system subject to examination. The hanger maps are also isometrics showing the location of hangers, snubbers and supports for each vessel, component, or piping system subject to examination.
- VIII. Log by System - The log is the daily status of the inspection section representative of the areas subject to examination during the inspection. This log keeps an up-to-date status of work complete and incomplete.
- IX. Personnel Certifications
- X. Equipment Certifications
- XI. NDE Procedures
- XII. Calibration Block Drawings
- XIII. Calibration Sheets
- XIV. Examination Reports - Reports for inservice inspection shall be prepared in accordance with IWA-6220 of ASME Section XI.

For eddy current examination of heat exchanger tubing, the report shall include a record indicating the tube(s) examined (this may be marked on a tube sheet sketch or drawing), the extent to which each tube was examined, the location and depth of each reported indication, and the identification or the operator(s) and data evaluator(s) who conducted each examination or part thereof, and magnetic tape and strip charts.



All procedures and equipment shall be identified sufficiently to permit duplication of the examination at a later date. This shall include initial calibration data for the equipment and any significant changes.

All required and pertinent information will be recorded on the appropriate data sheets by the performing organization. When portions of the inspection work are contracted, a detailed report will be submitted to TVA by the contractor with all pertinent and required information. TVA will retain the original copies of all raw data taken.

The QEB NDE Section shall review and submit the final report to the Plant Superintendent for review. These final reports shall be filed at the plant site with the data sheets of Appendix C of this program as discussed in Section 5.0 of this program. Data Sheet 1 in Appendix C will be completed and used as a cover sheet for the final report and to document the review process.

#### 16.1.1 Repair and Replacement Reports

The plant shall prepare a summary of repairs and replacements for all Class A and B components. The report shall include the applicable requirements of IWA-6220 of ASME Section XI and shall be submitted to the Chemical, Metallurgy and Standards Group within 45 days after each refueling outage. The report shall include repairs conducted during each refueling outage and all repairs conducted since the end of the preceding refueling outage. The Chemical, Metallurgy and Standards Group shall review the report to assure it contains applicable information required in IWA-6220 of ASME Section XI. After reviewing the summary report, the Chemical, Metallurgy and Standards Group shall forward it to the Quality Engineering Branch for submittal to the NRC as part of the inservice inspection report described in N-OQAM, Part II, Section 5.1 and 16.2 of this program, within 90 days from the end of each refueling outage.

#### 16.2 ISI Report for Class A and B Components

The QEB NDE Section shall prepare an ISI Summary Report for Class A and B components to be submitted within 90 days after the completion of the inservice inspection with the Nuclear Regulatory Commission Region II Office in accordance with IWA-6220, ASME Section XI.

The ISI Report shall have a cover sheet providing the following information:

- (1) Date of document completion.
- (2) Name and address of owner.
- (3) Name and address generating plant.
- (4) Name or number assigned to the nuclear power unit by TVA.
- (5) Commercial operation date for unit.

All reports shall provide the following information as a minimum.

- (a.) Numbers assigned to the components by the state.
- (b.) National Board Number assigned to the components by the manufacturer.
- (c.) Names of the components and descriptions including size, capacity, material, location, and drawings to aid identification.
- (d.) Name and address of principal manufacturer (e.g., Westinghouse, GE, etc.) and the principal contract number which will identify the subcontractors.
- (e.) Manufacturer's component identification numbers.
- (f.) Date of completion of the examination, (test, replacement, or repair).
- (g.) Name of ANII who witnessed or otherwise verified the examinations, (tests, replacements, or repairs), and his employer and business address, when required.
- (h.) Abstract of examinations, (tests, replacements, or repairs performed); conditions recorded; and corrective measures recommended or taken.
- (i.) Signature of ANII, when required.
- (j.) Owner's Data Report for inservice inspection, Form NIS-1, as shown in Appendix II of ASME Section XI.

- (k.) Completed examination reports.
- (l.) Completed calibration reports.
- (m.) List of component drawings.
- (n.) List of TVA NDE personnel and/or copies of contractor personnel certifications.
- (o.) List of TVA NDE equipment and/or copies of contractors equipment certifications.
- (p.) List of TVA NDE procedures used and/or copies if contractor NDE procedures.

The NDE Section shall submit the inservice examination report to the plant manager for retention as a quality assurance record in accordance with N-OQAM, Part III, Section 4.1. The NDE Section shall maintain a copy of the report for information, and shall submit applicable summaries of the report with a cover sheet as described above to the NRC via the Manager, Nuclear Licensing Branch, Office of Power.

16.3 Records for Classes A, B, C, and D Components

The following records shall be available for review:

- (1) Examination Plans
- (2) Examination Results and Reports
- (3) Examination Methods and Procedures
- (4) Evaluation of Results
- (5) Corrective Actions and Repairs

16.4 Records of System Pressure Tests

Records of the visual examinations conducted during system leakage or hydrostatic tests shall consist of an itemization of the number and location of leaks found in a system and the corrective actions taken.

16.5 Augmented Examination Reports

Augmented examination special reports shall be submitted to the Nuclear Regulatory Commission Region II Office within the time period specified for each report. For specific details on records, reports and reporting see Section 20.0, Augmented Inspections, of this program.

17.0 NOTIFICATION OF INDICATION

Plant management shall be formally notified of the presence of unacceptable indications detected during the performance of nondestructive examinations (excluding VT-2, visual examinations performed during system pressure tests). Unacceptable indications are defined by the applicable NDE procedure. Formal notification shall consist of completing and submitting to the Plant Superintendent the "Notification of Indication" form in Appendix D of this program.

Part I of the "Notification of Indication" shall be completed and signed by the NDE Level II or III examiner detecting the indication. The NDE Section Representative of the Quality Engineering Branch shall assign a sequential number review and sign the form. If the indication is detected by an outside contractor, the contractor's field supervisor shall review and sign the form. The original shall be sent to the plant superintendent and a copy to the ISI Programs Section.

The plant manager or his assistant shall be responsible for determining which organization (Outage Modification and Major Maintenance, Plant Maintenance) shall be responsible for preparing a disposition in Part II of the form and performing the associated corrective action. If the organization assigned responsibility for disposition is unable to determine a satisfactory disposition then the form should be sent to the Mechanical Branch for disposition. The individual responsible for preparation of the disposition shall sign and date Part II of the form. The cognizant supervisor of the appropriate organization shall review and approve the disposition and sign and date Part II of the form. Copies of the form shall be distributed to the plant manager and the ISI Programs Section. The original shall be returned to the NDE Section Representative. One copy shall be filed with the examination report.

If the organization assigned responsibility for repair is within NUC PR, they shall determine if the unacceptable condition is significant and potentially reportable in accordance with the requirements of N-OQAM, Part III, Section 7.2. Dispositions to correct the condition under the plant maintenance program shall

be processed in accordance with N-OQAM, Part II, Section 2.1. Dispositions other than restoring to original requirements shall be processed as modifications in accordance with N-OQAM, Part II, Section 3.0. Repair and replacement, activities, including coordination with the Authorized Inspection Agency (AIA), shall be performed in accordance with the requirements of N-OQAM, Part II, Section 2.3. Dispositions to accept the condition as-is shall include in Part II of the form the basis for the disposition. In addition for dispositions to accept the condition as-is, a USQD shall be prepared by the appropriate organization in accordance with established procedures. The organization responsible for corrective action shall include preservice examination requirements in the repair or replacement work instruction described in N-OQAM, Part II, Section 2.3.

Upon completion of corrective action the NDE Section Representative shall verify completion of corrective action, enter the work instruction and/or DCR numbers on the Notification of Indication form, enter the examination report number if re-examination was performed, and sign and date the form, Part III. The signed form shall remain with the examination report for use as a quality assurance record. If re-examination was performed, a copy of the signed form shall also remain with the re-examination report. Copies of the form shall also be distributed to those listed in the second paragraph above.

#### 18.0 CALIBRATION BLOCKS

Calibration blocks will be used for ultrasonic examinations (a calibration tube will be used for eddy current examination of steam generator tubing). The blocks will be fabricated to the general requirements of ASME Section V and ASME Section XI. The blocks shall be fabricated of the material to be examined or equivalent P numbers. Mill test reports shall be obtained and retained by the Quality Engineering Branch for all calibration blocks. The blocks shall employ drilled holes and/or notches for calibration reflectors (See Request for Relief ISI-2).

The NDE Section shall maintain as-built calibration block drawings. Copies of the original drawings and any revisions shall be submitted to the ISI Programs Section. The calibration blocks shall be stored at the plant site and maintained by the plant QA organization.

## 19.0 REQUESTS FOR RELIEF

Where TVA has determined that certain code requirements or examinations are impractical, TVA will submit written requests for relief to NRC with information to support the determinations and any proposed alternate examinations. The impractical code requirements or examinations shall be identified in this program, and references to particular requests for relief shall be included.

When impractical examination requirements are identified in the field, the NDE Section shall notify the ISI Programs Section such that the information may be included in this program and requests for relief may be prepared if necessary.

The Requests for Relief are listed in Appendix E.

## 20.0 AUGMENTED INSPECTIONS

Augmented inspections are performed in addition to ASME Section XI code requirements. The augmented inspections may be required by the NRC or self-imposed by TVA.

### 20.1 Feedwater System Piping and Supports

In order to satisfy the requirements of NRC-IE Bulletin 79-13, the following augmented examinations shall be performed.

The augmented examination of the feedwater nozzle-to-pipe welds (includes nozzle-to-transition piece welds and transition piece-to-pipe welds) and of adjacent pipe and nozzle areas are included in DPM No. SEQ80M7. The requirements are a radiographic examination supplemented by ultrasonic examination on completion of the hot functional testing and before fuel loading. During the first refueling outage, perform a volumetric examination of the feedwater nozzle-to-pipe welds, the feedwater piping welds to the first support, the feedwater line-to-containment penetration welds and an area of at least one pipe diameter of the main feedwater line downstream at the auxiliary feedwater to main feedwater connection. Also perform a visual inspection of all feedwater system piping supports and snubbers in containment to verify operability and conformance to design.

In the event cracking is identified during examination of the nozzle-to-pipe welds, all feedwater lines up to the first piping support or snubber outboard of the nozzle shall be volumetrically examined as stated in the first paragraph. All unacceptable code discontinuities shall be subject to repair unless justification for continued operation is provided.

Any cracking or other unacceptable code discontinuities identified shall be reported to the Director of the NRC Regional Office within 24 hours of identification. Plant management shall report this information to the Nuclear Regulatory Commission Region II Office per Plant Instruction AI-18. A written report of the results of examination shall be submitted by the QEB NDE Section within 30 days of completion of the examination. Refer to DPM No. SEQ80M7 for information to be included in this special report.

The requirements of NRC-IE Bulletin 79-13 were satisfied during the unit 1, cycle 1 outage. This information has been retained as historical information and is no longer applicable to Sequoyah Nuclear Plant, unit 1.

20.2 RPV Nozzle Safe Ends

The augmented examination requirements of the RPV nozzle-to-safe end welds are included in the final report - Sequoyah Nuclear Plant - Evaluation of cracking in reactor vessel nozzle stainless steel buttering. The examinations for unit 1 will be monitored at the normal inservice inspection intervals for dissimilar metal welds as required by Section XI of the ASME Code. This augmented examination does not require a special report. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

20.3 Reactor Coolant Pump Flywheel

The augmented examination requirements of the reactor coolant pump flywheel are included in Regulatory Position C.4.b of Regulatory Guide 1.14; (1) an inplace ultrasonic examination of the areas of higher stress concentration at the bore and keyway at approximately three-year intervals during the refueling or maintenance shutdown coinciding with the inservice inspection schedule as required by Section XI of the ASME Code, and (2) a surface examination of all exposed surfaces and complete ultrasonic examination at approximately 10-year intervals during the plant shutdown coinciding with the inservice inspection schedule as required by Section XI of the ASME Code.

This augmented examination does not require a special report unless the examination reveals a flaw. If the examination and evaluation indicate an increase in flaw size or growth rate greater than predicted for the service life of the flywheel, the results of the examination and evaluation should be submitted to the NRC for evaluation. Refer to Regulatory Guide 1.14 for information to be included. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

20.4 Pressurizer Relief Line

The augmented examination requirements of the pressurizer relief line are included in the Technical Specifications 4.0.5 and 4.4.3.2.4. The pressurizer relief line and repair welds shall be examined using improved ultrasonic detection and evaluation procedures prior to entering Mode 4 whenever the plant has been in cold shutdown for 72 hours or more if the examination has not been performed in the previous six months.

In the event these six-month period examinations find the piping free of unacceptable indications for three successive inspections, the inspection interval shall be extended to 36 month intervals ( $\pm 12$  months to coincide with a scheduled refueling outage). The report shall be submitted with the Final Inservice Inspection Report.

20.5 Steam Generator Tubing

The augmented examination requirements of the steam generator tubing are included in Technical Specifications 4.0.5, 4.4.5.0 and section 7.3.8 of this program. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the NRC within 15 days.

Plant management shall report this information to the Nuclear Regulatory Commission Region II Office within the time period specified. See Plant Instruction AI-18 for reporting instructions.

The complete results of the steam generator tube inservice inspection shall be submitted to the Nuclear Regulatory Commission in a special report pursuant to Technical Specification 6.9.2 within 12 months following completion of the inspection. The Chemical, Metallurgy, and Standards



Group Staff Specialist shall prepare this special report and submit the report of the Nuclear Regulatory Commission Region II Office within the stated time period. This special report shall include:

1. Number and extent of tubes inspected.
2. Location and percent of wall-thickness penetration for each indication of an imperfection.
3. Identification of tubes plugged.

Results of steam generator tube inspections which fall into Technical Specification Category C-3 require prompt notification of the Nuclear Regulatory Commission pursuant to Technical Specification 6.9.1 prior to resumption of plant operation.

The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures to prevent recurrence.

Refer to Technical Specifications 4.0.5 and 4.4.5.0 and section 7.3.8 of this program for information to be included.

#### 20.6 RPV Nozzle Cladding

All vessel nozzles cladding shall be ultrasonically examined at the end of each 10-year inspection interval, using techniques at least as sensitive as those used to conduct the supplemental examinations performed prior to fuel loading. The results of this examination shall be reported to NRC. This examination is done in accordance with Sequoyah technical specifications and satisfies Surveillance Requirement 4.4.10. This augmented examination does not require a special report. This examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

Nozzle forging material and cladding is identified in section 7.1.2 of this program.

#### 20.7 RPV Closure Head Circumferential Weld (W09-10)

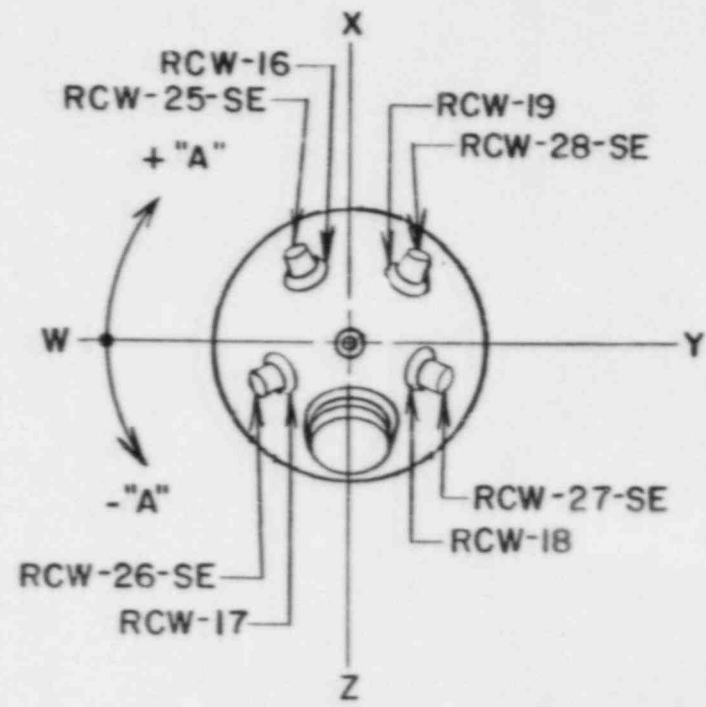
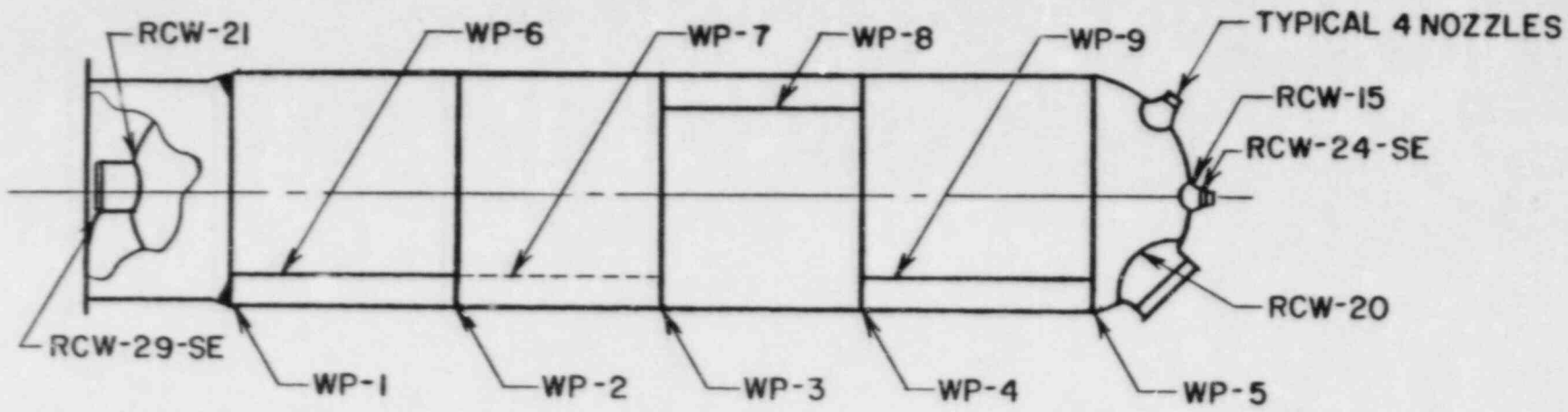
See subsection 7.1.1.3 of this program for augmented examination requirements. This augmented examination does not require a special report. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

21.0 REFERENCES

- 21.1 ASME Boiler and Pressure Vessel Code - Section XI 1974 Edition Summer 1975 addenda, Summer 1976 addenda; 1977 Edition, Summer 1978 addenda.
- 21.2 ASME Boiler and Pressure Vessel Code - Section V.
- 21.3 Instruction Manual - 173-inch I.D. Reactor Pressure Vessel - Rotterdam Dockyard Company, Contract No. 68C60-91934, N2M-2-3.
- 21.4 Sequoyah Nuclear Plant Standard Practice SQA41.
- 21.5 Sequoyah Nuclear Plant Final Safety Analysis Report.
- 21.6 Westinghouse Technical Manual - Pressurizer, TM 1440-C225, Contract No. 68C60-91934, N2M-2-6.
- 21.7 Westinghouse Technical Manual - Vertical Steam Generators, TM 1440-C224, Contract No. 68C31-91934, N2M-2-4.
- 21.8 Westinghouse Instruction Manual - Auxiliary Heat Exchangers, Contract No. 68C60-91934, N2M-2-25.
- 21.9 Westinghouse Instruction Book - Reactor Coolant Pump, Contract No. 68C60-91934, N2M-2-5.
- 21.10 Ingersoll-Rand Instruction Manual - Residual Heat Removal Pumps, Contract No. 68C60-91934, N2M-2-30.
- 21.11 Sequoyah Nuclear Plant Operational Quality Assurance Manual, Part II, Sections 2.3, 3.0, 5.1, and 6.3.
- 21.12 Sequoyah Nuclear Plant Technical Instruction TI-51.
- 21.13 Sequoyah Nuclear Plant Technical Specifications Unit 1.
- 21.14 Division Procedure Manual: N75C01, N76A10, N80E3, N83A3, SEQ80M7, SEQ82E1, and SQ82M1.

APPENDIX A  
TABLE B (Continued)  
SEQUOYAH INSERVICE INSPECTION PROGRAM  
CLASS B COMPONENTS

Component	Program Reference Section	Exam Method	Section XI Exam Category	40 Yr. Sample No. Length Welds/of Weld	1st Insp. Interval			Reference Dwg. No.		
					No. Welds	Length	Inspection Periods 3 yrs. 7 yrs. 10 yrs.			
D. Piping (cont'd)										
F. Upperhead Injection System										
a. UHI Circs >1/2" Nom. Wall Thickness										
	8.9.4.6	UT,PT	C-F,C-G	26	7	2	2	3	CHM-2337-C	
E. Pumps										
1. Residual Heat Removal (2) RHRP										
a. RHRP Component										
	Supports	8.9.1.2	VT-3	C-C,C-E	2	2	1	0	1	ISI-0099-A
*2. CVCS Centrifugal Charging Pumps (2) CCP										
a. CCP, Integrally Welded Supports										
		8.10.2.1	ST	C-C	2	2 Pmps/4 Ft	1	0	1	ISI-0146-A
	b. CCP Component Supports		VT-3	C-C, C-E	2	2	1	0	1	ISI-0146-A
	c. CCP Casing Welds		ST	C-G	2	2	1	0	1	ISI-0148-B
*3. SIS Safety Injection Pumps (2) SIP										
	a. SIP Component Supports		VT-3	C-C, C-E	2	2	1	0	1	ISI-0147-B
F. Exempted Components										
	8.12	VT-2	C-H	See Program Section 8.12						



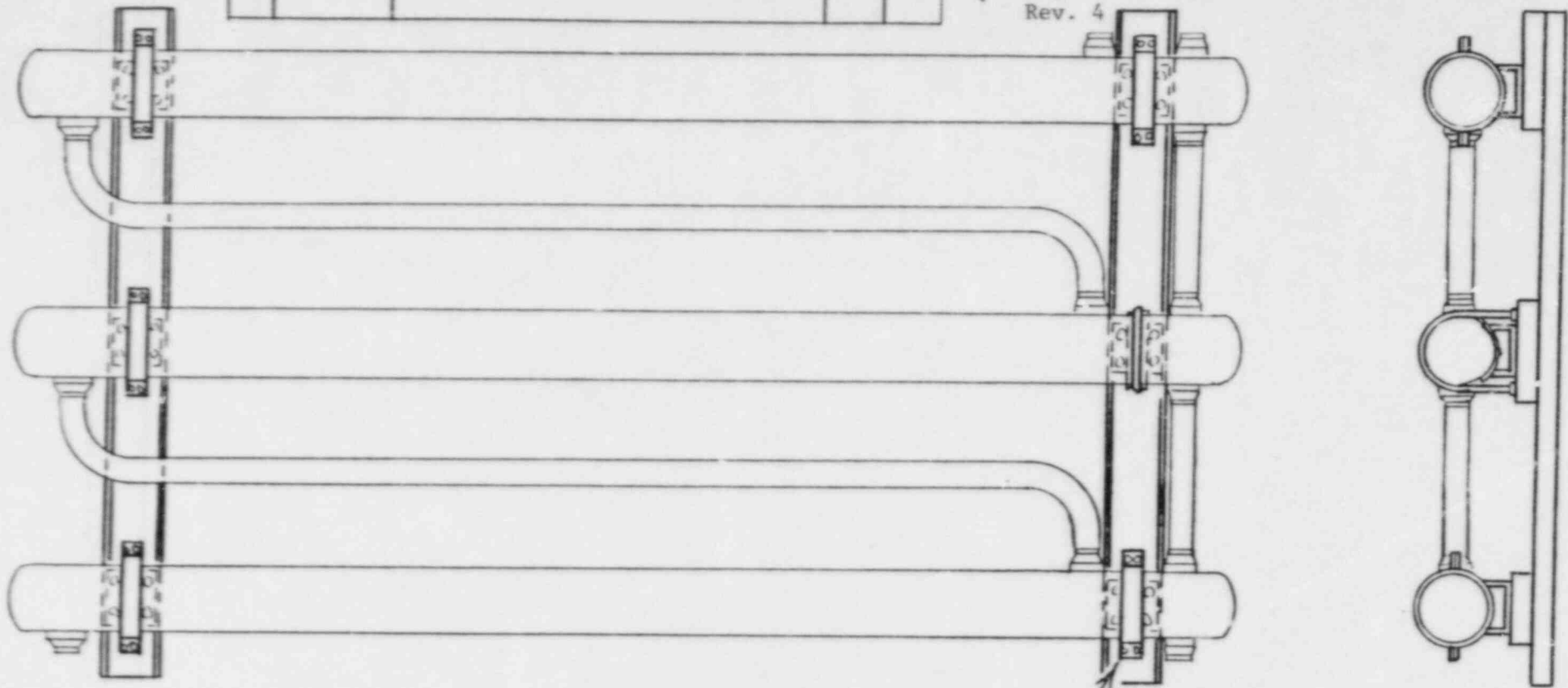
AS BUILT DIMENSIONS	
WELD NO.	"A"
WP-6	-60°
WP-9	-45°

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 SI-114.1  
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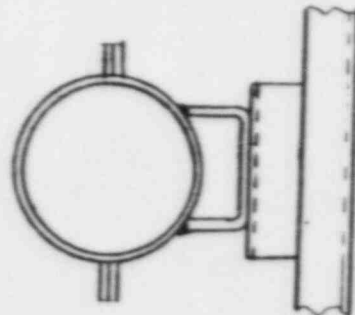
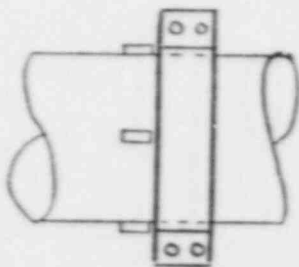
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1	11-8-83	ADD DIMENSION BLOCK KEY	EDC
NO.	DATE	REVISIONS	CK'D/A.P.P.
<b>TENNESSEE VALLEY AUTHORITY</b> DIVISION OF NUCLEAR POWER			
<b>SEQUOYAH NUCLEAR PLANT</b> UNITS # 18 # 2 PRESSURIZER			
SCALE:	SUBMITTED	APPROVED	DATE 3-5-84
DRAWN	EDC	JAB	SHEET 1 OF SHEETS
TRACED			
CHECKED			
CH-M-2363-A			

NO.	DATE	REVISIONS	CK'D	APP.
1	2-24-84	CORRECTED DWG. SIZE JAA	EDC	JAA

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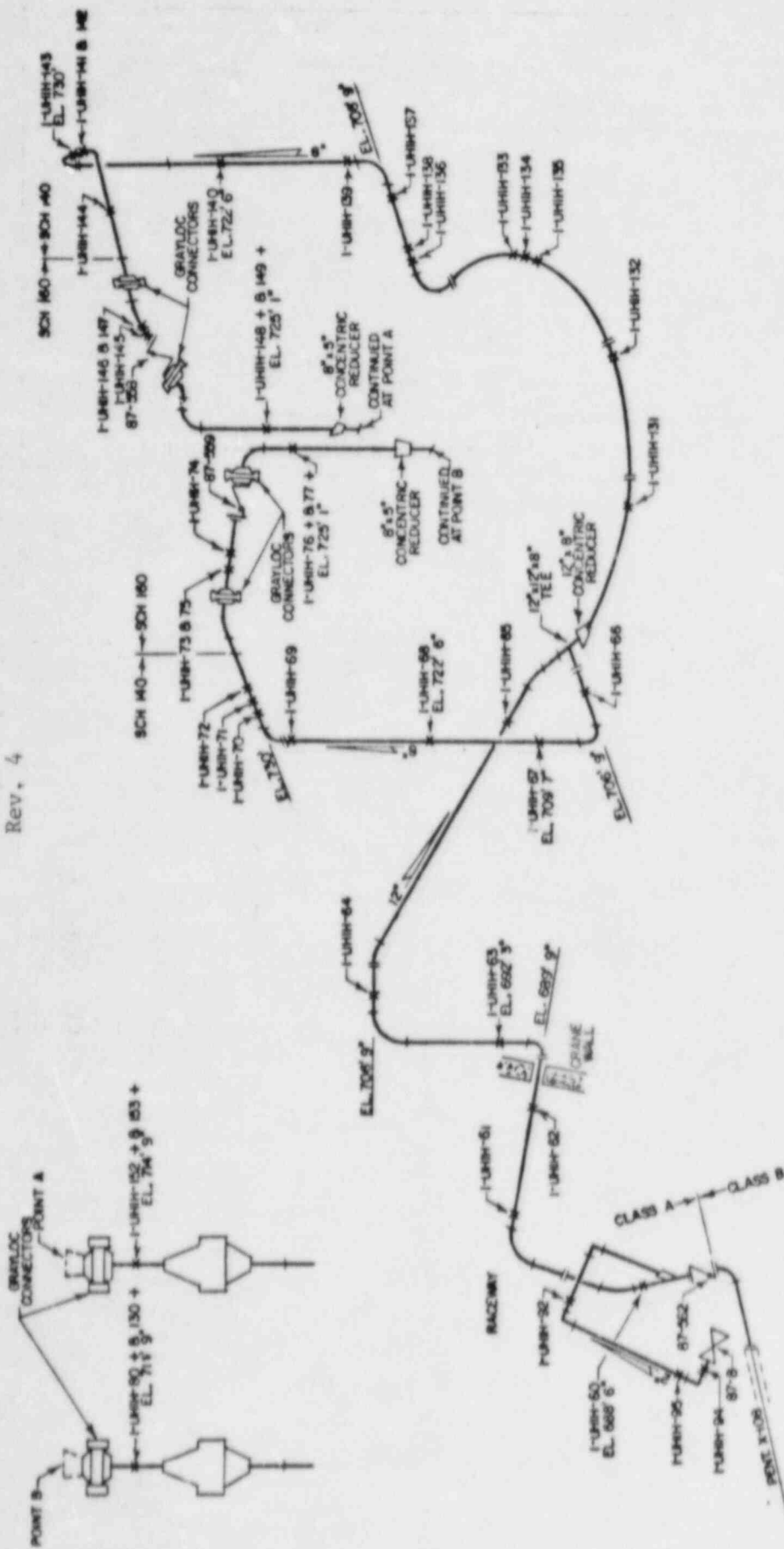
RHXH-1  
 (SEE DETAIL)



DETAIL

TENNESSEE VALLEY AUTHORITY			
DIVISION OF NUCLEAR POWER			
SEQUOYAH NUCLEAR PLANT			
UNITS #1 & #2			
REGENERATIVE HEAT EXCHANGER			
HANGER LOCATIONS			
SCALE: NTS	SUBMITTED	APPROVED	DATE 7-2-82
DRAWN	EDC	JAA	SHEET 3 OF 3 SHEETS
TRACED KEY			
CHECKED EDC			ISI-0066-A R-1

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SI-114,1  
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**SPECIFICATIONS:**  
 CLASS A  
 A375 TP316 STNL. STL. SEAML. PIPE  
 12" SCH 140  
 8" SCH 160  
 6" SCH 140  
 A403 WP316 STNL. STL. SEAML. W/FTOS.  
 12" SCH 140  
 8" SCH 160  
 6" SCH 140  
 ADLUMPY HEAD ADAPTER  
 UPPER SECTION ASME SA-192  
 LOWER SECTION ASME SB-166 (INCONEL)  
 UPPER SECTION ASME SA-192  
 LOWER SECTION ASME SB-166 (INCONEL)  
 A375 TP304 SEAMLESS STNL. STL. PIPE  
 2" SCH 160  
 A182 F304 6000" S.W. STNL. STL. FTOS.  
 2" SCH 160

REF DRAWING:  
 47-1443  
 NACCO A-7277  
 FUR-500-14

REVISIONS

NO.	DATE	CORRECTED NO. OF SHEETS	BY	CHKD
1	2 24 84	1	BLAKE	OK (DUPP)

REVISIONS

NO. 325  
 DATE 12 1 83  
 DRAWN BY J.C.G.

TEKNOLOGIJA

OPM-2437-C R1

UPPERHEAD INJECTION

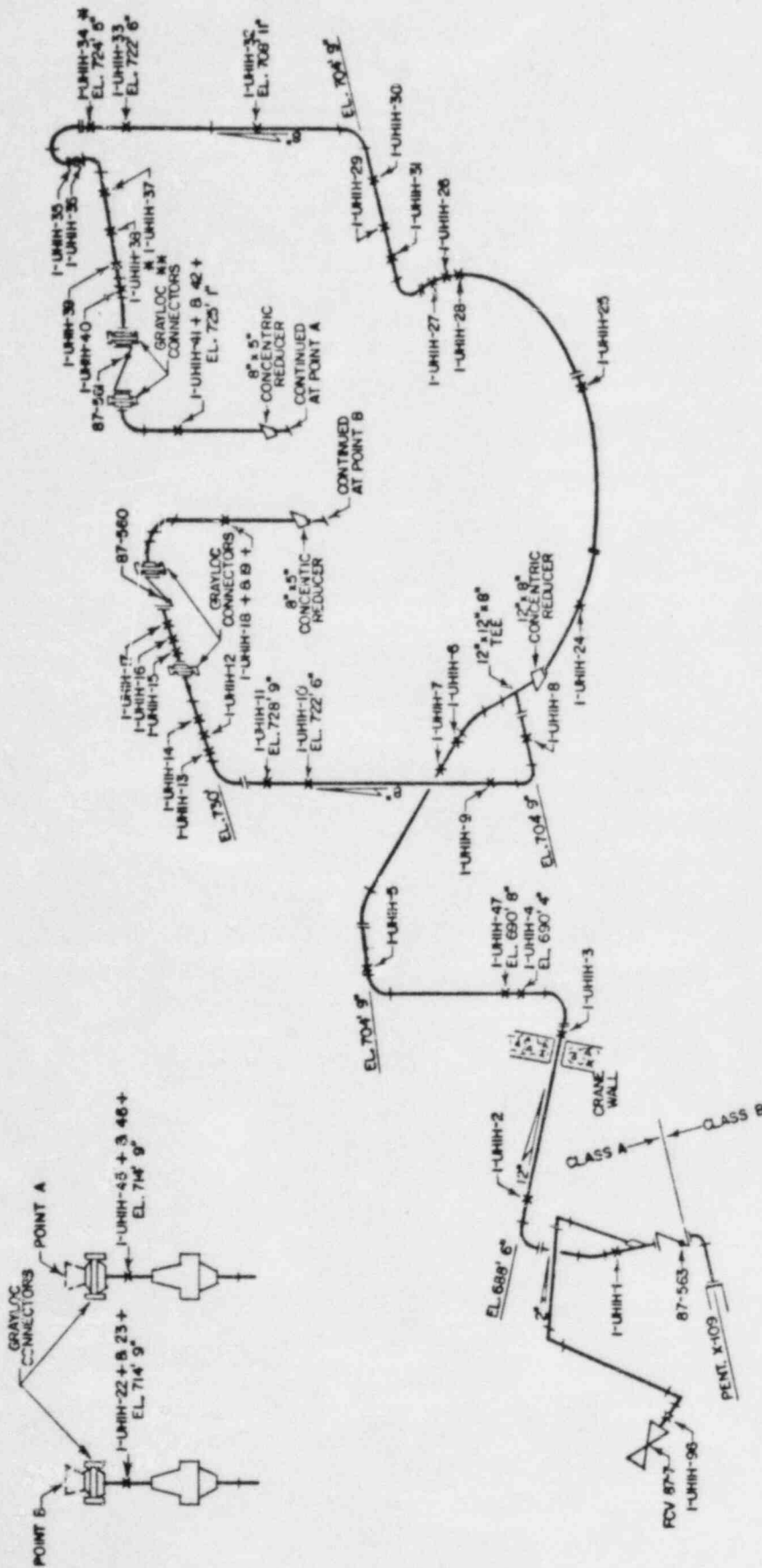
UNIT # 1

SCOVIAH NUCLEAR PLANT

DIVISION OF NUCLEAR POWER

MISSOURI VALLEY AUTHORITY

SQNL  
 SI-114.1  
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 Rev. 4



**SPECIFICATION-S:**

- CLASS A**  
 A376 TP316 STNL. STL. SEAML. PIPE  
 12" SCH 140  
 8" SCH 160  
 8" SCH 140  
 A403 WP316 STNL. STL. SEAML. W/FTGS.  
 12" SCH 140  
 8" SCH 160  
 8" SCH 140  
 AUXILIARY HEAD ADAPTER  
 UPPER SECTION ASME SA-182  
 LOWER SECTION ASME SA-166 (INCONEL)  
 6" APPROXIMATELY 1" THICK WALL  
 A376 TP304 SEAMLESS STNL. STL. PIPE  
 2" SCH 160  
 AIG2 F304 6000" S.W. STNL. STL. FTGS.  
 2" SCH 160

**REF DRAWINGS:**  
 47W435  
 NAVOO A-7278  
 I-UP1-500-1W

- + ATTACHED TO GUARD PIPE
- \* WELDED ATTACHMENTS
- \*\* REQUIRES HANGER REMOVAL FOR LUG EXAMINATION

UNIT # 1  
 UPPERHEAD INJECTION

SCQUOYAH NUCLEAR PLANT

REVISIONS

NO.	DATE	CORRECTED NO. OF SHEETS	BY	CHK'D BY
1	2.24.84			

DATE: 5/15/80  
 SHEET NO: 2 OF 3  
 DRAWING NO: CH-M-2437-C

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**MATERIAL SPECIFICATIONS**

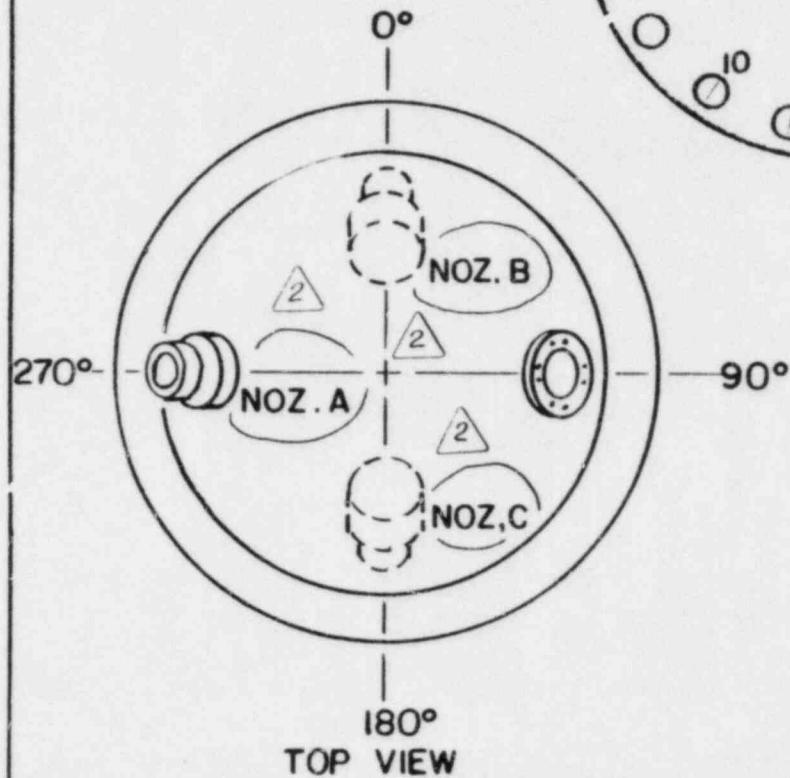
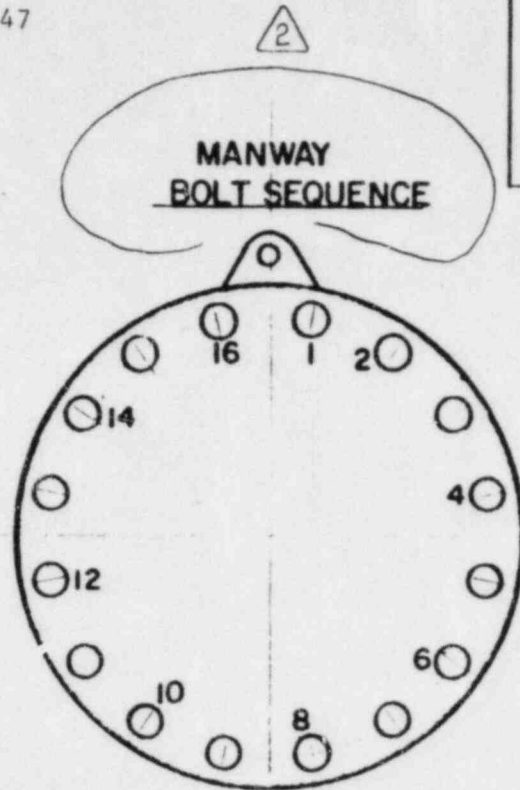
**STUDS**

SA 193 B 7

2 1/2" DIA. x 11 3/4" LG

**NUTS**

SA 194 CL-2H



NO.	DATE	REVISIONS	CK'D APP.
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1	11-9-83	ADD TOP VIEW KEY	<i>EDC</i>

TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
 UNITS #1 & #2  
 UPPER HEAD INJECTION  
 WATER ACCUMULATOR

SCALE: NTS	SUBMITTED	APPROVED	DATE 7-30-82
DRAWN KEV	<i>EDC</i>	<i>HJB</i>	SHEET 2 OF 2 SHEETS
TRACED			ISI-0070-A
CHECKED <i>EDC</i>			

R2

-190-

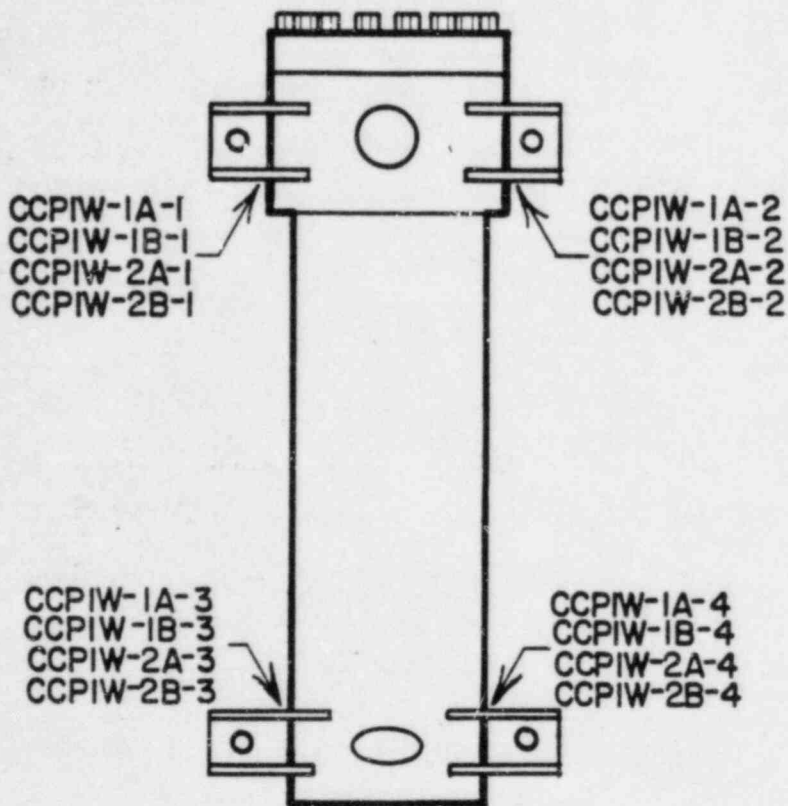


REFERENCE DRAWINGS

FC-45619

NOTES

1. THE PUMP FEET ARE BOLTED TO A COMMON SUPPORT.
2. PUMP SUPPORT NUMBERS  
 CCPH-1A-A  
 CCPH-1B-B  
 CCPH-2A-A  
 CCPH-2B-B
3. THE PUMP FEET ARE INTERGALLY WELDED TO THE CASING.



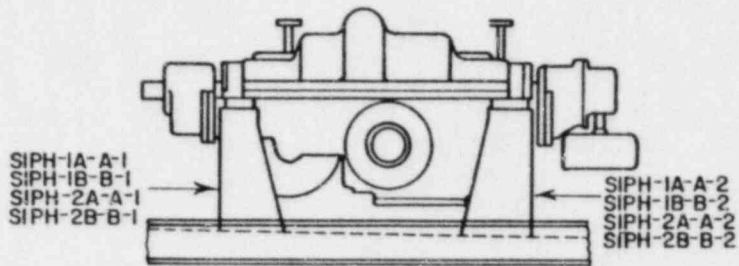
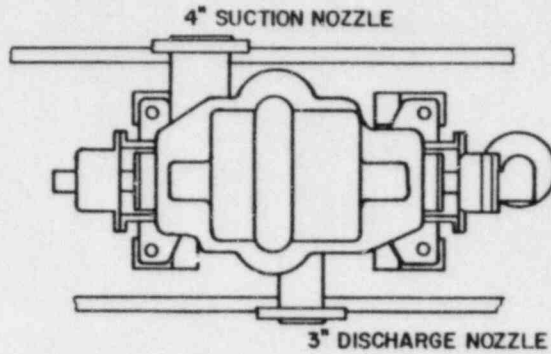
TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
 UNITS #1 & #2  
 CENTRIFUGAL CHARGING PUMP SUPPORTS

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DRAWN REV	EDC	HJB	SHEET OF SHEETS
TRACED			ISI-0146-A
CHECKED EDC			

REFERENCE DRAWINGS  
 FC-45648-4  
 TVA SAFETY CLASS B

SQNP  
 SI-114.1  
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TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOYAH NUCLEAR PLANT  
 UNITS #1 & #2  
 SAFETY INJECTION PUMP SUPPORTS

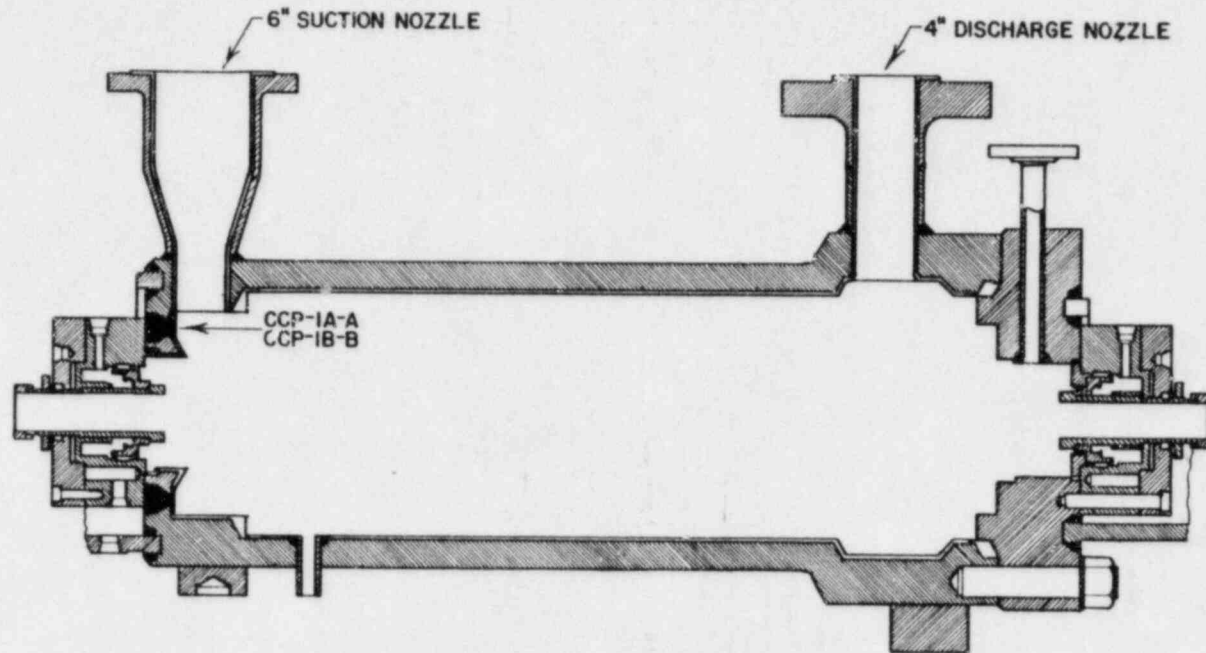
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DESIGN: <i>[Signature]</i>			
DRAWN: <i>[Signature]</i>			
CHECKED: <i>[Signature]</i>			
			ISSUE OF SHEET: ISI-0147-B

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REFERENCE DRAWINGS  
J-286-N  
TVA SAFETY CLASS B



TENNESSEE VALLEY AUTHORITY  
DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
UNIT #1  
CENTRIFUGAL CHARGING PUMP  
CASE WELDS

SCALE	DATE	REVISION	APPROVED	DATE
AS SHOWN	3-29-84	200	ASL	3-29-84
ISSUED				
REVISED				
CHECKED				
DESIGNED				
DRAWN				
BY				
FOR				
PROJECT				
NO.				
SHEET				
OF				
TOTAL				
DATE				
BY				
FOR				
PROJECT				
NO.				
SHEET				
OF				
TOTAL				

ISI-0148-B

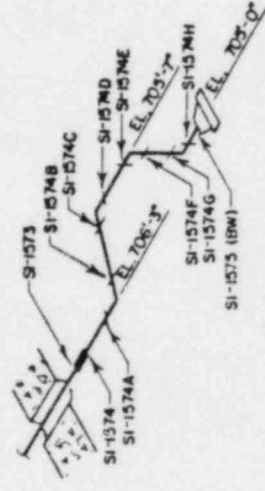
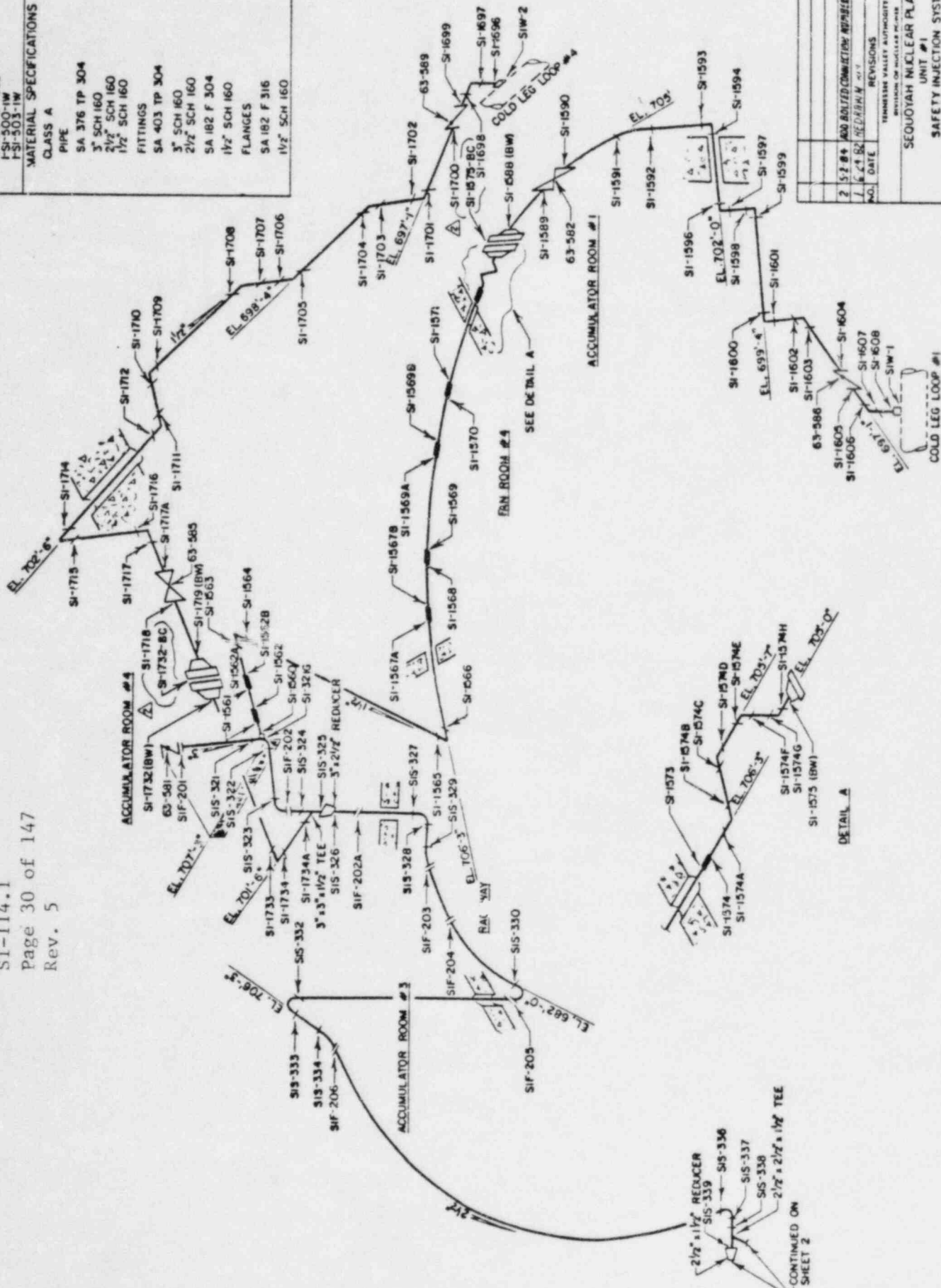
RO

-191X1-

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REFERENCE DRAWINGS  
 MAXCO A-7258  
 HSI-300-1W  
 HSI-303-1W

MATERIAL SPECIFICATIONS  
 CLASS A  
 PIPE  
 SA 376 TP 304  
 3" SCH 160  
 2 1/2" SCH 160  
 1 1/2" SCH 160  
 FITTINGS  
 SA 403 TP 304  
 3" SCH 160  
 2 1/2" SCH 160  
 SA 182 F 304  
 1 1/2" SCH 160  
 FLANGES  
 SA 182 F 316  
 1 1/2" SCH 160



CONTINUED ON  
 SHEET 2

NO. DATE		REVISIONS	
2	5-2-84	ADD BOLTED CONNECTION REPAIRS (SI-1600)	DB
1	6-21-82	REDESIGN	DB
DRAWN BY		CHECKED BY	
SCALE		DATE	
TENNESSEE VALLEY AUTHORITY DIVISION OF NUCLEAR POWER SECOYAH NUCLEAR PLANT UNIT #1 SAFETY INJECTION SYSTEM WELD LOCATIONS			
APP. 875	DATE	BY	CHKD
		DB	DB
PROJECT		SHEET	
SI-114.1		OHM-2333-C	
SHEET		OF	
30		147	









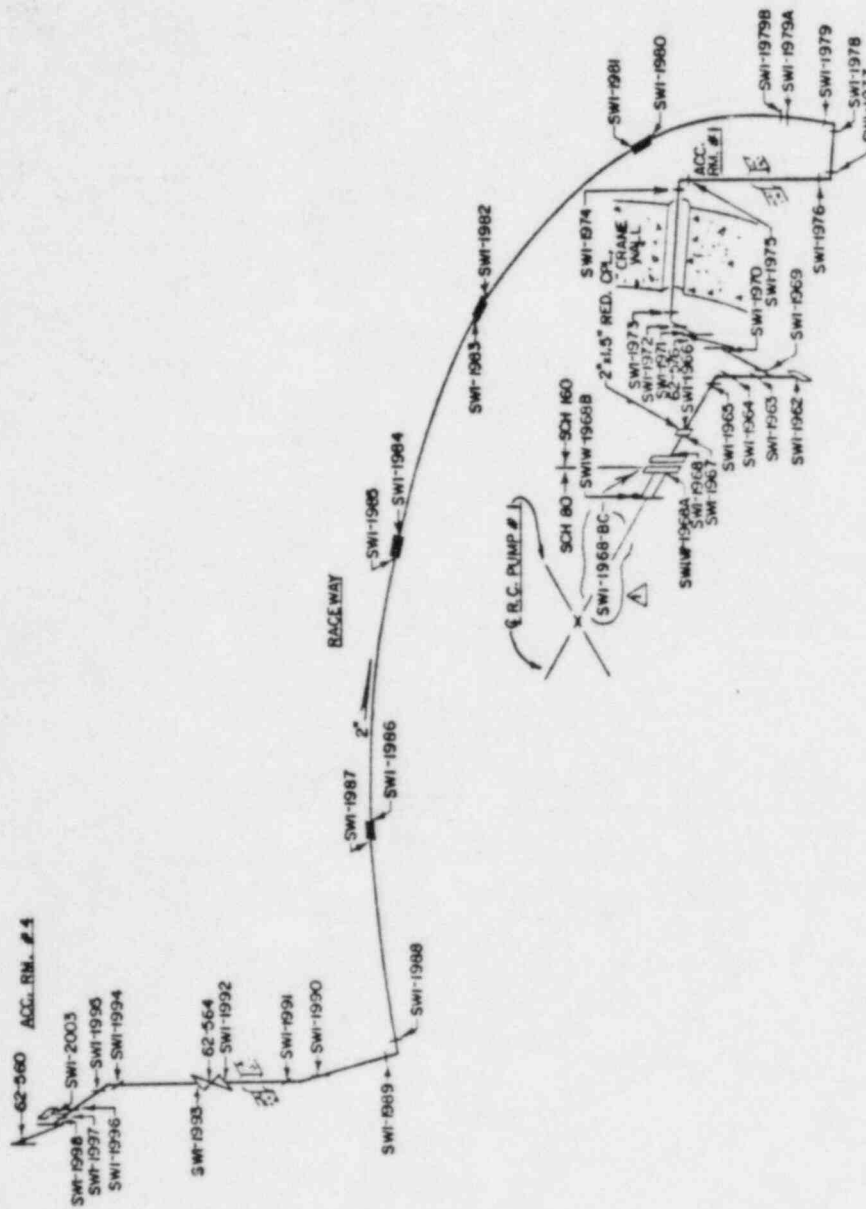








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NOTES  
 CLASS A  
 3/76 TP 304 SEAMLESS S/S PIPE  
 2\"/>

REF. DWG.  
 I-CYC-500-IN R5

NO.	DATE	BY	REVISED	DESCRIPTION
1	5/2/84	ADD	BOXED CONNECTION NUMBERS	SEE APP. CK DIAG.

TENNESSEE VALLEY AUTHORITY DIVISION OF POWER PRODUCTION	
SEQUOYAH NUCLEAR PLANT	
SEAL WATER INJECTION CHEMICAL AND VOLUME CONTROL SYSTEM	
SCALE	AS SHOWN
DATE	1-8-80
BY	WAB
CHKD	WAB
APP'D	WAB
U.S. NUC. REG. NO.	CH-M-2358-C







SONP

SI-114.1

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REFERENCE DRAWINGS

47K465-61  
47K465-62

LEGEND

● RIGID SUPPORT

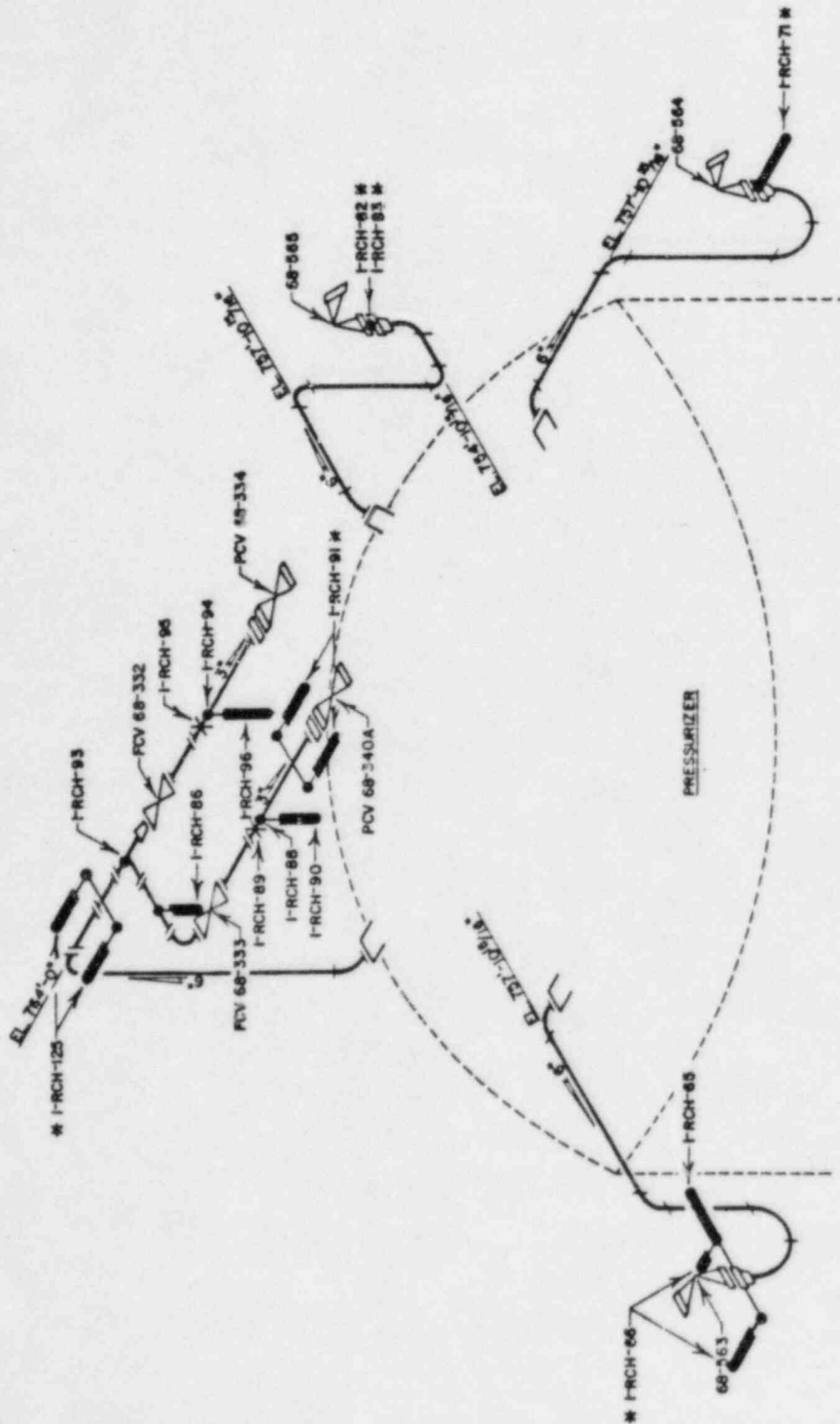
○ SKUBBER

▬ RIGID STRUTT

⊗ CONSTANT SUPPORT

⊕ WELDED ATTACHMENTS

\* TVA SAFETY CLASS A



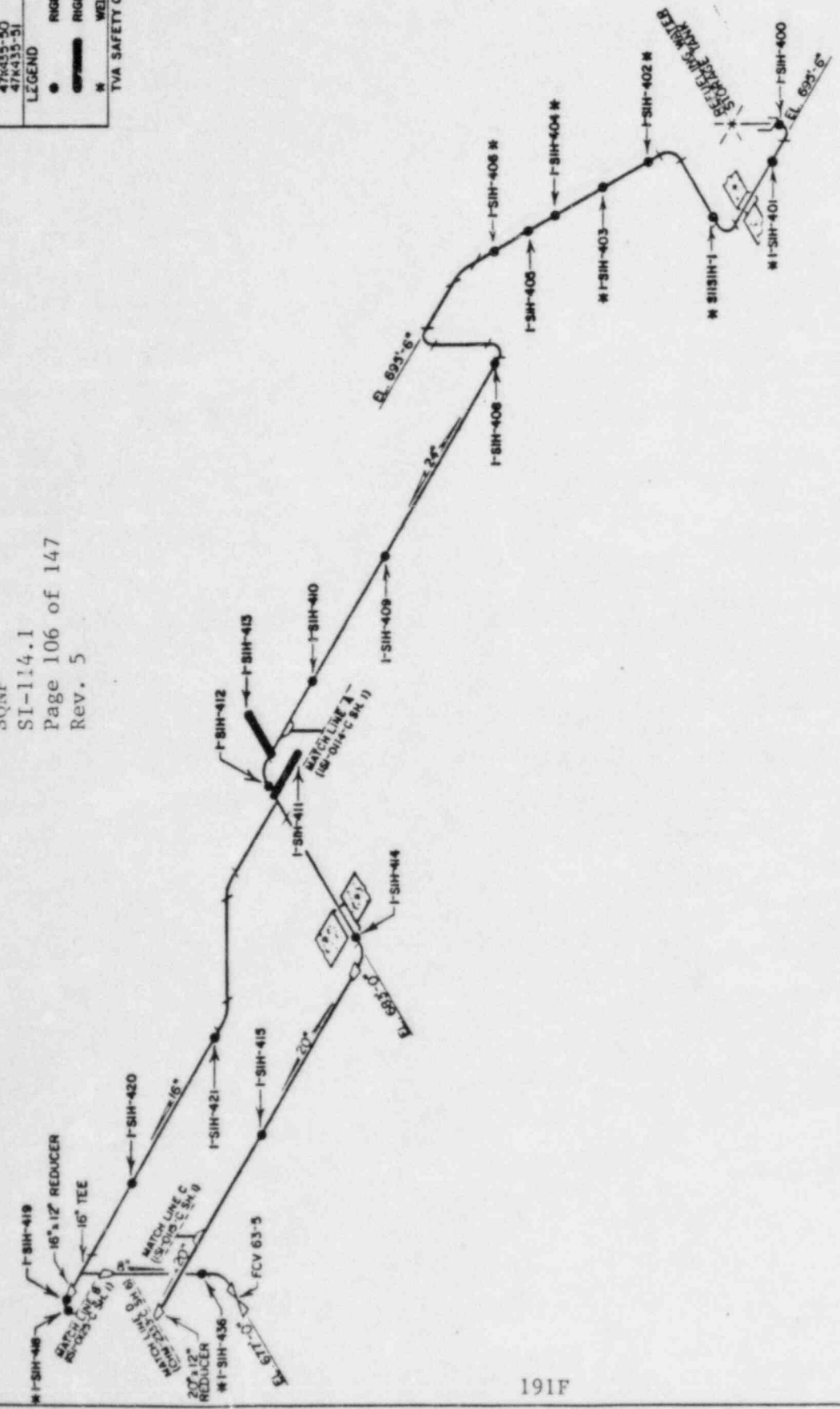
NO.	DATE	BY	DESCRIPTION
1	5-7-84	RODWINN (GENERAL REVISION)	REVISED
2	4-5-83	DELTE PAPER (DIRECT NUMBER)	REVISED
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REFERENCE DRAWINGS  
 47K435-90  
 47K435-91

LEGEND  
 ● RIGID SUPPORT  
 ○ RIGID STRUTTY  
 \* WELDED ATTACHMENTS  
 TVA SAFETY CLASS C

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 Rev. 5



191F

NO.	DATE	REVISIONS	CK'D APP.
1	5-2-84	ADD DELETE (CORRECT SUPPORT NOS AND ADD SUPPORT SYMBOLS)	52/228

TENSAR VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOYAH NUCLEAR PLANT  
 UNIT #1  
 SAFETY INJECTION SYSTEM  
 SUPPORT LOCATIONS

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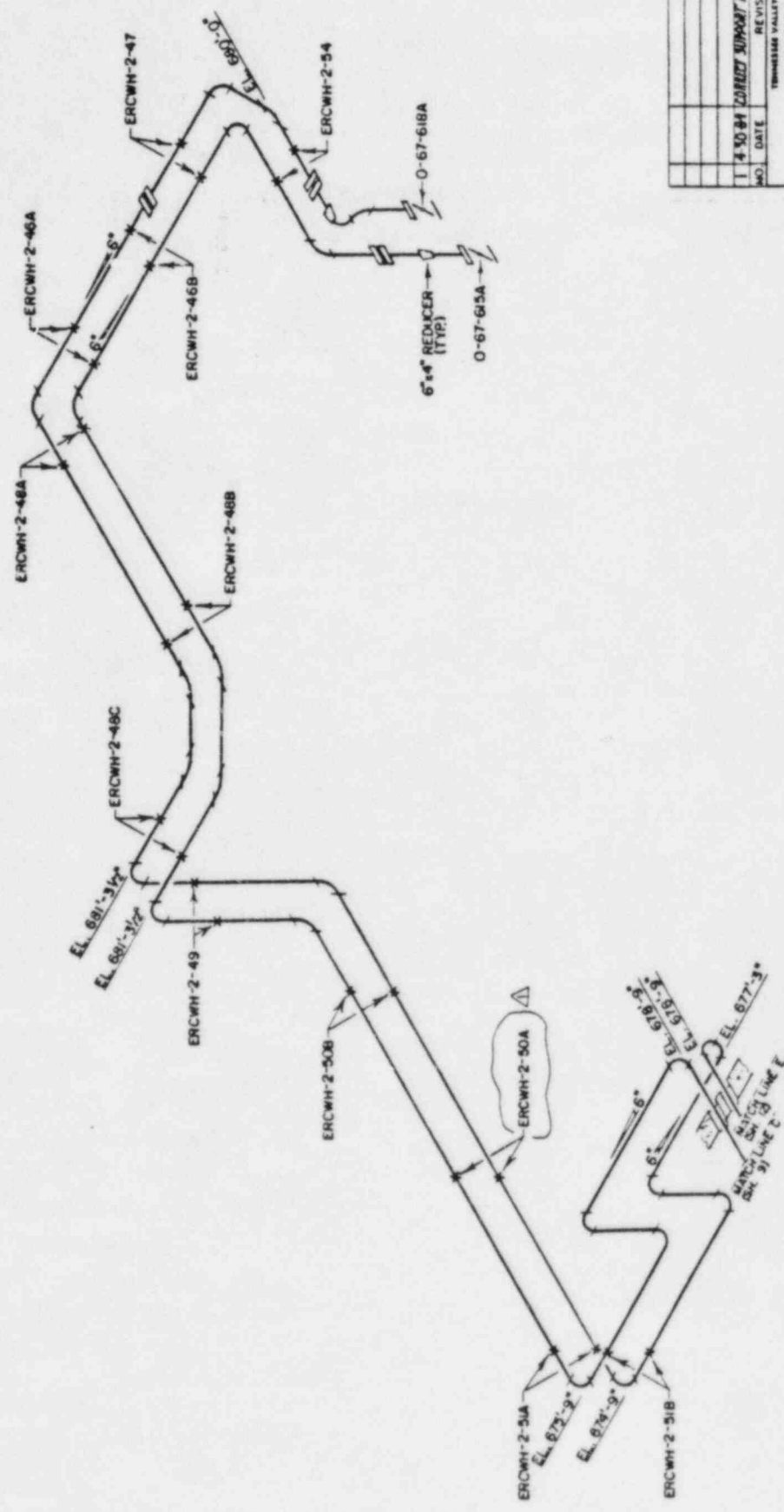
SI-0116-C





REFERENCE DRAWINGS  
47N450 SERIES  
TVA SAFETY CLASS C

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TIMBERLAKE VALLEY AUTHORITY		REVISIONS	
DIVISION OF NUCLEAR POWER		NO. DATE	
1	4	30	84
CORRECT SUPPORT IN PLACE		25/1/85	
BY		J.S.	
CHECKED BY		J.S.	
DESIGNED BY		J.S.	
DRAWN BY		J.S.	
PROJECT NO.		15-0123-C	
SHEET NO.		11	

REFERENCE DRAWINGS

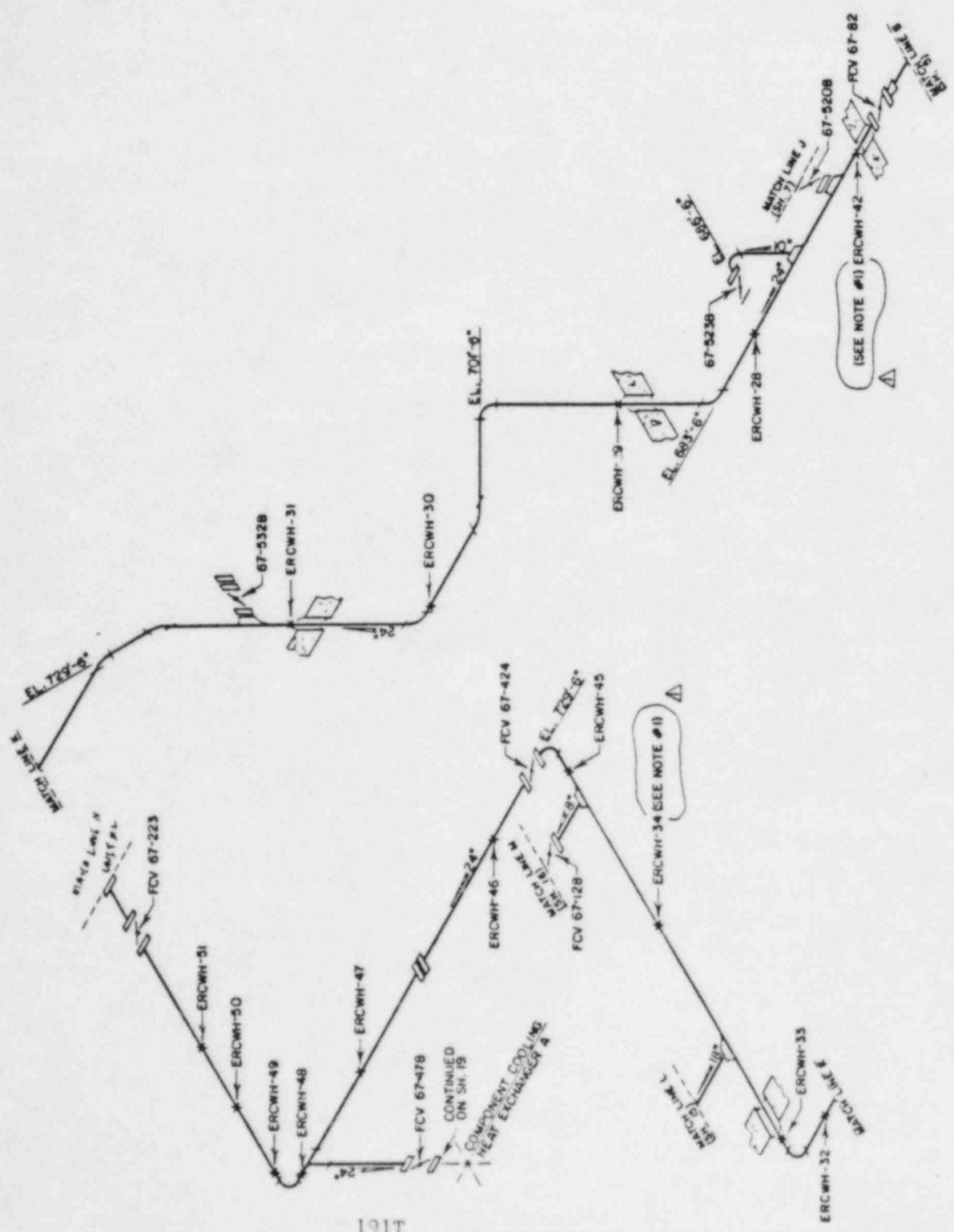
47K450-50  
47K450-51

NOTES

1. THESE SUPPORTS ARE COMMON TO THE PIPING ON SHEET 6.  
TVA SAFETY CLASS C



SONP  
SI-114.1  
Page 120 of 147  
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191T

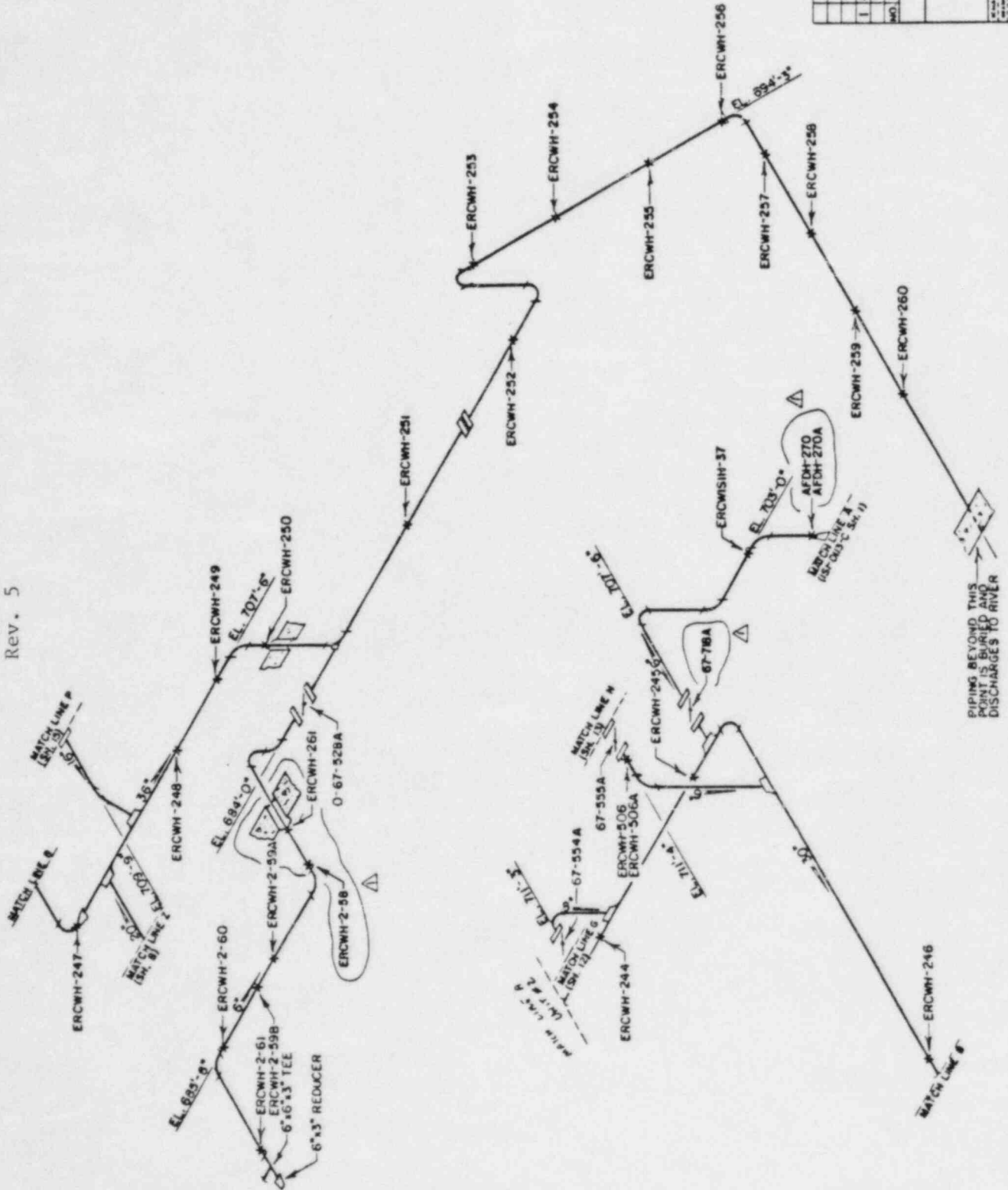
NO.	DATE	REVISIONS	BY	APP.
1	4-30-84	ADD NOTES #1	WJ	WJ

DIVISION OF NUCLEAR POWER  
 SEQUOIA NUCLEAR PLANT  
 UNIT #1  
 ESSENTIAL RAW COOLING WATER SYSTEM  
 SUPPORT LOCATIONS  
 SHEET 120 OF 147  
 T-0123-C  
 R1



REFERENCE DRAWINGS  
 47K 450-67  
 47K 450-69  
 47W 450 SERIES  
 TVA SAFETY CLASS C

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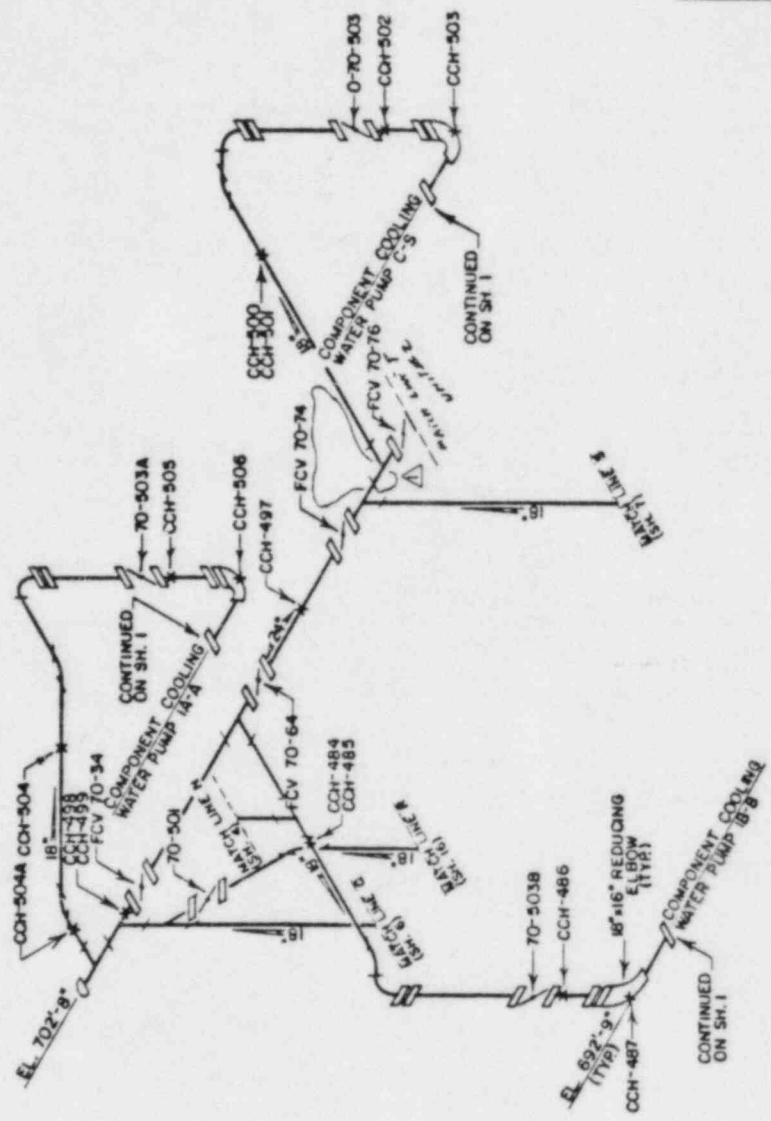


NO.	DATE	REVISIONS	BY	CHK'D
1	4 30 84	ADD SUPPORTS MOVED WALL AND CONNECTED WITH NUMBER 87	...	...

TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOYAH NUCLEAR PLANT  
 UNIT #1  
 ESSETIAL RAW COOLING WATER SYSTEM  
 SUPPORT LOCATIONS  
 SI-0123-C

REFERENCE DRAWINGS  
47K464-63  
TVA SAFETY CLASS C

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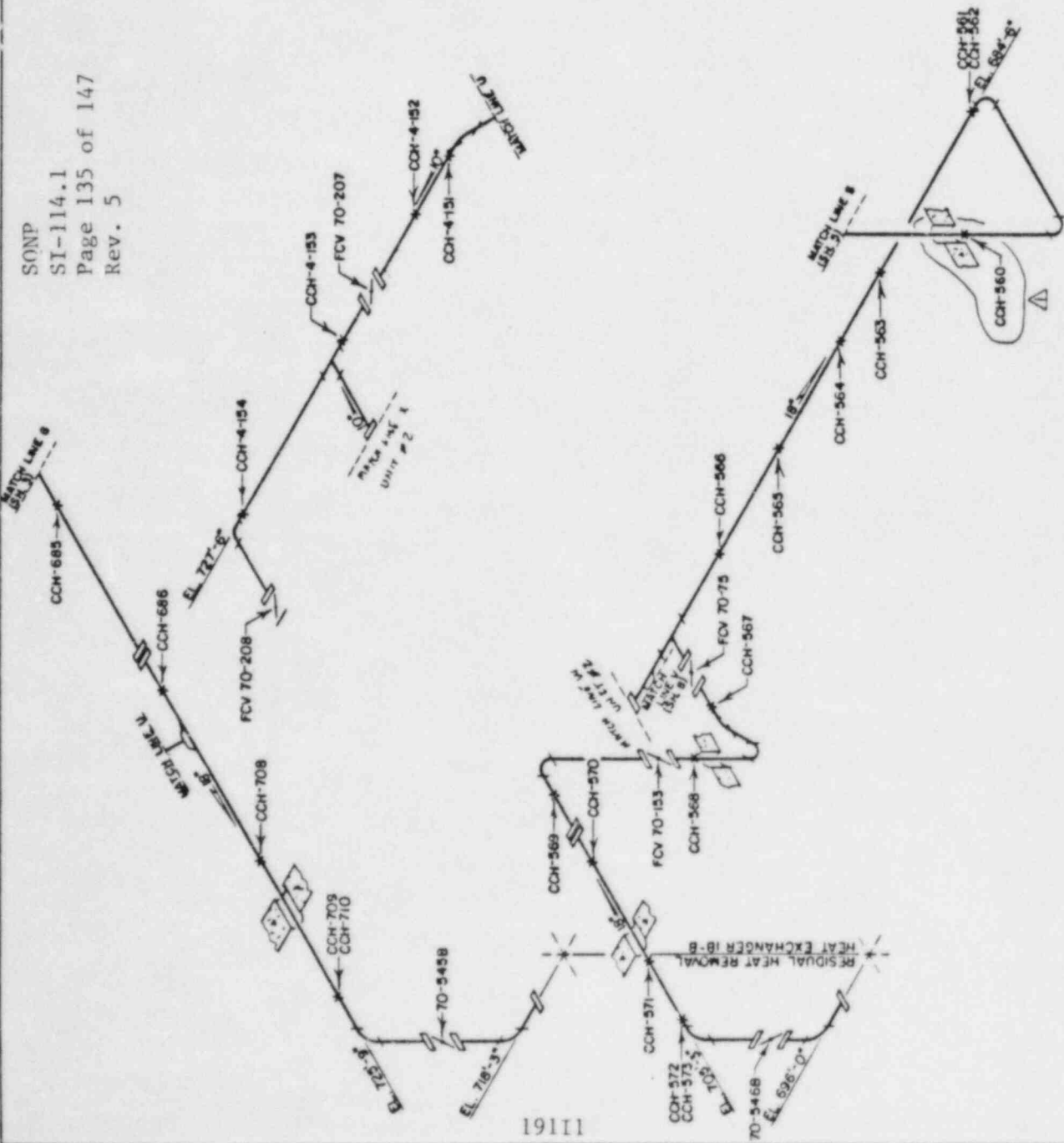
1	4-30-84	DALETTI SUPPMT	REV	SP/ABP
NO.	DA-E	REVISIONS	REV	EX'D/APP
TENNESSEE VALLEY AUTHORITY DIVISION OF NUCLEAR POWER				
SEQUOYAH NUCLEAR PLANT UNIT #1				
COMPONENT COOLING SYSTEM				
SUPPORT LOCATIONS				
DATE	BY	CHKD	APP'D	REV
11/25/83	ABP	ABP	ABP	1
				SI-0126-C





REFERENCE DRAWINGS  
 47K464-34  
 47K464-35  
 47K464-36  
 TVA SAFETY CLASS C

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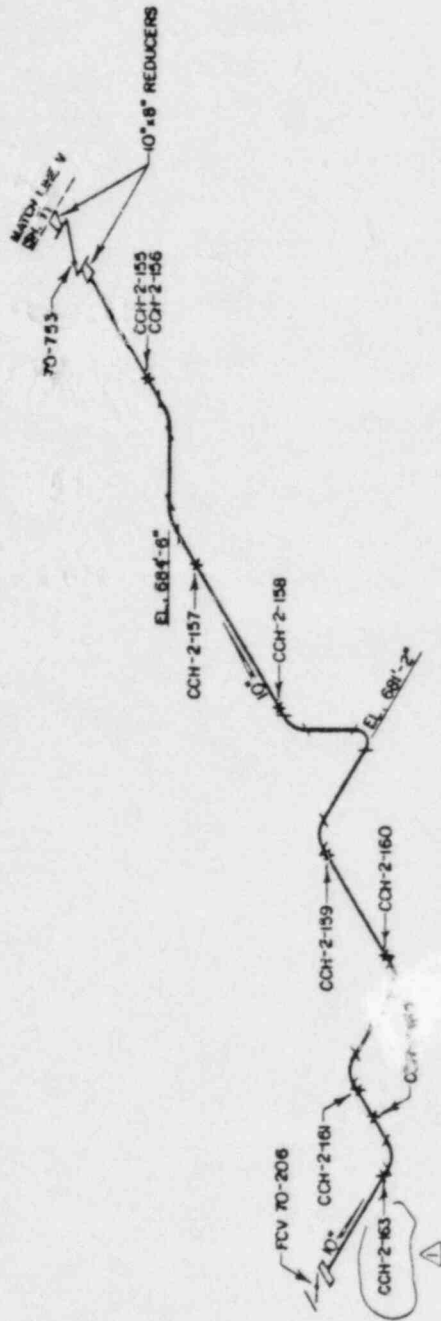
11161

NO.	DATE	REVISIONS	CK'D APP.
1	4-30-87	CONCRETE SUPPORT LOCATION REV	200/138
Tennessee Valley Authority DIVISION OF NUCLEAR POWER			
SECOYAH NUCLEAR PLANT UNIT #1			
COMPONENT COOLING SYSTEM			
SUPPORT LOCATIONS			
25	83	200/138	200/138
TVA SAFETY CLASS C			

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REFERENCE DRAWINGS  
 47K464-99  
 TVA SAFETY CLASS C

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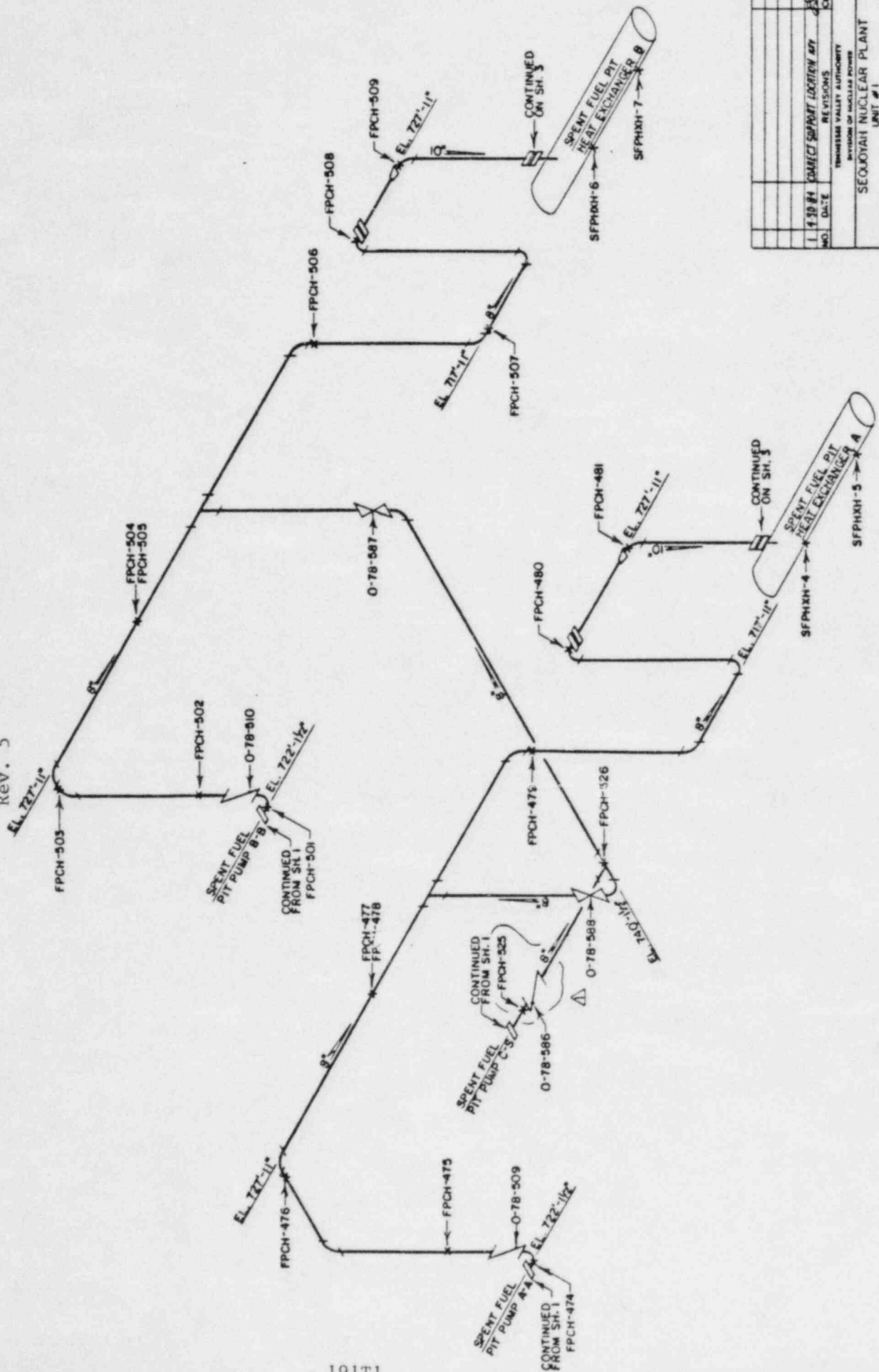
191J1

NO.	DATE	REVISIONS	BY	CHK'D
1	4-30-84	CONVERT SUPPORT NUMBER MAP	EC	AK
Tennessee Valley Authority DIVISION OF NUCLEAR POWER				
SEQUOYAH NUCLEAR PLANT				
UNIT # 1				
COMPONENT COOLING SYSTEM				
SUPPORT LOCATIONS				
APP'D	DATE	BY	CHK'D	DATE
EC	7/2/88	AK	AK	7/2/88
NO.	DATE	REVISIONS	BY	CHK'D
SI-0126-C				



REFERENCE DRAWINGS  
 47K454-90  
 47K454-91  
 TVA SAFETY CLASS C

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191T1

Tennessee Valley Authority	
DIVISION OF NUCLEAR POWER	
SECOYAH NUCLEAR PLANT	
UNIT #1	
FUEL POOL COOLING AND CLEANING SYSTEM	
SUPPORT LOCATIONS	
NO.	DATE
1	11-19-84
CONNECT SUPPORT LOCATION 477	
REV.	BY
1	11-19-84
2	11-19-84
3	11-19-84
4	11-19-84
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100	11-19-84

ENCLOSURE 2

Inservice Inspection Program  
for Sequoyah Nuclear Plant, Unit 2

Revisions 1 through 5 to SI-114.2

SI-114.2  
HISTORY OF REVISIONS

Revision 1:

Pages: 61, 62, and 231

Revision 2:

In-Service Inspection Program Cover Sheet

Revision 3:

Pages: 61 and 62

Revision 4:

Table of Contents, 28, 29, 33, 49-52C, 62-67, 74, 82, 93, 95-99,  
Appendix A: 119, 126, 130, 132, 135-137, 145-148, 164-167, 176, 177, 179,  
180, 183, 185, 187, 196, 199, 200, 214, 215, 218, 228-283

Revision 5:

Pages: 122D, 282A-282A67, 284-331

16.3 Records for Class A, B, C, and D Components

The following records shall be available for review:

- (1) Examination Plans
- (2) Examination Results and Reports
- (3) Examination Methods and Procedures
- (4) Evaluation of Results
- (5) Corrective Actions and Repairs

16.4 Records of System Pressure Tests

Records of the visual examinations conducted during system leakage or hydrostatic tests shall consist of an itemization of the number and location of leaks found in a system and the corrective actions taken.

16.5 Augmented Examination Reports

Augmented examination special reports shall be submitted to the Nuclear Regulatory Commission Region II Office within the time period specified for each report. For specific details on records, reports and reporting see section 20.0, Augmented Inspections, of this program.

17.0 NOTIFICATION OF INDICATION

Plant management shall be formally notified of the presence of unacceptable indications detected during the performance of nondestructive examinations. Unacceptable indications are defined by the applicable NDE procedure. Formal notification shall consist of completing and submitting to the Plant Superintendent the "Notification of Indication" form in Appendix D of this program.

Part I of the "Notification of Indication" shall be completed and signed by the NDE Level II or III examiner detecting the indication. The NDE Section representative shall assign a sequential number and review and sign the form. If the indication is detected by an outside contractor, the contractor's field supervisor shall review and sign the form. The original shall be sent to the plant superintendent and a copy to the ISI Programs Section.



The plant superintendent or his assistant shall designate the organization (Field Services, Plant Maintenance, or the Office of Engineering Design and Construction-OEDC) responsible for preparing a disposition in Part II of the form and performing the associated corrective action. If the organization assigned responsibility for disposition is unable to determine a satisfactory disposition then the form should be sent to the Mechanical Branch for disposition. The individual responsible for preparation of the disposition shall sign and date Part II of the form. The cognizant supervisor of the appropriate organization shall review and approve the disposition and sign and date Part II of the form. Copies of the form shall be distributed to the plant superintendent and the ISI Programs Section. The original shall be returned to the NDE Section Representative. One copy shall be filed with the examination report.

If the organization assigned responsibility for corrective action is within NUC PR, they shall evaluate the unacceptable condition and process, as appropriate, in accordance with the requirements of N-OQAM, Part III, Section 7.2. Dispositions to correct the condition under the plant maintenance program shall be processed in accordance with N-OQAM, Part II, Section 2.1. Dispositions other than restoring to original requirements shall be processed as modifications in accordance with N-OQAM, Part II, Section 3.0. Repair and replacement, activities, including coordination with the Authorized Inspection Agency (AIA), shall be performed in accordance with the requirements of N-OQAM, Part II, Section 2.3. Dispositions to accept the condition as-is shall include in Part II of the form the basis for the disposition. In addition for dispositions to accept the condition as-is, a USQD shall be prepared by the appropriate organization in accordance with established procedures. If Construction is responsible for corrective action, it shall be performed in accordance with the disposition on the Notification of Indication form and to organization responsible for corrective action shall include preservice examination requirements in the repair or replacement work instruction described in N-OQAM, Part II, Section 2.3.

Upon completion of corrective action the NDE Section Representative shall verify completion of corrective action, enter the work instruction and/or DCR numbers on the Notification of Indication form, enter the examination report number if re-examination was performed, and sign and date the form, Part III. The signed form shall remain with the examination report for use as a quality assurance record. If re-examination was performed, a copy of the signed form shall also remain with the re-examination report. Copies of the form shall also be distributed to the plant superintendent and the ISI Programs Section.

APPENDIX D

NOTIFICATION OF INDICATION

PART I - FINDINGS

NOI No. \_\_\_\_\_ Plant/Unit \_\_\_\_\_  
Examination Report No. \_\_\_\_\_ Component ID \_\_\_\_\_  
Delete Drawing No. \_\_\_\_\_

Description of Indication: (Sketch/Photograph if needed)  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

Signature of Examiner/Certif. Level \_\_\_\_\_ Date \_\_\_\_\_  
Signature of Field Supervisor (Contractor) \_\_\_\_\_ Date \_\_\_\_\_  
NDE Section Representative \_\_\_\_\_ Date \_\_\_\_\_

PART II-DISPOSITION

\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_  
Disposition Prepared By \_\_\_\_\_ Date \_\_\_\_\_  
Disposition Approved By \_\_\_\_\_ Date \_\_\_\_\_

PART III-VERIFICATION

Verification of Completed Corrective Action and/or Examination By NDE  
Section Representative  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_  
Signature \_\_\_\_\_ Date \_\_\_\_\_

SQNP  
SI-114.2  
Cover Sheet  
Page 1 of 1  
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Inservice Inspection Program

Cover Sheet

Owner: Tennessee Valley Authority

Address of Corporate Office: Knoxville Office Complex  
400 Commerce Avenue  
Knoxville, Tennessee 37902

Name and Address of Nuclear Power Plant: Sequoyah Nuclear Plant  
P.O. Box 2000  
Daisy, Tennessee 37319

Applicable Nuclear Power Units: Sequoyah Nuclear Plant, Unit 2

Commercial Operation Date: June 1, 1982

16.3 Records for Class A, B, C, and D Components

The following records shall be available for review:

- (1) Examination Plans
- (2) Examination Results and Reports
- (3) Examination Methods and Procedures
- (4) Evaluation of Results
- (5) Corrective Actions and Repairs

16.4 Records of System Pressure Tests

Records of the visual examinations conducted during system leakage or hydrostatic tests shall consist of an itemization of the number and location of leaks found in a system and the corrective actions taken.

16.5 Augmented Examination Reports

Augmented examination special reports shall be submitted to the Nuclear Regulatory Commission Region II Office within the time period specified for each report. For specific details on records, reports and reporting see section 20.0, Augmented Inspections, of this program.

17.0 NOTIFICATION OF INDICATION

Plant management shall be formally notified of the presence of unacceptable indications detected during the performance of nondestructive examinations (excluding VT-2, visual examinations performed during system pressure tests). Unacceptable indications are defined by the applicable NDE procedure. Formal notification shall consist of completing and submitting to the Plant Superintendent the "Notification of Indication" form in Appendix D of this program.

Part I of the "Notification of Indication" shall be completed and signed by the NDE Level II or III examiner detecting the indication. The NDE Section representative shall assign a sequential number and review and sign the form. If the indication is detected by an outside contractor, the contractor's field supervisor shall review and sign the form. The original shall be sent to the plant superintendent and a copy to the ISI Programs Section.

The plant manager or his assistant shall designate the organization (Field Services, Plant Maintenance), responsible for preparing a disposition in Part II of the form and performing the associated corrective action. If the organization assigned responsibility for disposition is unable to determine a satisfactory disposition then the form should be sent to the Mechanical Branch for disposition. The individual responsible for preparation of the disposition shall sign and date Part II of the form. The cognizant supervisor of the appropriate organization shall review and approve the disposition and sign and date Part II of the form. Copies of the form shall be distributed to the plant superintendent and the ISI Programs Section. The original shall be returned to the NDE Section Representative. One copy shall be filed with the examination report.

If the organization assigned responsibility for repair is within NUC PR, they shall determine if the unacceptable condition is significant and potentially reportable in accordance with the requirements of N-OQAM, Part III, Section 7.2. Dispositions to correct the condition under the plant maintenance program shall be processed in accordance with N-OQAM, Part II, Section 2.1. Dispositions other than restoring to original requirements shall be processed as modifications in accordance with N-OQAM, Part II, Section 3.0. Repair and replacement, activities, including coordination with the Authorized Inspection Agency (AIA), shall be performed in accordance with the requirements of N-OQAM, Part II, Section 2.3. Dispositions to accept the condition as-is shall include in Part II of the form the basis for the disposition. In addition for dispositions to accept the condition as-is, a USQD shall be prepared by the appropriate organization in accordance with established procedures. The organization responsible for corrective action shall include preservice examination requirements in the repair or replacement work instruction described in N-OQAM, Part II, Section 2.3.

Upon completion of corrective action the NDE Section Representative shall verify completion of corrective action, enter the work instruction and/or DCR numbers on the Notification of Indication form, enter the examination report number if re-examination was performed, and sign and date the form, Part III. The signed form shall remain with the examination report for use as a quality assurance record. If re-examination was performed, a copy of the signed form shall also remain with the re-examination report. Copies of the form shall also be distributed to the plant superintendent and the ISI Programs Section.

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All of the bolts from each pump shall be ultrasonically examined during the inspection interval. The bolts may be examined in place under tension or when removed. All of the bolts from one pump shall be examined during the first inspection period, and all of the bolts from a different pump shall be examined during the second inspection period. All of the bolting from the two remaining pumps shall be examined during the third inspection period.

Removed, the bolts shall be ultrasonically and magnetic particle examined. This examination needs to be performed only once during the inspection interval and may be deferred to the third inspection period. Provisions for this examination are included in MI-2.2.

When a main flange connection is disassembled, the threads in the base material and flange ligaments between threaded bolt holes shall be visually examined in accordance with visual examination method VT-1 (see Section 11.1). Provisions for this examination are included in MI-2.2.

The main flange bolts are fabricated of 4340 steel, heat treated to A-540, Gr B24.

#### 7.5.2 RCP Pressure Retaining Bolting 2 Inches and Smaller in Diameter

Each RCP includes two sets of pressure retaining bolting 2 inches and smaller in diameter. The bolting sets include the number 1, and 2 seal assembly bolting. The number 1 and 2 seal assembly bolting from each pump shall be visually examined in accordance with visual examination method VT-1 (see Section 11.1).

The bolting may be examined in place under tension or when removed. It is preferable to perform the examinations when the bolts are removed if the connection(s) is disassembled. Removal of bolting is performed in accordance with MI-2.1 and provides for examination of bolting.

All of the bolting from one RCP shall be examined during the first inspection period, and all of the bolting from a different pump shall be examined during the second inspection period. All of the bolting from the remaining two pumps shall be examined during the third inspection period.



The number 1, and 2 seal housing bolting includes 12 socket head cap screws at 2 inches in diameter, 12 socket head cap screws at 1 inch in diameter, respectively.

#### 7.5.3 RCP Integrally-Welded Support Members

There are three unit 2 reactor coolant pumps, serial numbers 566 (RCP-1 casing), 704 (RCP-3 casing), and 709 (RCP-4 casing), with integrally welded supports. Each pump has three integrally welded pump feet. These integrally welded pump feet whose design base is  $> 5/8$  of an inch shall be surface examined during the inspection interval in accordance with Table A of Appendix A. The examination shall cover 100 percent of the required area of each support attachment.

All of the supports from one RCP shall be examined during the first inspection period, and all of the supports from a different pump shall be examined during the second inspection period. All of the supports from the remaining two pumps shall be examined during the third inspection period.

#### 7.5.4 RCP Component Supports

Each RCP includes three integrally cast pump feet bolted to the support system. All component supports from each pump shall be visually examined during the inspection interval in accordance with visual examination method VT-3 (see Section 11.1). Support components extend from the RCP to and including the attachment to the supporting structure.

All of the supports from one RCP shall be examined during the first inspection period, and all of the supports from a different pump shall be examined during the second inspection period. All of the supports from the remaining two pumps shall be examined during the third inspection period.

#### 7.5.5 RCP Casing Welds

The unit 2 reactor coolant pumps do not include casing welds.

#### 7.5.6 RCP Casing

If a pump is disassembled for maintenance during the inspection interval, the internal pressure boundary surfaces shall be visually examined in accordance with visual examination method VT-1 (see Section 11.1). Disassembly of RCP's is performed in accordance with MI-2.2 and provides for these visual examinations.

Components that are exempted from examination in accordance with IWC-1220 of ASME Section XI are discussed in Section 8.7 of this program.

Where examinations specify a percentage of the total length of weld to be examined, the areas(s) examined shall be documented in the examination report. Where a percentage of weld length is not referenced, the entire weld length shall be examined. (See Request for Relief ISI-3.)

Table B in Appendix A supplies additional information such as reference drawing numbers and ASME Section XI Table-IWC-2500-1 examination categories.

## 8.1 Steam Generators (4)

### 8.1.1 Steam Generator Secondary Side Circumferential Shell Welds

There are three circumferential shell welds at gross structural discontinuities on each generator. The entire length of these three shell welds from one steam generator shall be ultrasonically examined during the inspection interval. The number of welds to be examined during each inspection period shall be in accordance with Table B of Appendix A.

One of the three welds on each steam generator is partially inaccessible for examination due to the upper steam generator support arrangement (weld nos. SGW-D1, D2, D3, and D4; see Request for Relief ISI-4). The weld selected for examination shall be ultrasonically examined on a best-effort basis.

The vessel shell sections are fabricated of SA-533, GR. A, Class 1, steel plate.

### 8.1.2 Steam Generator Secondary Side Circumferential Head Welds

Each steam generator includes a circumferential head-to-shell weld. The entire length of one head-to-shell weld shall be ultrasonically examined during the inspection interval in accordance with Table B of Appendix A. The weld selected for examination may be from the generator selected for examination by Section 8.1.1.

The vessel head section is fabricated of SA-533, GR. A, Class 1, steel plate.

Two of the supports from one RHRP shall be examined during the first inspection period. The final (3rd) support from the same RHRP shall be examined during the second inspection period. All of the supports from the remaining RHRP shall be examined during the third inspection period.

#### 8.10.1.3 RHRP Supports - Mechanical and Hydraulic

There are no mechanical and hydraulic supports associated with the RHRP.

#### 8.10.1.4 RHRP Pressure Retaining Bolting

The stuffing box extension to pump casing connection bolting is not greater than two inches in diameter.

The connection includes 24 studs at 1-1/4 inches in diameter with nuts and washers. The studs are fabricated to SA-193, GR. B7, and the nuts to SA-194, GR. 2H.

#### 8.10.1.5 RHRP Casing Welds

The RHRP does not include any casing welds. The casing is a one piece forging fabricated to SA-182, F304.

### 8.10.2 CVCS Centrifugal Charging Pumps (2) CCP

#### 8.10.2.1 CCP-Integrally-Welded Supports

Each CCP includes four integrally-welded supports whose design base is greater than 3/4 inch thick. These supports shall be surface examined during the inspection interval in accordance with Table B of Appendix A.

#### 8.10.2.2 CCP-Component Supports

Each CCP includes 4 integrally-welded feet bolted to the support system. All component supports shall be visually examined during the inspection interval in accordance with visual examination method VT-3 (see section 11.1). The examination shall be distributed during the inspection interval in accordance with Table B of Appendix A. Support components extend from the CCP to and including the attachment to the supporting structure.

8.10.2.3 CCP Supports - Mechanical and Hydraulic

There are no mechanical and hydraulic supports associated with the CCP.

8.10.2.4 CCP Pressure Retaining Bolting

There is no CCP bolting greater than two inches in diameter.

8.10.2.5 CCP-Casing Welds

The CCP casing does not include any casing welds. The casing consists of an upper and lower housing fabricated to SA-182 F304.

8.10.3 CVCS Positive Displacement Pump (1) PDP/Reciprocating Charging Pump

8.10.3.1 PDP Integrally-Welded Supports

There are no integrally-welded supports associated with the PDP.

8.10.3.2 PDP Component Supports

The PDP includes a support component bolted to the pump feet which are integrally forged with the pump. The component support shall be visually examined during the inspection interval in accordance with visual examination method VT-3 (see section 11.1). The examinations shall be distributed during the inspection interval in accordance with Table B of Appendix A. Support components extend from the PDP to and including the attachment to the supporting structure.

8.10.3.3 PDP Supports - Mechanical and Hydraulic

There are no mechanical and hydraulic supports associated with the PDP.

8.10.3.4 PDP Pressure Retaining Bolting

The cylinder tie, cylinder head and valve cover, and the stuffing box flange bolting connections are not greater than 2 inches in diameter.

The connection includes 8 studs at 1-1/8" in diameter with nuts and washers for the cylinder ties; 40 cylinder head and valve cover studs at 1 1/4" in diameter with nuts and washers; and 20 studs at 1 1/4" in diameter with nuts and washers for the stuffing box flange. The studs are fabricated to SA-193 GRB7 and the nuts to SA-194, GR7.

#### 8.10.3.5 PDP Casing Welds

The PDP casing does not include any casing welds. The casing is fabricated to SA-182 F304, SA-204 Type 304, SA-479 T304, and SA-479, 410.

#### 8.10.4 Safety Injection Pumps (2)-SIP

##### 8.10.4.1 SIP Integrally-Welded Supports

There is no integrally-welded supports associated with the SIP.

##### 8.10.4.2 SIP Component Supports

Each SIP includes a support component bolted to the pump casing. All component supports from each pump shall be visually examined during the inspection interval in accordance visual examination method VT-3 (see section 11.1). The examinations shall be distributed during the inspection interval in accordance with Table B of appendix A. Support components extend from the SIP to and including the attachment to the supporting structure.

##### 8.10.4.3 SIP Supports - Mechanical and Hydraulic

There are no mechanical and hydraulic supports associated with SIP.

##### 8.10.4.4 SIP Pressure Retaining Bolting

The SIP case capnuts are not greater than 2 inches in diameter. The casing bolting includes 32 studs at 2 inches in diameter with nuts and washers and 10 studs at 1 1/2 inches in diameter with nuts and washers. The studs are fabricated to SA-193, GRB7 and the nuts to SA-194, GR7.

##### 8.10.4.5 SIP Casing Welds

The SIP casing does not include any casing welds. The casing consists of upper and lower housing fabricated to A351 GF8.

## 8.11 Valves

### 8.11.1 Valve Integrally-Welded Supports

Examination of valve integrally-welded support members is included in Section 8.3.3.

### 8.11.2 Valve Component Supports

Examination of valve component supports is included in Section 8.3.4.

### 8.11.3 Valve Pressure-Retaining Bolting

There is no Class B pressure-retaining bolting greater than 2 inches in diameter.

### 8.11.4 Valve Body Welds

There are no Class B valves with body welds.

## 8.12 Pressure Retaining Components

All Class B pressure retaining components in systems or portions of systems which are not required to operate during normal reactor operation but for which periodic system or component functional tests are required (excluding open-ended portions of systems), shall be subjected to a system functional leakage test with each inspection period in accordance with IWC-5221 of ASME Section XI. This is performed in accordance with SI's to be developed later. Those Class B pressure retaining components in systems or portions of systems not subject to the functional leakage tests (excluding open-ended portions of systems), shall be subjected to a system hydrostatic test with each inspection interval in accordance with IWC-5222 of ASME Section XI. This is performed in accordance with SI-265.3. The components shall be visually examined during the pressure tests in accordance with visual examination method VT-2 (see Section 11.1). All Class B (ASME Class 2) system pressure tests shall be performed in accordance with DPM SEQ82E1.

## 8.13 Exempted Components

### 8.13.1 Exempted Components (Except Piping Welds)

Components exempted from examination include: (1) Components in systems where both the design pressure and temperature are equal to or less than 275 psig and 200°F, respectively; (2) components in systems or portions of systems, other than emergency core cooling

systems, which do not function during normal reactor operation; (3) components which perform an emergency core cooling function for which the control of the chemistry of the contained fluid is verified by periodic sampling and test, and (4) component connections, piping, and associated valves, and vessels (and their supports), that are 4-inch nominal pipe size and smaller.

#### 8.13.2 Exempted Components (Piping Welds Only)

Piping exempted from examination include: (a) Piping systems where both the design pressure and temperature are equal to or less than 275 psig and 200°F, respectively; (b) piping systems or portions of systems other than emergency core cooling systems which do not function during normal reactor operation; (c) piping that is 4-inch nominal pipe size and smaller; (d) components which perform an emergency core cooling function provided the control of the chemistry of the contained fluid is verified by periodic sampling and test.

### 9.0 COMPONENTS SUBJECT TO EXAMINATION - TVA SAFETY CLASS C AND D

Class C and D (ASME Class 3) pressure retaining components identified in sections 9.1.1 and 9.1.2 shall be visually examined, VT-2, in conjunction with the system pressure test in accordance with IWA-5000, IWD-2000, and IWD-5000.

The component supports and restraints within the boundaries identified in sections 9.1.1 and 9.1.2 for components exceeding 4-inch nominal pipe size shall be visually examined, VT-3, during each inspection period.

Mechanical and hydraulic snubbers, spring loaded and constant weight supports within the boundaries identified in sections 9.1.1 and 9.1.2 for components exceeding 4-inch nominal pipe size shall be visually examined, VT-4, during each inspection period.

#### 9.0.1 Piping and Valve Component Supports

All piping and valve component supports shall be visually examined during each inspection period in accordance with visual examination methods VT-3 and VT-4 (see Section 11.1). This examination includes integrally-welded and nonintegrally-welded component supports. Component supports extend from the piping and valves to and including the attachment to the supporting structure. The setting of snubbers, shock absorbers and spring-type hangers shall be verified. The examinations (100%) shall be conducted during each inspection period during the inspection interval in accordance with Table C of Appendix A.

9.0.1.1 Auxiliary Feedwater System Piping and Valve Component Supports

The auxiliary feedwater system piping includes 20 component supports. All of these shall be examined during each inspection period.

9.0.1.2 Chemical and Volume Control System Piping and Valve Component Supports

The chemical and volume system piping includes six component supports. All of these shall be examined during each inspection period.

9.0.1.3 Component Cooling System Piping and Valve Component Supports

The component cooling system piping includes 205 component supports. All of these shall be examined during each inspection period.

9.0.1.4 Containment Spray System Piping and Valve Component Supports

The containment spray system piping including 27 component supports. All of these shall be examined during each inspection period.

9.0.1.5 Essential Raw Cooling Water System Piping and Valve Component Supports

The essential raw cooling water system piping includes 211 component supports. All of these shall be examined during each inspection period.

9.0.1.6 Fuel Pool Cooling System Piping and Valve Component Supports

The fuel pool cooling system component supports are examined in accordance with SI-114.1, Unit 1 program.

9.0.1.7 Residual Heat Removal System Piping and Valve Component Supports

The residual heat removal system piping includes seven component supports. All of these shall be examined during each inspection period.



9.0.1.8 Safety Injection System Piping and Valve Component Supports

The safety injection system piping includes 26 component supports. All of these shall be examined during each inspection period.

9.1 System Pressure Tests

All Class C and D (ASME Class 3) system pressure tests shall be performed in accordance with DPM SEQ82E1.

9.1.1 System Inservice Tests

Pressure retaining components within the boundary of systems or portions of systems required to operate in support of normal plant safety functions of shutting down and maintaining the reactor in the cold shutdown condition, and pressure retaining piping, pumps, and valves within the boundary or systems or portions of systems required to operate in support of residual heat removal from spent fuel storage pool, shall be visually examined in accordance with visual examination method VT-2 (see section 11.1) while the applicable systems are in service under operating pressure in accordance with IWD-5221 of ASME Section XI.

These examinations and tests shall be performed during each inspection period.

9.1.2 System Functional Tests

Pressure retaining components within the boundary of systems or portions of systems required to operate in support of the postaccident safety functions of emergency core cooling, containment heat removal and atmospheric cleanup, and long-term residual heat removal from the reactor vessel shall be visually examined in accordance with visual examination method VT-2 (see Section 11.1) during a system functional test conducted to verify operability in systems in accordance with IWD-5222 of ASME Section XI.

9.1.3 System Hydrostatic Tests

Pressure retaining components identified in Sections 9.1.1 and 9.1.2 shall be visually examined in accordance with visual examination method VT-2 (see Section 11.1) during a system hydrostatic test conducted in accordance with IWD-5223 of ASME Section XI. These examinations and tests shall be performed over each inspection interval.

- (2) Examination Results and Reports
- (3) Examination Methods and Procedures
- (4) Evaluation of Results
- (5) Corrective Actions and Repairs

16.4 Records of System Pressure Tests

Records of the visual examinations conducted during system leakage or hydrostatic tests shall consist of an itemization of the number and location of leaks found in a system and the corrective actions taken.

16.5 Augmented Examination Reports

Augmented examination special reports shall be submitted to the Nuclear Regulatory Commission Region II Office within the time period specified for each report. For specific details on records, reports and reporting see section 20.0, Augmented Inspections, of this program.

17.0 NOTIFICATION OF INDICATION

Plant management shall be formally notified of the presence of unacceptable indications detected during the performance of nondestructive examinations (excluding VT-2, visual examinations performed during system pressure tests). Unacceptable indications are defined by the applicable NDE procedure. Formal notification shall consist of completing and submitting to the Plant Superintendent the "Notification of Indication" form in Appendix D of this program.

Part I of the "Notification of Indication" shall be completed and signed by the NDE Level II or III examiner detecting the indication. The NDE Section representative shall assign a sequential number and review and sign the form. If the indication is detected by an outside contractor, the contractor's field supervisor shall review and sign the form. The original shall be sent to the plant superintendent and a copy to the ISI Programs Section.

The plant manager or his assistant shall designate the organization (Field Services, Plant Maintenance), responsible for preparing a disposition in Part II of the form and performing the associated corrective action. If the organization assigned responsibility for disposition is unable to determine a satisfactory disposition then the form should be sent to the Mechanical Branch for disposition. The individual responsible for preparation of the disposition shall sign and date Part II of the form. The cognizant supervisor of the appropriate organization

shall review and approve the disposition and sign and date Part II of the form. Copies of the form shall be distributed to the plant superintendent and the ISI Programs Section. The original shall be returned to the NDE Section Representative. One copy shall be filed with the examination report.

If the organization assigned responsibility for repair is within NUC PR, they shall determine if the unacceptable condition is significant and potentially reportable in accordance with the requirements of N-OQAM, Part III, Section 7.2. Dispositions to correct the condition under the plant maintenance program shall be processed in accordance with N-OQAM, Part II, Section 2.1. Dispositions other than restoring to original requirements shall be processed as modifications in accordance with N-OQAM, Part II, Section 3.0. Repair and replacement, activities, including coordination with the Authorized Inspection Agency (AIA), shall be performed in accordance with the requirements of N-OQAM, Part II, Section 2.3. Dispositions to accept the condition as-is shall include in Part II of the form the basis for the disposition. In addition for dispositions to accept the condition as-is, a USQD shall be prepared by the appropriate organization in accordance with established procedures. The

organization responsible for corrective action shall include preservice examination requirements in the repair or replacement work instruction described in N-OQAM, Part II, Section 2.3.

Upon completion of corrective action the NDE Section Representative shall verify completion of corrective action, enter the work instruction and/or DCR numbers on the Notification of Indication form, enter the examination report number if re-examination was performed, and sign and date the form, Part III. The signed form shall remain with the examination report for use as a quality assurance record. If re-examination was performed, a copy of the signed form shall also remain with the re-examination report. Copies of the form shall also be distributed to the plant superintendent and the ISI Programs Section.

#### 18.0 CALIBRATION BLOCKS

Calibration blocks will be used for ultrasonic examinations (a calibration tube will be used for eddy current examination of steam generator tubing). The blocks will be fabricated to the general requirements of ASME Section V and ASME Section XI. The blocks shall be fabricated of the material to be examined or equivalent P numbers. Mill test reports shall be obtained and retained by the Quality Engineering Branch for all calibration blocks. The blocks shall employ drilled holes and/or notches for calibration reflectors (See Request for Relief ISI-2).

The NDE Section shall maintain as-built calibration block drawings. Copies of the original drawings and any revisions shall be submitted to the ISI Programs Section. The calibration blocks shall be stored at the plant site and maintained by the plant QA organization.

#### 19.0 REQUESTS FOR RELIEF

Where TVA has determined that certain code requirements or examinations are impractical, TVA will submit written requests for relief to NRC with information to support the determinations and any proposed alternate examinations. The impractical code requirements or examinations shall be identified in this program, and references to particular requests for relief shall be included.

When impractical examination requirements are identified in the field, the NDE Section shall notify the ISI Programs Section such that the information may be included in this program and requests for relief may be prepared if necessary.

The Requests for Relief are listed in Appendix E.

#### 20.0 AUGMENTED INSPECTIONS

Augmented inspections are performed in addition to ASME Section XI code requirements. The augmented inspections may be required by the NRC or self-imposed by TVA.

Augmented examinations of components that require reporting (verbal or written) to the NRC shall be the responsibility of the organization designated in the particular examination. These reports shall be submitted to the Nuclear Regulatory Commission Region II Office within the time period specified for each report. Each augmented exam shall state the required reporting time and the document requiring the information to be included in the report.

##### 20.1 Feedwater System Piping and Supports

The requirements of NRC-IE Bulletin 79-13 were satisfied during the unit 2, cycle 1 outage. Due to the safety-related ramifications of the steam generator nozzle transition section cracking problem, TVA will perform an augmented inspection (reference memorandum L29 831222 836) of one feedwater nozzle during each refueling outage on a rotating basis. The augmented examination of the steam generator nozzle transition section shall include nozzle-to-transition piece welds and transition piece-to-pipe welds. These welds shall be ultrasonically examined and

ultrasonic sensitivities should be equivalent to those required by NRC-IE Bulletin 83-02. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

#### 20.2 RPV Nozzle Safe Ends

The augmented examination requirements of the RPV nozzle-to-safe end welds are included in the final report - Sequoyah Nuclear Plant - Evaluation of cracking in reactor vessel nozzle stainless steel buttering. TVA will reinspect nozzle 2RC-17SE and one additional unit 2 nozzle (2RC-24SE) at the first regularly scheduled cold shutdown outage. Future examinations for unit 2 will be monitored at the normal inservice inspection intervals for dissimilar metal welds as required by Section XI of the ASME Code. This augmented examination does not require a special report. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

#### 20.3 Reactor Coolant Pump Flywheel

The augmented examination requirements of the reactor coolant pump flywheel are included in Regulatory Position C.4.b of Regulatory Guide 1.14; (1) an in-place ultrasonic examination of the areas of higher stress concentration at the bore and keyway at approximately three-year intervals during the refueling or maintenance shutdown coinciding with the inservice inspection schedule as required by Section XI of the ASME Code, and (2) a surface examination of all exposed surfaces and complete ultrasonic examination at approximately 10-year intervals during the plant shutdown coinciding with the inservice inspection schedule as required by Section XI of the ASME Code.

This augmented examination does not require a special report unless the examination reveals a flaw. If the examination and evaluation indicate an increase in flaw size or growth rate greater than predicted for the service life of the flywheel, the results of the examination and evaluation should be submitted to the NRC for evaluation. Refer to Regulatory Guide 1.14 for information to be included. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

#### 20.4 Steam Generator Tubing

The augmented examination requirements of the steam generator tubing are included in Technical Specifications 4.0.5, 4.4.5.0 and section 7.3.8 of this program. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam

generator shall be reported to the NRC within 15 days. Plant management shall report this information to the Nuclear Regulatory Commission Region II Office within the time period specified. See plant instruction AI-18 for reporting instructions.

The complete results of the steam generator tube inservice inspection shall be submitted to the Nuclear Regulatory Commission in a special report pursuant to Technical Specification 6.9.2 within 12 months following completion of the inspection. The Chemical, Metallurgy, and Standards Group Staff Specialist shall prepare this special report and submit the report of the Nuclear Regulatory Commission Region II Office within the stated time period. This special report shall include:

1. Number and extent of tubes inspected.
2. Location and percent of wall-thickness penetration for each indication of an imperfection.
3. Identification of tubes plugged.

Results of steam generator tube inspections which fall into Technical Specification Category C-3 require prompt notification of the Nuclear Regulatory Commission pursuant to Technical Specification 6.9.1 prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures to prevent recurrence.

Refer to Technical Specifications 4.0.5 and 4.4.5.0 and section 7.3.8 of this program for information to be included.

#### 20.5 RPV Nozzle Cladding

All vessel nozzles cladding shall be ultrasonically examined at the end of each 10-year inspection interval, using techniques at least as sensitive as those used to conduct the supplemental examinations performed prior to fuel loading. The results of this examination shall be reported to NRC. This examination is done in accordance with Sequoyah technical specifications and satisfies Surveillance Requirement 4.4.10. This augmented examination does not require a special report. The examination data sheets shall be included in the 90 day ISI report discussed in section 16.2 of this program.

Nozzle forging material and cladding is identified in section 7.1.2 of this program.

20.6 Steam Generator Feeding J-Tubes

Due to the potential of a loose part (J-Tube) in the secondary side of the steam generators the following J-Tube augmented examination plan shall be initiated.

At refueling outages, less than 920 EFPD, all J-Tubes in one steam generator must be ultrasonically examined for wall thickness in the area above the weld joining the J-Tube to the feeding. This inspection plan would provide wall thickness data on two steam generators per year, one per unit.

For operating periods greater than 920 EFPD, all J-Tubes should be ultrasonically examined on two steam generators. The probability of J-Tube thinning increases significantly after 920 EFPD of operation.

If J-Tube thinning greater than 40 percent is detected during an examination, all J-Tubes in all steam generators must be ultrasonically examined.

This examination plan as outlined shall be adequate to detect J-Tube degradation and provide a planned J-Tube maintenance program. This augmented examination is self imposed by TVA (L29 840109 859) and does not require a report. The examination data sheets shall be included in the 90 day ISI Report discussed in section 16.2 of this program.

21.0 REFERENCES

- 21.1 ASME Boiler and Pressure Vessel Code - Section XI, 1974 Edition, Summer 1975 addenda, Summer 1976 addenda; 1977 Edition, Summer 1978 addenda.
- 21.2 ASME Boiler and Pressure Vessel Code - Section V.
- 21.3 Instruction Manual - 173-inch I.D. Reactor Pressure Vessel -Rotterdam Dockyard Company, Contract No. 68C60-91934, N2M-2-3.
- 21.4 Sequoyah Nuclear Plant Standard Practice SQA41.
- 21.5 Sequoyah Nuclear Plant Final Safety Analysis Report.
- 21.6 Westinghouse Technical Manual - Pressurizer, TM 1440-C225, Contract No. 68C60-91934, N2M-2-6.
- 21.7 Westinghouse Technical Manual - Vertical Steam Generators, TM 1440-C224, Contract No. 68C31-91934, N2M-2-4.
- 21.8 Westinghouse Instruction Manual - Auxiliary Heat Exchangers, Contract No. 68C60-91934, N2M-2-25.
- 21.9 Westinghouse Instruction Book - Reactor Coolant Pump, Contract No. 68C60-91934, N2M-2-5.
- 21.10 Ingersoll-Rand Instruction Manual - Residual Heat Removal Pumps, Contract No. 68C60-91934, N2M-2-30.
- 21.11 Sequoyah Nuclear Plant Operational Quality Assurance Manual, Part II, Sections 2.3, 3.0, 5.1, and 6.3.
- 21.12 Sequoyah Nuclear Plant Technical Instruction TI-51.
- 21.13 Sequoyah Nuclear Plant Technical Specifications Unit 2.
- 21.14 Division Procedures Manual: N75C01, N76A10, N80E3, SEQ80M7, SEQ82E1, and SQ82M1.



APPENDIX A  
 TABLE A  
 SEQUOYAH INSERVICE INSPECTION PROGRAM  
 CLASS A COMPONENTS

Component	Program Reference Section	Exam Method	Section XI Exam Category	40 Yr. Sample No. Length Welds/of Weld	1st Insp. Interval No. Length Welds/of Weld	Inspection Periods			Reference Dwg. No.
						3 yrs.	7 yrs.	10 yrs.	
<u>C. Steam Generators</u>									
4. Pressure Retaining Bolting	7.3.6	VT-1	B-G-2	4 gens./2 mwy	8 mwy	2	2	4	MSG-0002-B
5. Tubing	7.3.8	ET	B-Q	4 gens./3%	4 gens./3%	2	1	1	76M1
<u>D. Pressure Retaining Bolting</u>									
1. Reactor Coolant System	7.4.3.1	VT-1	B-G-2	8 Sets/8 Bolts	8 Sets	2	3	3	ISI-0013-C
2. Chemical and Volume Control System (SWI)	7.4.3.2	VT-1	B-G-2	4 Sets/4 Bolts	4 Sets	1	1	2	ISI-0009-C
3. Residual Heat Removal System	7.4.3.3	VT-1	B-G-2	N/A	See Program Section 7.4.3.3				
4. Safety Injection System	7.4.3.4	VT-1	B-G-2	4 Sets/4 Bolts	4 Sets	1	1	2	ISI-0002-C
5. Upper Head Injection System	7.4.3.5	VT-1	B-G-2	12 Sets/2 Bolts	12 Sets	4	4	4	ISI-0001-C

APPENDIX A  
 TABLE A  
 SEQUOYAH INSERVICE INSPECTION PROGRAM  
 CLASS A COMPONENTS

Component	Program Reference Section	Exam Method	Section XI Exam Category	40 Yr. Sample No. Length Welds/of Weld	1st Insp. Interval No. Length Welds/of Weld	Inspection Periods			Reference Dwg. No.
						3 yrs.	7 yrs.	10 yrs.	
E. Piping (Continued)									
D. Safety Injection System	7.4.8.4	VT-3, VT-4	B-K-2	103	103	35	34	34	MSG-0009-C
E. Upper Head Injection System	7.4.8.5	VT-3, VT-4	B-K-2	88	88	30	29	29	MSG-0014-C
F. Reactor Coolant Pumps									
1. Pressure-Retaining Bolting	7.5.1	UT	B-G-1	96	96	24	24	48	CHM-2675-B
2. Pressure-Retaining Bolting	7.5.1	UT,PT, or MT	B-G-1	96	96	24	24	48	CHM-2675-B
3. Pressure-Retaining Bolting	7.5.2	VT-1	B-G-1	4RCP/2 Sets*	96	24	24	48	CHM-2675-B

\*See Program Reference Section for explanation of inspection sample.

APPENDIX A  
 TABLE B  
 SEQUOYAH INSERVICE INSPECTION PROGRAM  
 CLASS B COMPONENTS

Component	Program Reference Section	Exam Method	Section XI Exam Category	40 Yr. Sample No. Length Welds/of Weld	1st Insp. Interval No. Length Welds/of Weld	Inspection Periods			Reference Dwg. No.
						3 yrs.	7 yrs.	10 yrs.	
D. Piping (Continued)									
c. Main Steam System	8.9.2.3	VT-3, VT-4	C-E	66	66	22	22	22	MSG-0017-C
d. Feedwater System	8.9.2.4	VT-3, VT-4	C-E	35	35	12	11	12	MSG-0016-C
e. Containment Spray System	8.9.2.5	VT-3, VT-4	C-E	5	5	1	2	2	MSG-0011-C
f. Upper Head Injection System	8.9.2.6	VT-3, VT-4	C-E	7	7	2	2	3	MSG-0014-C
3. Circumferential and Longitudinal									
A. Residual Heat Removal System									
a. RHR Circs. >½" Nom. Wall Thickness	8.9.4.1	UT, PT	C-F, C-G	22	5	1	2	2	ISI-0003-C
b. RHR Long. >½" Nom. Wall Thickness	8.9.4.1	UT PT	C-F, C-G	4	1	1	0	0	ISI-0003-C

APPENDIX A  
 TABLE B  
 SEQUOYAH INSERVICE INSPECTION PROGRAM  
 CLASS B COMPONENTS

Component	Program Reference Section	Exam Method	Section XI Exam Category	40 Yr. Sample No. Length Welds/of Weld	1st Insp. Interval			Reference Dwg. No.	
					No. Length Welds/of Weld	3 yrs.	7 yrs.		10 yrs.
D. Piping (Continued)									
D. Feedwater System									
a.	FW Circs >1/2" Nom. Wall Thickness								
	8.9.4.4	UT, MT	C-F, C-G	8	2	0	1	1	CHM-2403-C
b.	FW Long. >1/2" Nom. Wall Thickness								
	8.9.4.4	UT, MT	C-F, C-G	1	1	1	0	0	CHM-2403-C
E. Containment Spray System									
a.	CS Circs <1/2" Nom. Wall Thickness								
	8.9.4.5	PT	C-F, C-G	5	1	1	0	0	ISI-0007-C
b.	CS Long. <1/2" Nom. Wall Thickness								
	8.9.4.5	PT	C-F, C-G	2	1	0	1	0	ISI-0007-C
F. Upperhead Injection System									
	8.9.4.6	UT, PT	C-F, C-G	26	7	2	2	3	ISI-0001-C
F. Pumps									
1. Residual Heat Removal Pumps (2) RHRP									
a.	RHRP Support Components								
	8.9.1.2	VT-3	C-C, C-E	2	2	1	0	1	

APPENDIX A  
 TABLE B  
 SEQUOYAH INSERVICE INSPECTION PROGRAM  
 CLASS B COMPONENTS

Component	Program	Exam Method	Section XI	40 Yr. Sample No. Length Welds/of Weld	1st Insp. Interval	Inspection Periods			Reference Dwg. No.
	Reference Section		Exam Category		No. Length Welds/of Weld	3 yrs.	7 yrs.	10 yrs.	
E. <u>Pumps (Cont.)</u>									
2. <u>CVCS Centrifugal Charging Pumps (2) CCP</u>									
a. CCP, Integrally Welded Supports									
	8.10.2.1	ST	C-C	2	2 Pmps/4 Ft	1	0	1	ISI-0146-A
b. CCP Component Supports									
		VT-3	C-C, C-E	2	2	1	0	1	ISI-0146-A
3. <u>SIS Safety Injection Pumps (2) SIP</u>									
a. SIP Component Supports									
		VT-3	C-C, C-E	2	2	1	0	1	ISI-0147-B
4. <u>CVCS Positive Displacement Pump (1) PDP/Reciprocating Charging Pump</u>									
a. PDP Component Supports									
		VT-3	C-D, C-E	1	1	0	0	1	ISI-0164-A
F. <u>Exempted Components</u>									
	8.12	VT-2	C-H	See Program Section 8.12					

APPENDIX A  
 TABLE C  
 SEQUOYAH INSERVICE INSPECTION PROGRAM  
 CLASS C COMPONENTS

Component	Program Reference Section	Method of Insp.	Section XI Exam Category	40 Yr. Sample No. Length Welds/of Weld	1st Insp. Interval No. Length Welds/of Weld	Inspection Periods			Reference Dwg. No.
						3 yrs.	7 yrs.	10 yrs.	
<b>A. Piping</b>									
1. All Classes C and D	9.0, 9.1	VT-2	D-A, D-B D-C	N/A	N/A	100%	100%	100%	N/A
<b>B. Component Supports</b>									
1. Auxiliary Feedwater System	9.0.1.1	VT-3 VT-4	D-A	30	20	20	20	20	ISI-0146-C
2. Chemical and Volume Control System	9.0.1.2	VT-3 VT-4	D-A	6	6	6	6	6	ISI-0149-C
3. Component Cooling System	9.0.1.3	VT-3 VT-4	D-A, D-B	205	205	205	205	205	ISI-0154-C
4. Containment Spray System	9.0.1.4	VT-3 VT-4	D-B	27	27	27	27	27	ISI-0153-C
5. Essential Raw Cooling Water System	9.0.1.5	VT-3 VT-4	D-A, D-B	211	211	211	211	211	ISI-0153-C
6. Fuel Pool Cooling System	9.0.1.6	VT-3 VT-4	D-C	N/A	N/A	See Program Section 9.0.1.6			



APPENDIX A  
 TABLE D

LIST OF DRAWINGS - UNIT 2

Reactor Vessel

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
ISI-0008-B	Reactor Coolant Piping	159-160
ISI-0001-C	Upper Head Injection System	135-137
CH-M-2341-B	Reactor Vessel Stud Locations and Details	100-101
CH-M-2343-B	Reactor Vessel Seam Welds	102
CH-M-2358-A	Reactor Vessel Closure Head	103
CH-M-2359-A	Control Rod Drive Housing	107-108
CH-M-2360-A	Reactor Vessel Inlet Nozzles	109
CH-M-2361-A	Reactor Vessel Outlet Nozzles	110
ISI-0014-A	Auxiliary Head Adapter	106
ISI-0016-A	Reactor Vessel Clad Patches (PSI Only)	111
MSG-0001-B	Closure Head Cladding Patches (PSI Only)	112
MSG-0004-C	Reactor Vessel Bottom Head Penetrations	113
ISI-0097-C	Closure Head Penetrations	104-105

Pressurizer

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
CH-M-2362-A	Pressurizer Support Skirt Weld	118
CH-M-2663-A	Pressurizer Seam Welds	119
MSG-0002-B	Pressurizer and Steam Generator Cladding Patches	116
MSG-0006-A	Pressurizer Heater Penetrations	120

Steam Generators

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
ISI-0008-B	Reactor Coolant Piping	159-160
CH-M-2345-B	Steam Generator	115
76M1	Vertical Steam Generators (Tube Sheet Arrangement)	114
MSG-0002-B	Pressurizer and Steam Generators Cladding Patches	116
MSG-0005-A	Steam Generator/Feedwater Transition Spool Piece	117



APPENDIX A  
 TABLE D

LIST OF DRAWINGS - UNIT 2

Heat Exchangers

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
CH-M-2404-A	Residual Heat Removal Heat Exchanger	123
ISI-0066-A	Regenerative Heat Exchanger	124-126
ISI-0067-A	Excess Letdown Heat Exchanger	127
ISI-0068-A	Letdown Heat Exchanger	128

Piping and Valve Weld Maps

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
ISI-0008-B	Reactor Coolant Piping (Main Loops)	159-160
ISI-0002-C	Safety Injection System	138-149
ISI-0013-C	Reactor Coolant System	163-173
ISI-0009-C	Chemical and Volume Control System	161-162
ISI-0003-C	Residual Heat Removal System	150-157
ISI-0001-C	Upper Head Injection System	135-137
MSG-0008-C	Seal Water Injection (Chemical and Volume Control System)	176-180
CH-M-2403-C	Feedwater System	133-134
ISI-0015-C	Main Steam System	174-175
ISI-0007-C	Containment Spray System	158

Class A and B  
 Piping and Valve Support Drawings

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
MSG-0013-C	Reactor Coolant System	203-214
MSG-0012-C	Chemical and Volume Control System	202-202
MSG-0015-C	Seal Water Injection (Chemical and Volume Control System)	219-223
MSG-0010-C	Residual Heat Removal System	193-199
MSG-0009-C	Safety Injection System	181-192
MSG-0014-C	Upper Head Injection System	215-218
MSG-0017-C	Main Steam System	226-227
MSG-0016-C	Feedwater System	224-225
MSG-0011-C	Containment Spray System	200

APPENDIX A  
 TABLE C

LIST OF DRAWINGS - UNIT 2

Class C and D Piping and Valve  
 Support Drawings

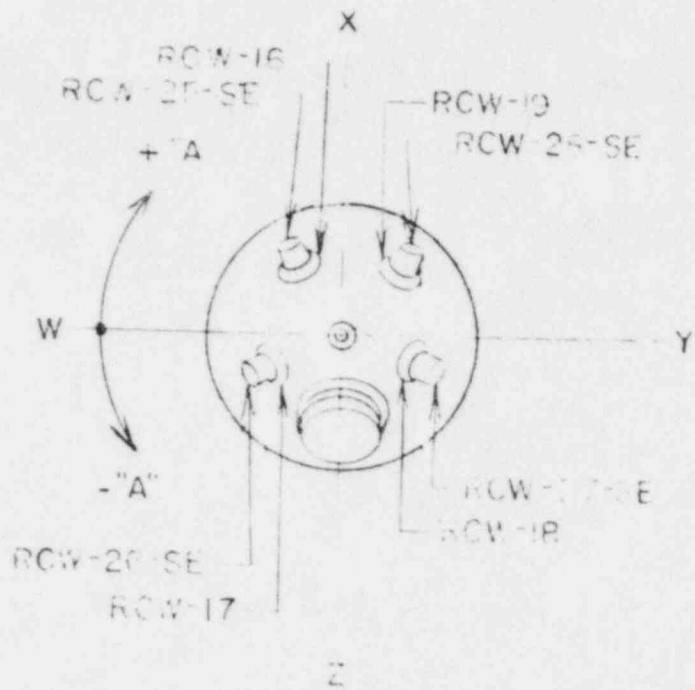
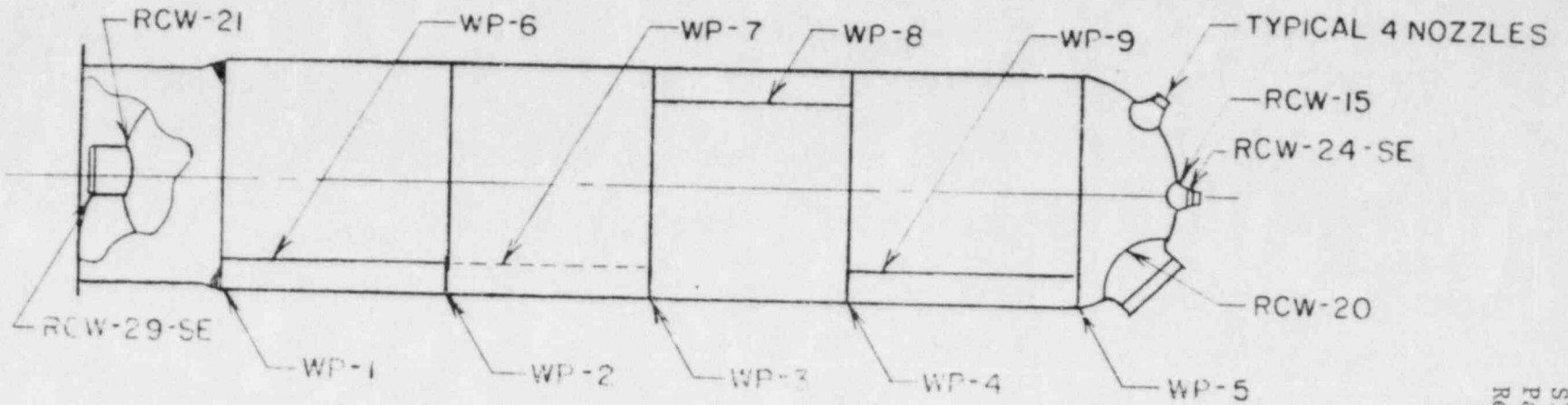
<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
ISI-0146-C	Auxiliary Feedwater System	
ISI-0149-C	Chemical and Volume Control System	
ISI-0150-C	Residual Heat Removal System	
ISI-0151-C	Safety Injection System	
ISI-0153-C	Containment Spray System	
ISI-0154-C	Component Cooling System	
ISI-0158-C	Essential Raw Cooling Water System	

Pumps

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
CH-M-2675-B	Reactor Coolant Pump Main Flange and Lower Seal House Bolt Pattern	121
ISI-0098-A	Reactor Coolant Pump Support Feet	122
ISI-0099-A	Residual Heat Removal Pump Supports	122A
ISI-0146-A	Centrifugal Charging Pump Supports	122B
ISI-0147-B	Safety Injection Pump Supports	122C

Tanks

<u>Drawing No.</u>	<u>Title</u>	<u>Appendix A Page No.</u>
ISI-0070-A	Upperhead Injection Water Accumulator	129-130
ISI-0071-A	Accumulator Surge Tank	131
ISI-0074-A	Boron Injection Tank (Unit 2)	132



AS BUILT DIMENSIONS	
FIELD NO.	"A"
WP-6	-60"
WP-9	-45"

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NO.	DATE	REVISIONS	CK'D APP.
2	3-5-84	REDRAWN JAA	<i>JAA</i>
1	11-8-83	ADD DIMENSION BLOCK	<i>JAA</i>

TENNESSEE VALLEY AUTHORITY  
DIVISION OF NUCLEAR POWER

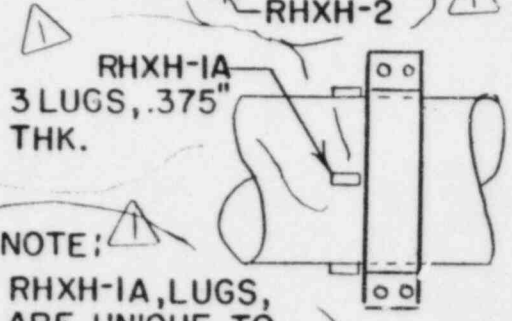
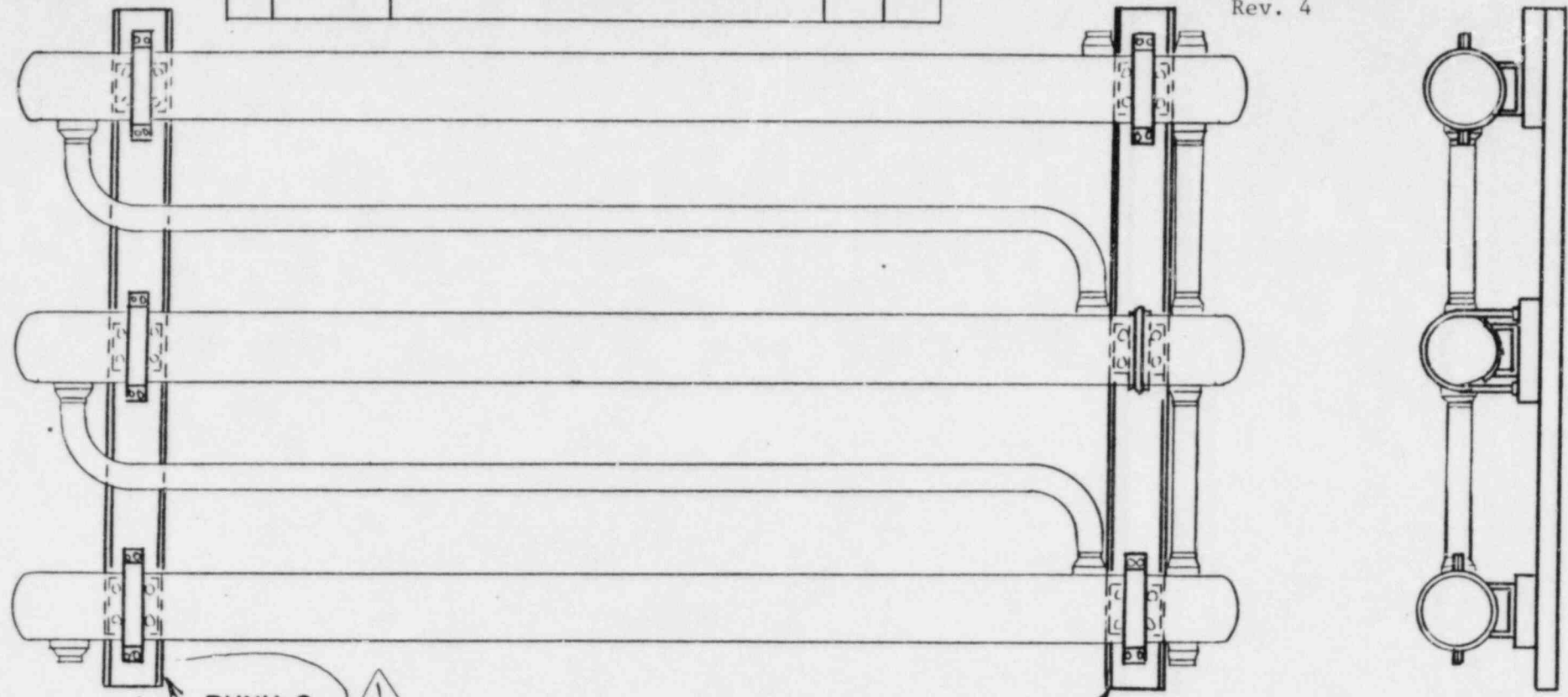
SEQUOYAH NUCLEAR PLANT  
UNITS # 1 & # 2  
PRESSURIZER

SCALE	SUBMITTED	APPROVED	DATE 3-5-84
DRAWN	<i>JAA</i>	<i>JAA</i>	SHEET 1 OF 1 SHEETS
TRACED	<i>JAA</i>		CH-M-2363-A
CHECKED	<i>JAA</i>		

-119-

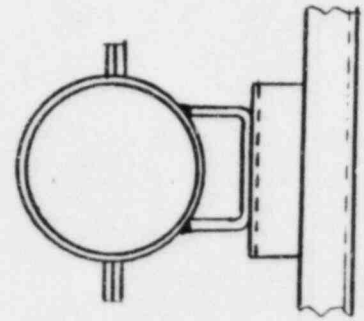
NO.	DATE	REVISIONS	CK'D	APP.
1	2-24-84	CORRECTED DWG. SIZE JAA ADD NOTES + HANGER NO.	EDC	JAB

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NOTE: RHXH-1A, LUGS, ARE UNIQUE TO BOTTOM CLAMP ON SUPPORT RHXH-1

DETAIL



RHXH-1  
(SEE DETAIL)

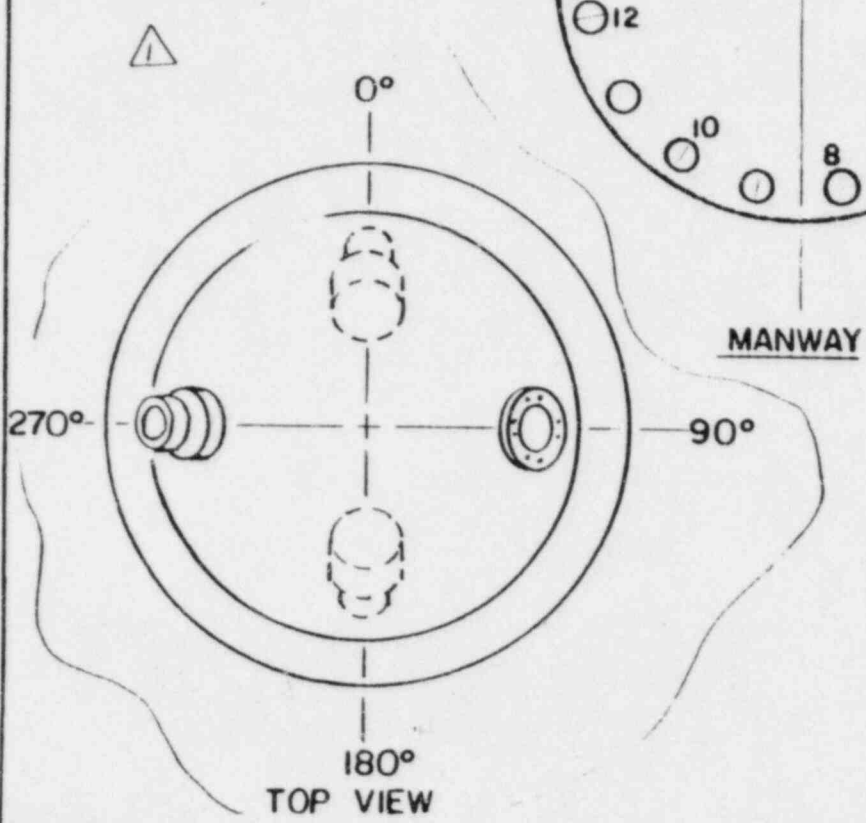
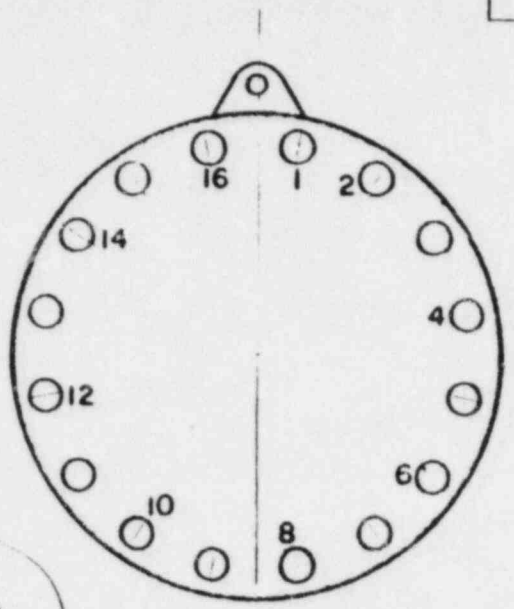
TENNESSEE VALLEY AUTHORITY DIVISION OF NUCLEAR POWER			
SEQUOYAH NUCLEAR PLANT UNITS #1 & #2 REGENERATIVE HEAT EXCHANGER HANGER LOCATIONS			
SCALE: NTS	SUBMITTED	APPROVED	DATE 7-2-82
DRAWN	JAB	JAB	SHEET 3 OF 3 SHEETS
TRACED KEY	EDC	JAB	ISI-0066-A
CHECKED EDC			

R-1

MATERIAL SPECIFICATIONS

STUDS  
 SA 193 B 7  
 2 1/2" DIA. x 11 3/4" LG  
 NUTS  
 SA 194 CL-2H

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NO.	DATE	REVISIONS	CK'D APP.
1	11-9-83	ADD TOP VIEW KEY	EDC JIB

TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
 UNITS #1 & #2  
 UPPER HEAD INJECTION  
 WATER ACCUMULATOR

SCALE NTS	SUBMITTED	APPROVED	DATE 7-30-82
DRAWN BEV	EDC	JIB	SHEET 2 OF 2 SHEETS
TRACED			ISI-0070-A
CHECKED EDC			

R1

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REFERENCE DRAWINGS

113E275  
 A-7496  
 A-7497

MATERIAL SPECIFICATIONS  
 CLASS B

SHELL AND HEAD

SA 516 GR 70  
 SA 240 TP 304L CLADDING

NOZZLES

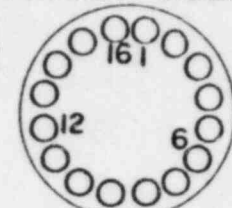
SA 350 FL 2

FITTINGS

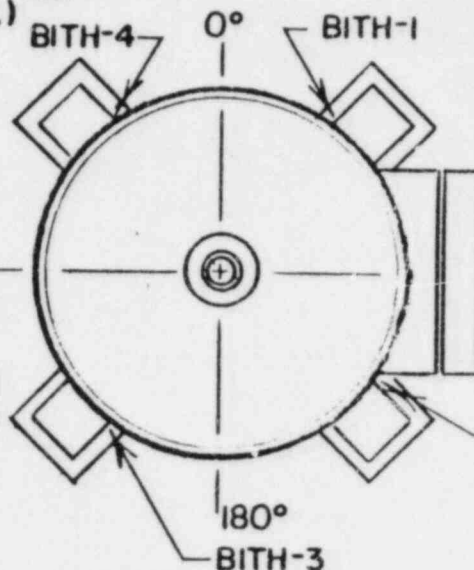
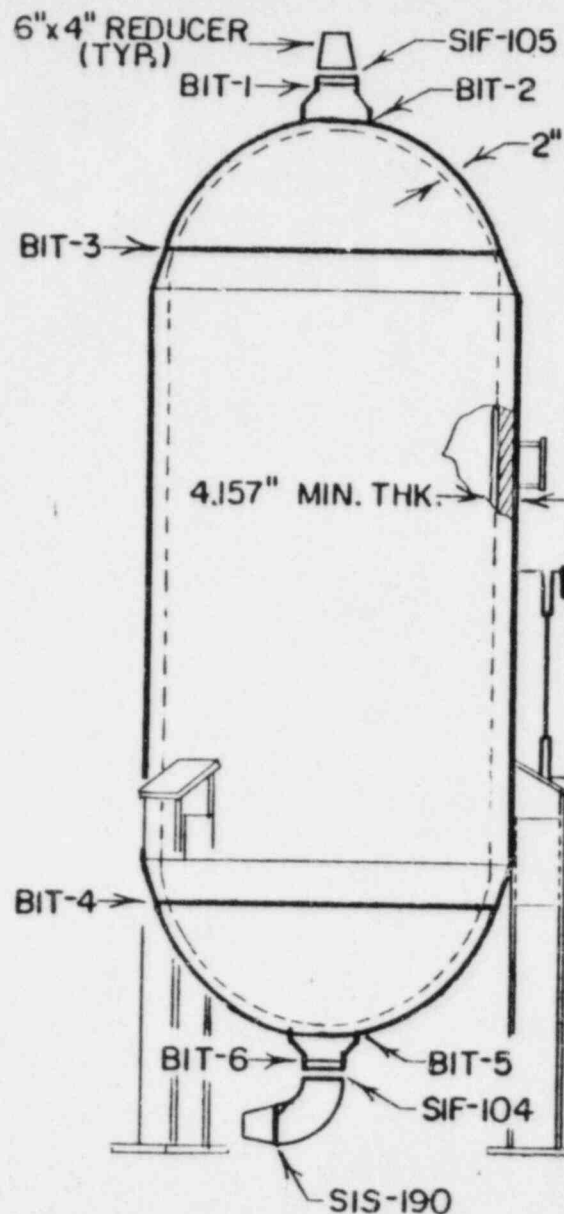
SA 403 WP 316

6" SCH 160

MANWAY BOLTS



SECTION "A"- "A"

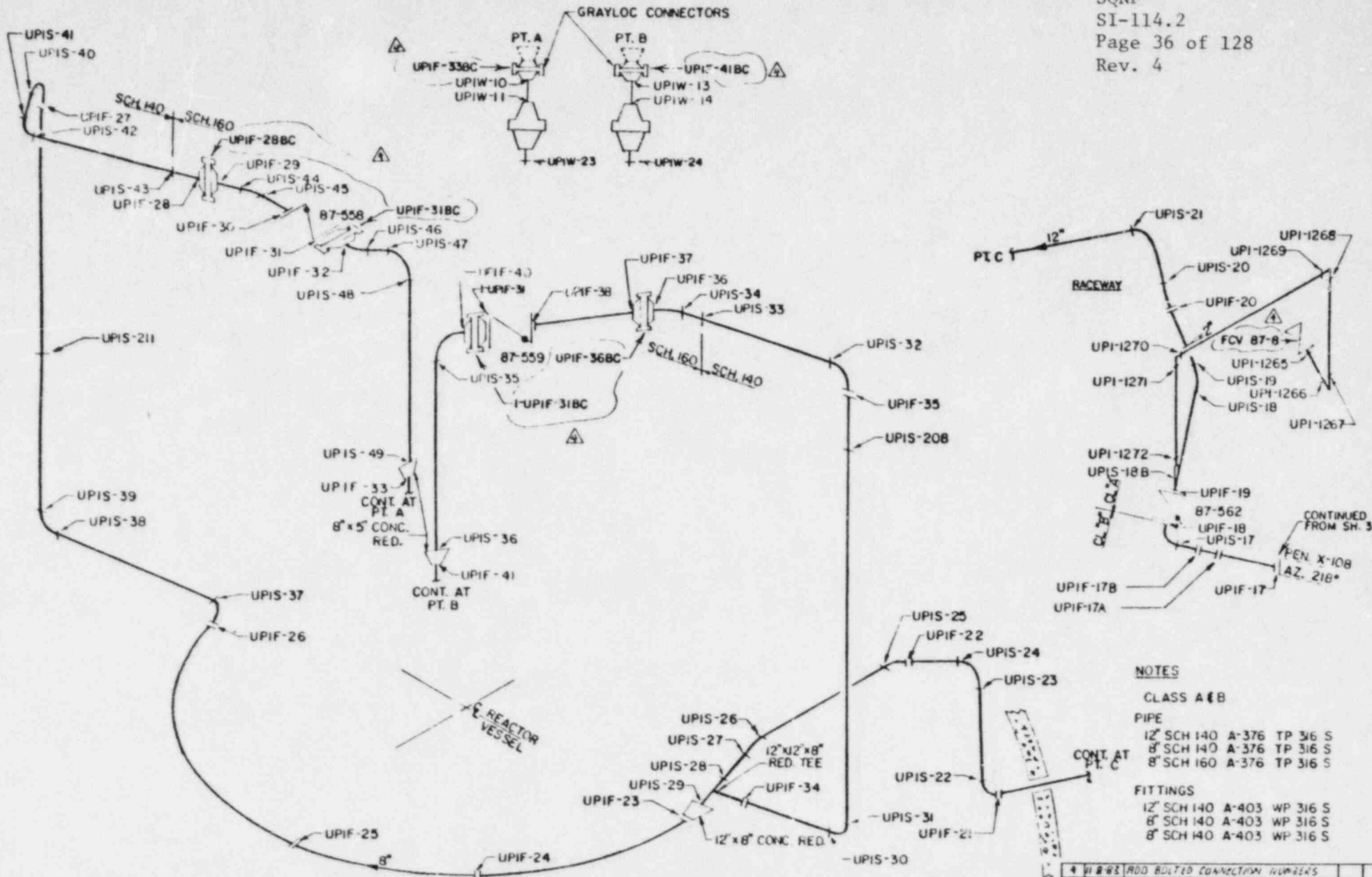


NO.	DATE	REVISIONS	CK'D	APP
1	11-8-83	ADD MANWAY BOLTING REV	ESC	ABB

TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
 UNIT #2  
 BORON INJECTION TANK

SCALE: NTS	SUBMITTED: ESC	APPROVED: ABB	DATE: 7-23-82
DRAWN: ESC	CHECKED: ESC		SHEET 1 OF 1 SHEETS
			ISI-0074-A



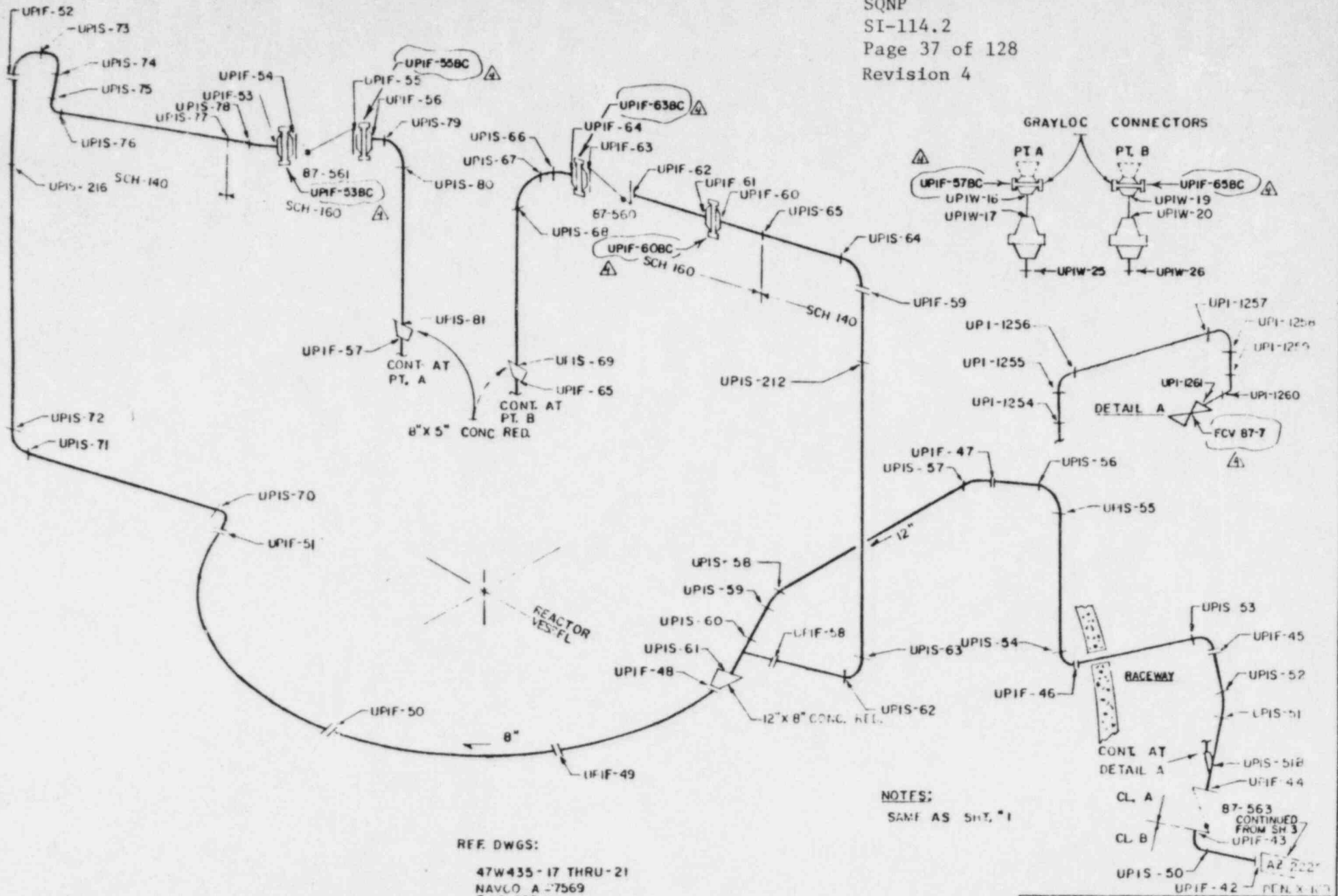
- NOTES**
- CLASS A & B PIPE
  - 12" SCH 140 A-376 TP 316 S
  - 8" SCH 140 A-376 TP 316 S
  - 8" SCH 160 A-376 TP 316 S
  - FITTINGS
  - 12" SCH 140 A-403 WP 316 S
  - 8" SCH 140 A-403 WP 316 S
  - 8" SCH 140 A-403 WP 316 S

4	W 882	ADD BOLTED CONNECTION NUMBERS	
		INDICATE VALVE NUMBER REV	REV
3	4-123	CORRECT WELD NO & ADD IDENTIFICATION NO	REV
2	1-1-2	ADD IDENTIFICATION NO	REV
1	22-81	CORRECT CONNECTION ID	REV
NO	DATE	REV	BY

REF. DRAWING:  
 47W435-17 THRU 21  
 NAVCO A-756B  
 2-UPI-500-IW

TENNESSEE VALLEY AUTHORITY  
 DIVISION OF POWER PRODUCTION  
 SEQUOYAH NUCLEAR PLANT  
 UNIT # 2  
 UPPER HEAD INJECTION  
 1-3  
 ISI-0001-C

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NOTES:  
SAME AS SH. #1

REF DWGS:  
47W435-17 THRU-21  
NAVLO A-7569  
2-UPI-500-1W

NO.	DATE	REVISIONS
2	5-12-81	CORRECT CONFIGURATION
1	2-2-81	CORRECT CONFIGURATION

TENNESSEE VALLEY AUTHORITY  
DIVISION OF POWER PRODUCTION

SEQUOIA NUCLEAR PLANT  
UNIT # 2  
UPPER HEAD INJECTION

NO.	DATE	REVISIONS	BY	CHKD
4	11-8-85	ADD BOLTED CONNECTION NUMBERS AND ORDERED WIRE NUMBERS	...	...
3	9-1-83	CORRECT WELDING AND CONNECTION NUMBERS	...	...

NO.	DATE	REVISIONS	BY	CHKD
4	11-8-85	ADD BOLTED CONNECTION NUMBERS AND ORDERED WIRE NUMBERS	...	...
3	9-1-83	CORRECT WELDING AND CONNECTION NUMBERS	...	...

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REFERENCE DRAWINGS

A-7567  
A-7570  
MATERIAL SPECIFICATIONS  
CLASS B  
PIPE  
SA 376 TP 316  
12" SCH 140  
FITTINGS  
SA 403 WP 316  
12" SCH 140  
FLANGES  
SA 182 F 316

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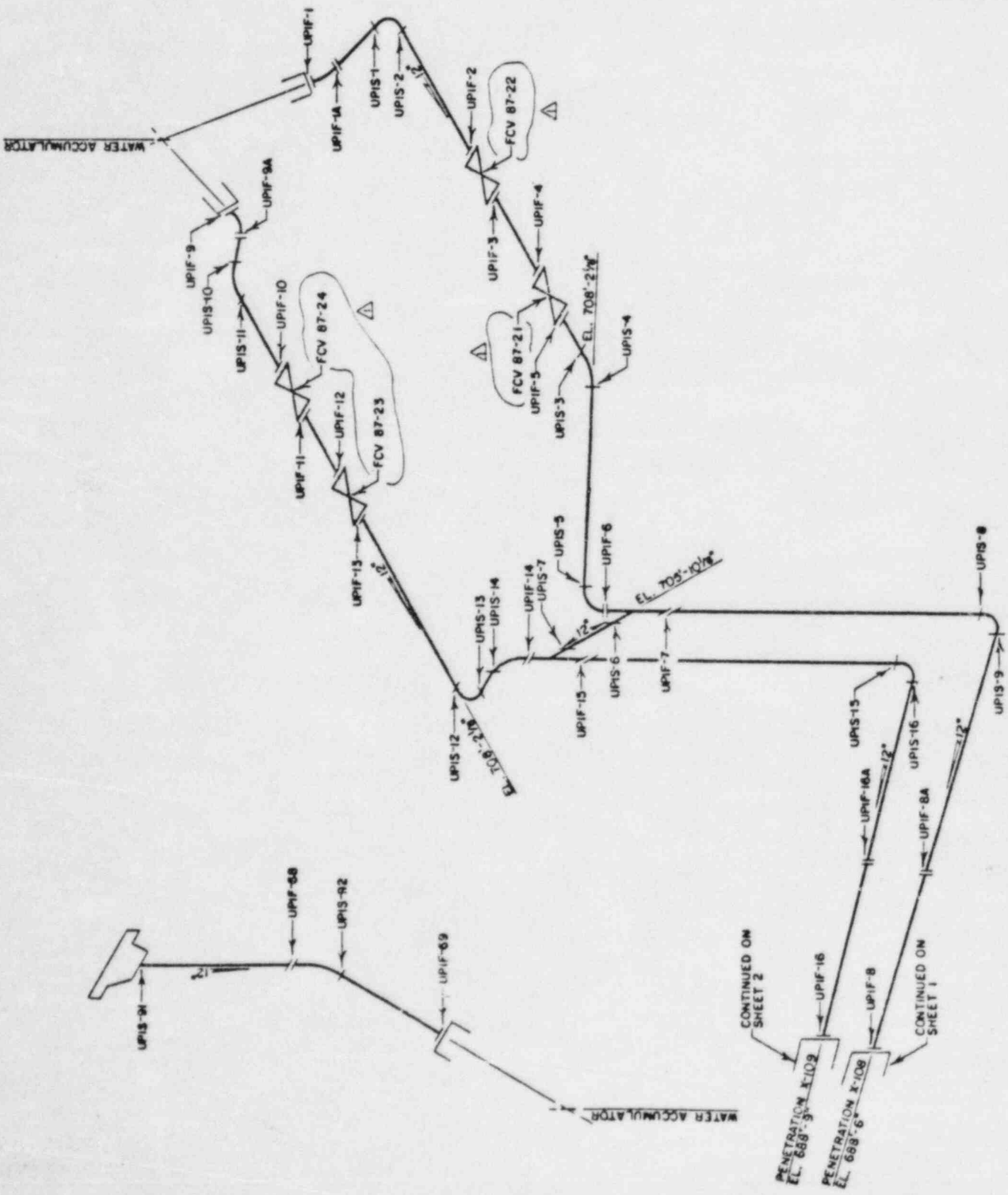
NO.	DATE	REVISIONS	BY	APP'D
1	11-8-81	START WAVE NUMBERS	MM	CD/APP

THURGOOD LAWRENCE  
DIVISION OF SUCCESS POWER

SEQUOYAH NUCLEAR PLANT  
UNIT #2  
UPPER HEAD INJECTION SYSTEM  
WELD LOCATIONS

DATE: 11/8/81  
BY: [Signature]  
APP'D: [Signature]

SI-0001-C



CONTINUED ON SHEET 2

CONTINUED ON SHEET 1

PENETRATION X-109  
EL. 688'-9"

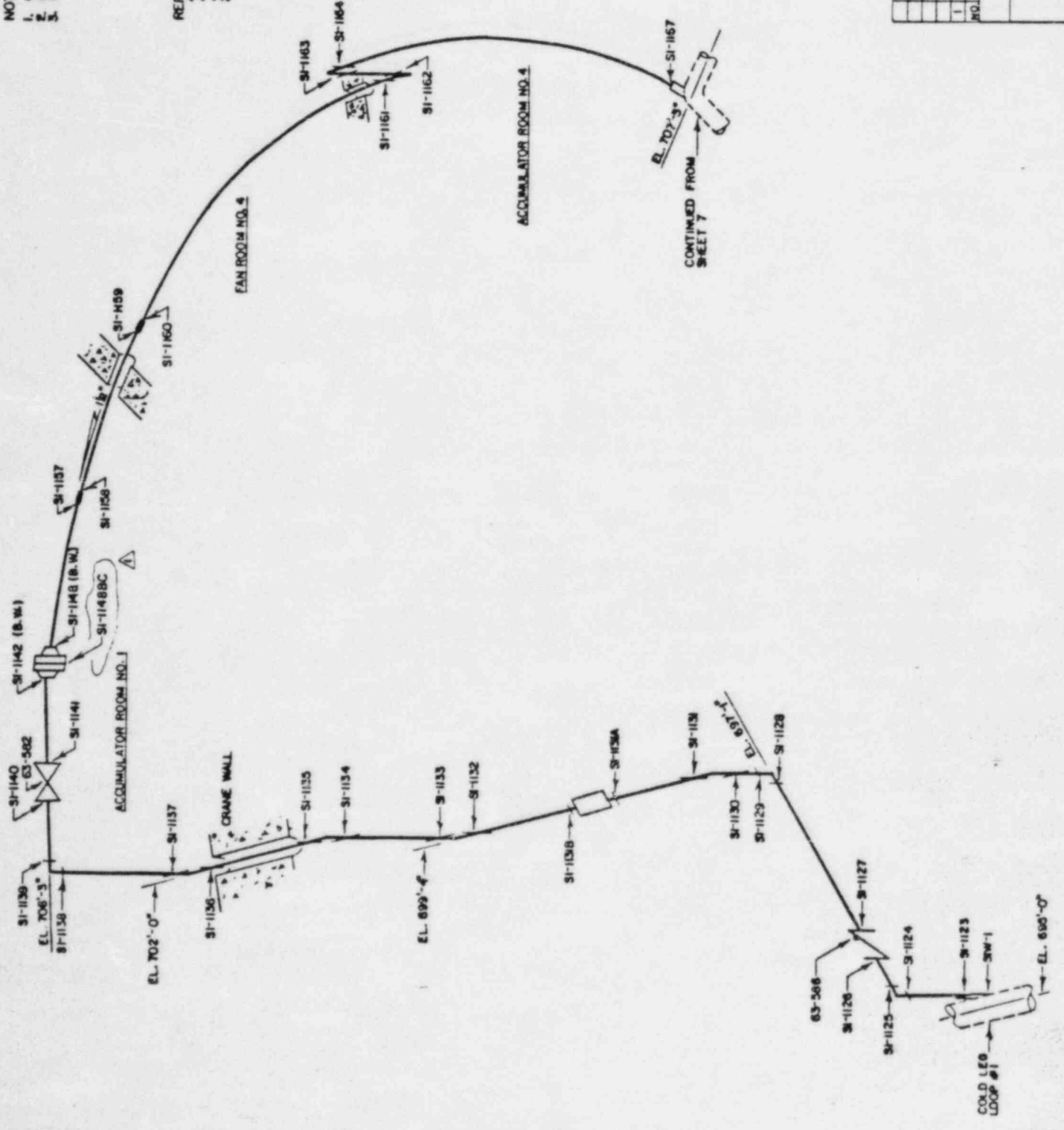
PENETRATION X-108  
EL. 688'-6"

NOTES

1. CLASS A
2. FOR SPECIFICATIONS SEE SHEET 9
3. B. W. DESIGNATES A BUTT WELD

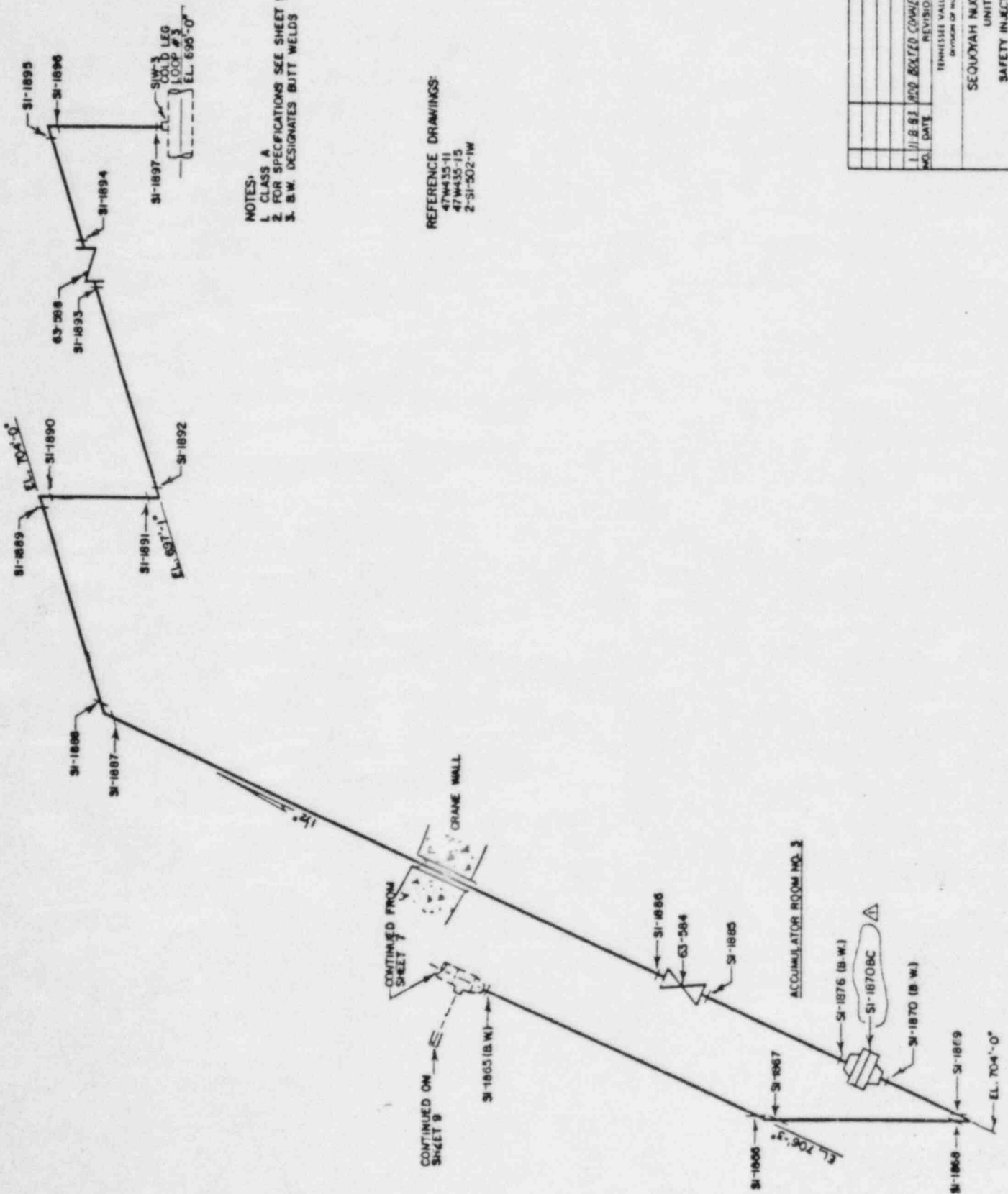
REFERENCE DRAWINGS:

- 47W435-11
- 47W435-2
- 47W435-13
- 2-S-500-1W





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NOTES:  
 1. CLASS A  
 2. FOR SPECIFICATIONS SEE SHEET 9  
 3. B.W. DESIGNATES BUTT WELDS

REFERENCE DRAWINGS:  
 47M435-11  
 47M435-15  
 2-SI-502-1W

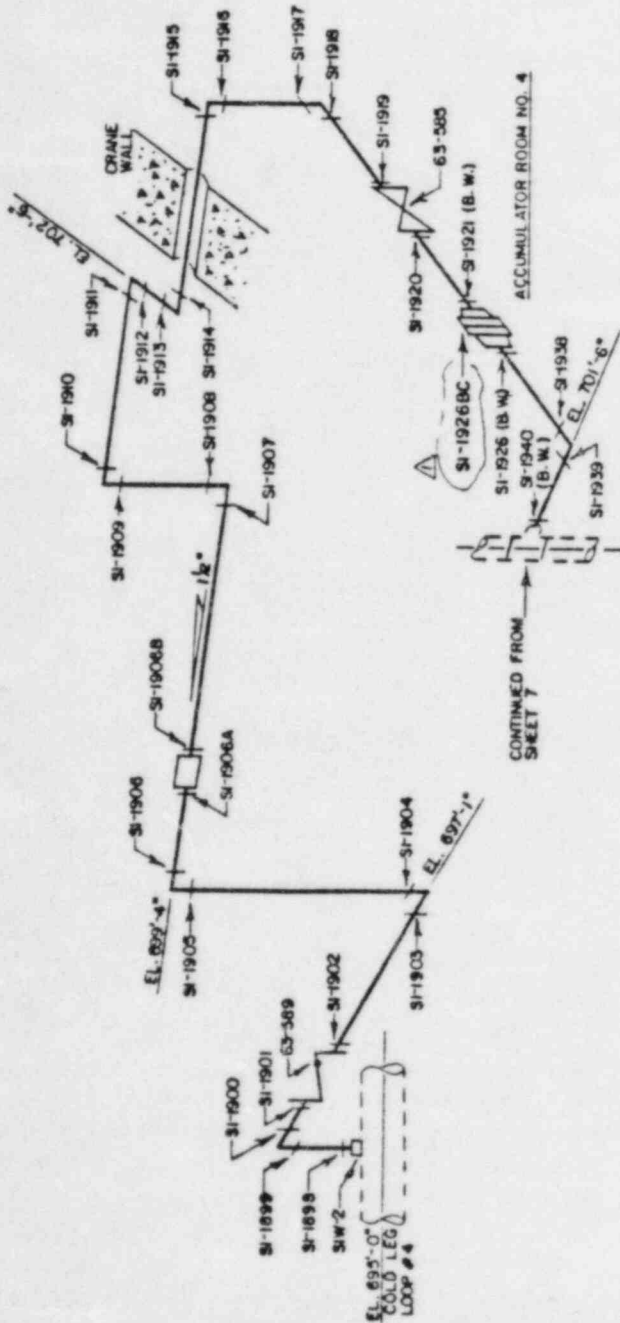
NO.	DATE	REVISIONS	BY
1	11.81.80	REVISED CONNECTION NO. 40V	FLC/AR
			KV DAPP

TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOYAH NUCLEAR PLANT  
 UNIT #2  
 SAFETY INJECTION SYSTEM

151-0002-C  
 10-12-81  
 10-10-82  
 R1

REFERENCE DRAWINGS:  
 47W433-12  
 2-SI-503-NW

- NOTES:  
 1. CLASS A  
 2. FOR SPECIFICATIONS SEE SHEET 9  
 3. B. W. DESIGNATES BUTT WELDS

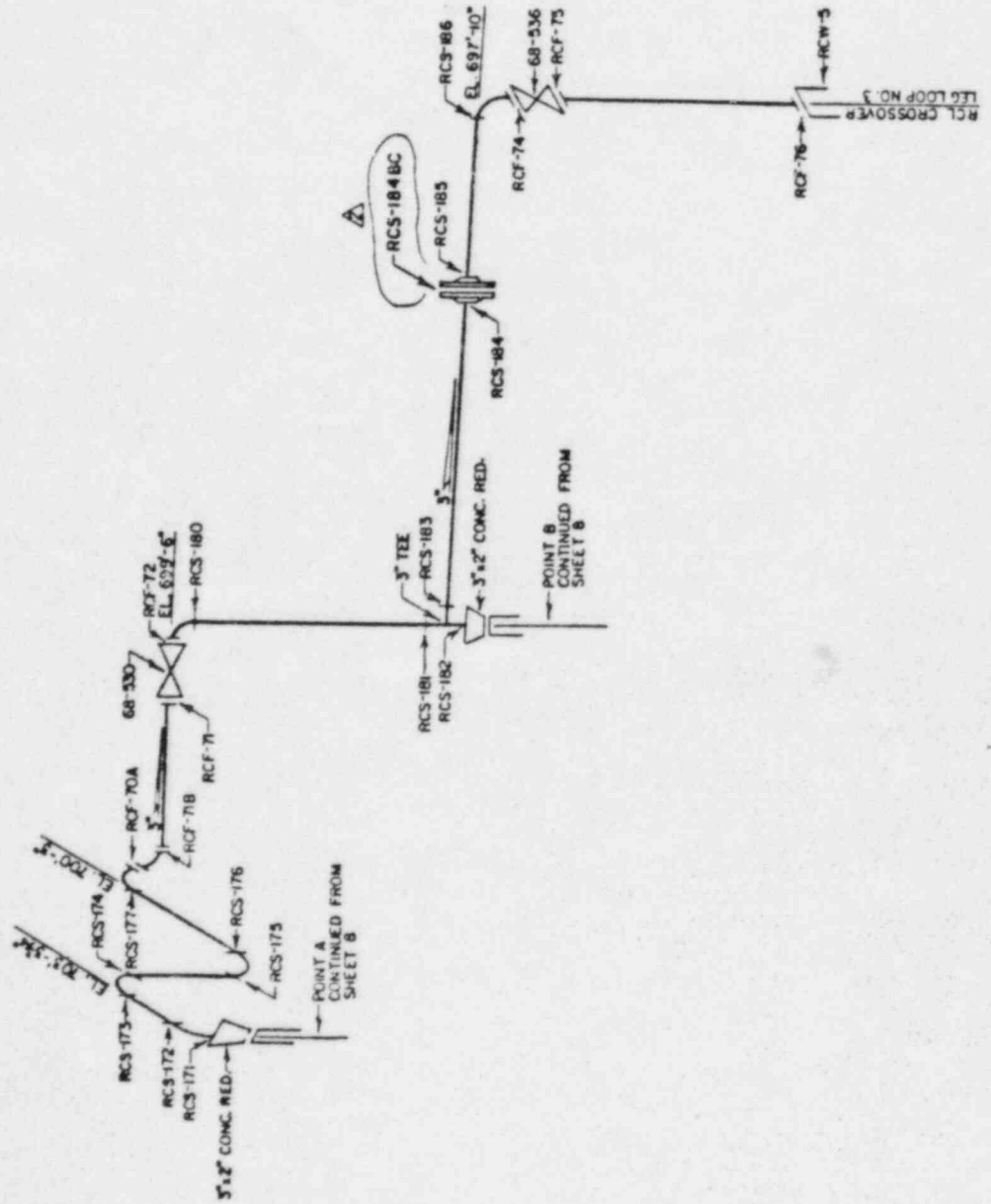


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NO.	DATE	REVISIONS	BY	CHK'D BY
1	8-83	ADD BENTED CONNECTION NO. 20	JK	JK
TENNESSEE VALLEY AUTHORITY DIVISION OF NUCLEAR POWER				
SEQUOIA NUCLEAR PLANT UNIT #2 SAFETY INJECTION SYSTEM				
SCALE	DATE	BY	CHK'D BY	NO.
AS SHOWN	12-19	JK	JK	11
T.V.A. PROJECT NO. 11-12-19 DRAWING NO. SI-114.2 SHEET NO. 49 OF 128				

MATERIAL SPL. CLASS A  
SEE SHEET I  
REFERENCE DRAWINGS  
47M463 SERIES  
NAVDOD A-7566

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NO.	DATE	REVISIONS	BY	CHK'D BY
2	11-8-87	ADD BOLTED CONNECTION AND NO. 2	...	...
1	3-81	CORRECT PWD. NOS.	...	...

TENSILE VALVE AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOIA NUCLEAR PLANT  
 UNIT #2  
 REACTOR COOLANT SYSTEM



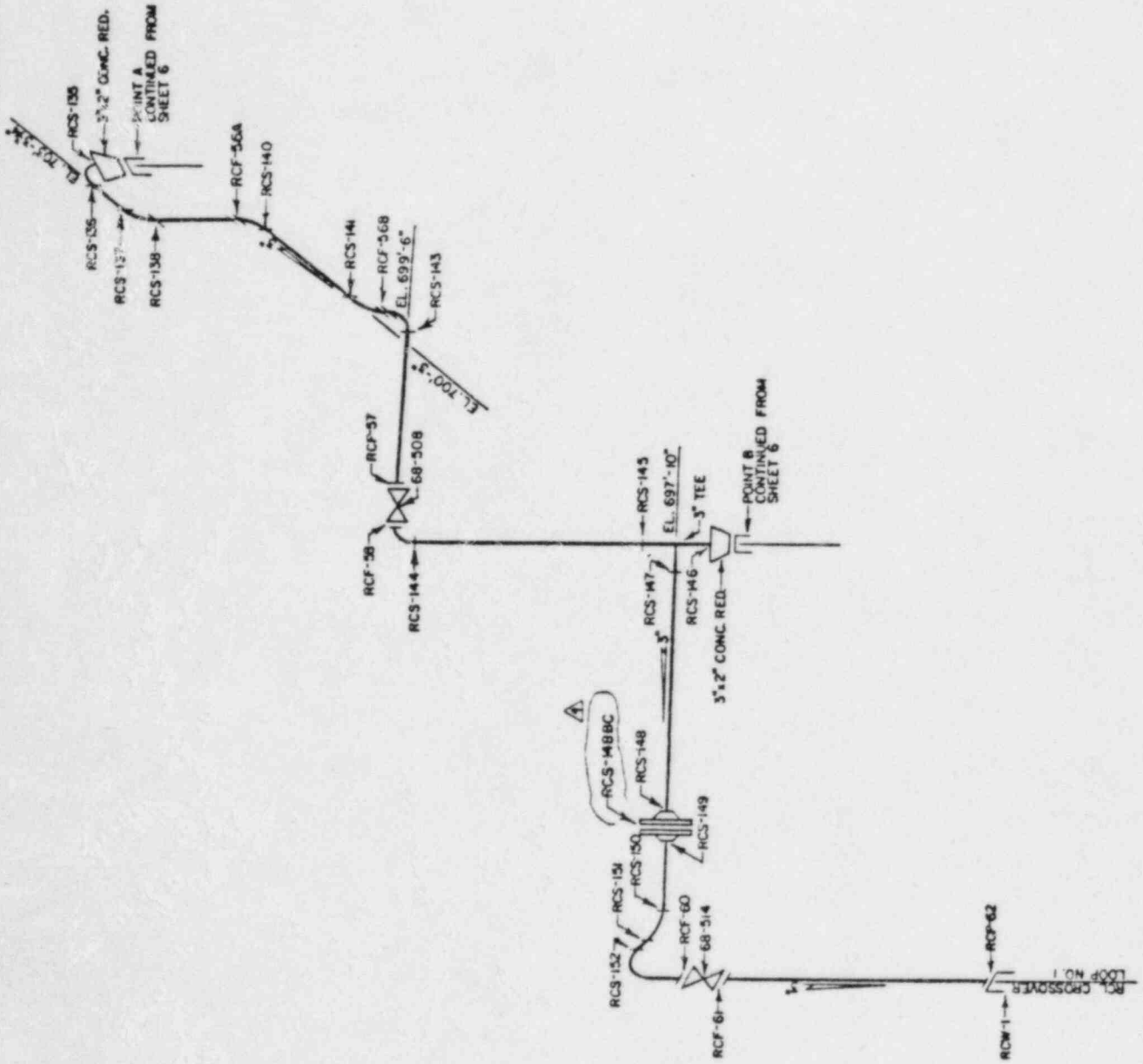




SPECIFICATIONS  
CLASS A  
SEE SHEET 1

REFERENCE DRAWINGS  
47M463 SERIES  
NAVCOD A-7563

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NO.	DATE	REVISIONS	EXTENSION
1	11-8-63	ADD BOXED CONNECTION NO. 419	
2	1-2-64	CONTRACT WELD NO. 419	
3	3-4-64	CONTRACT WELD NO.	
4	1-20-64	ADD WELD NO. 419	

Tennessee Valley Authority	
Division of Nuclear Power	
Sequoyah Nuclear Plant	
Unit #2	
Reactor Coolant System	



SPECIFICATIONS  
CLASS A  
SEE SHEET 1

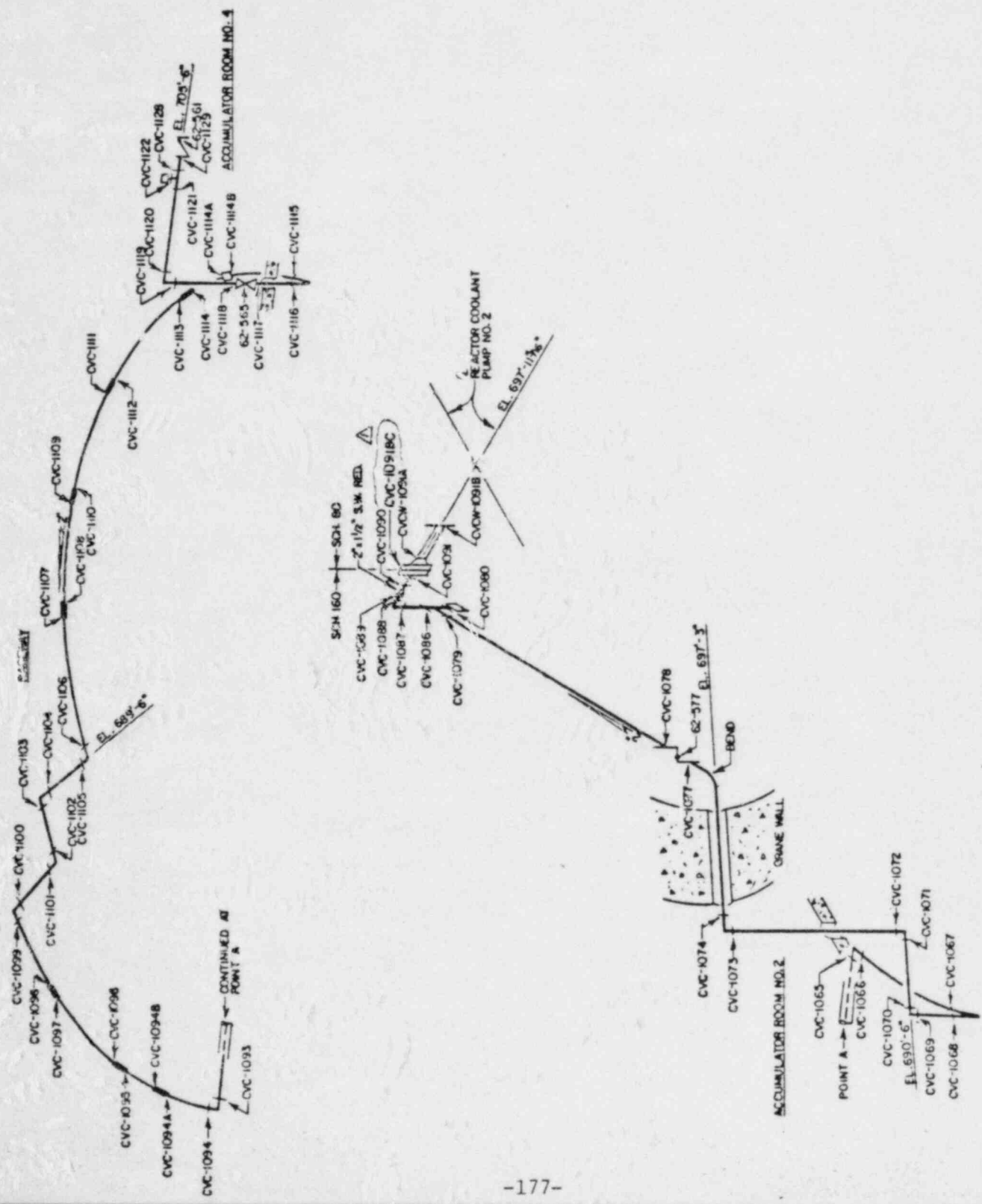
REFERENCE DRAWINGS  
47W405-12  
47W405-14  
47W405-15  
2-CVC-501-1W  
2-CVC-501-2W

NOTES  
1. WELD NO. CVCW-K98A IS BUTT WELDED

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NO.	DATE	REVISIONS	CK'D	APP'D
1	11/8/83	ADD BONDED CONNECTION NO. 41V	CK'D	APP'D

TENNESSEE VALLEY AUTHORITY  
DIVISION OF NUCLEAR POWER  
SEJUNTAH NUCLEAR PLANT  
UNIT #2  
SEAL WATER INJECTOR  
CHEMICAL AND VOLUME  
CONTROL SYSTEM







REFERENCE DRAWINGS

47W433 SERIES  
 A-7540  
 2-SI-506-1W  
 0600102-09-01

PIPE SCHEDULES

CLASS A  
 10" SCH 140  
 6" SCH 160  
 2" SCH 160  
 CLASS B  
 8" SCH 140  
 6" SCH 160

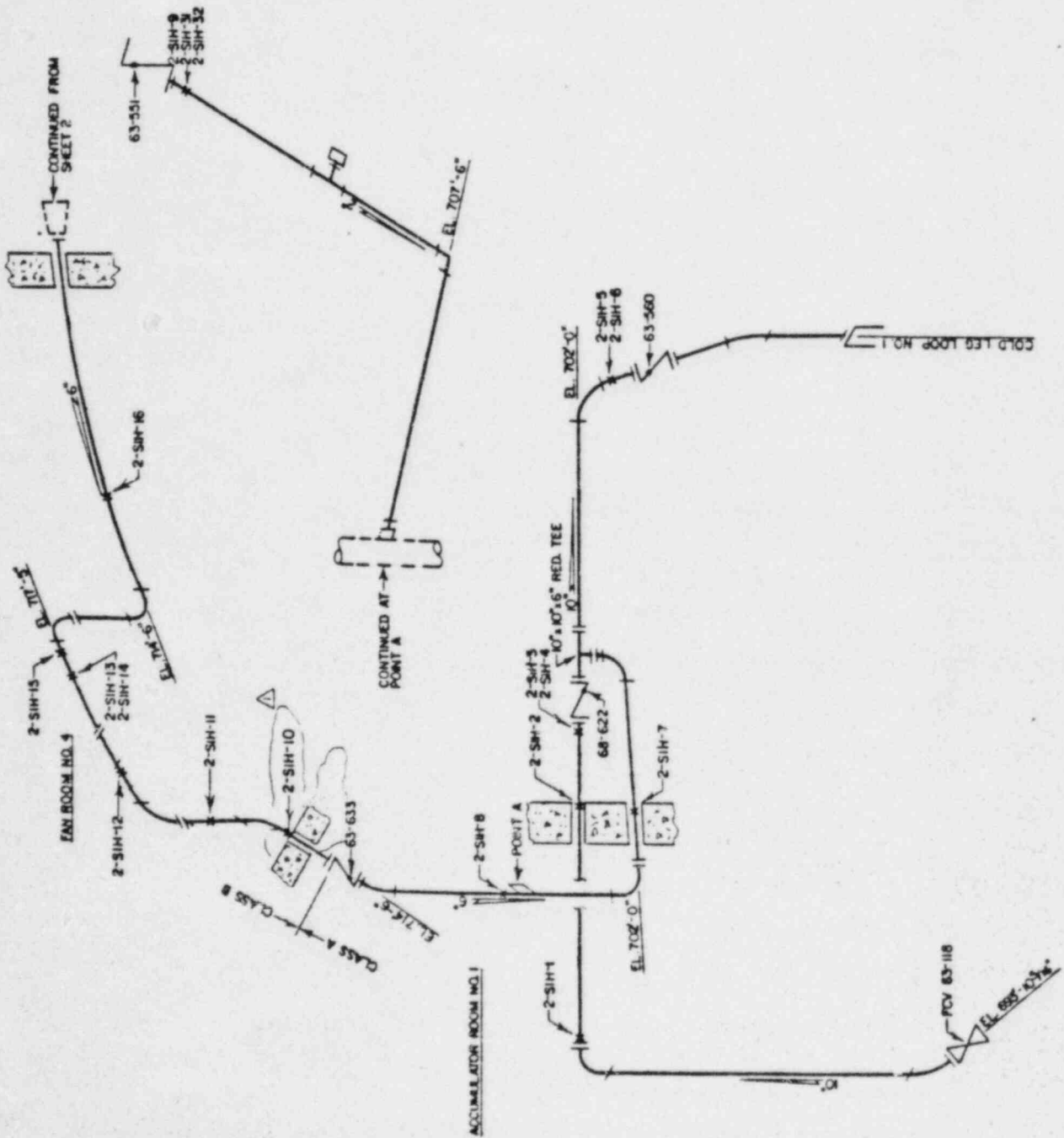
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NO.	DATE	REVISIONS	BY	APP.
1	11-8-93	CORRECT HANGER LOCATIONS	...	...

TECHNICAL AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOIA NUCLEAR PLANT  
 UNIT #2  
 SAFETY INJECTION SYSTEM  
 HANGER LOCATIONS

NSG 0009 C

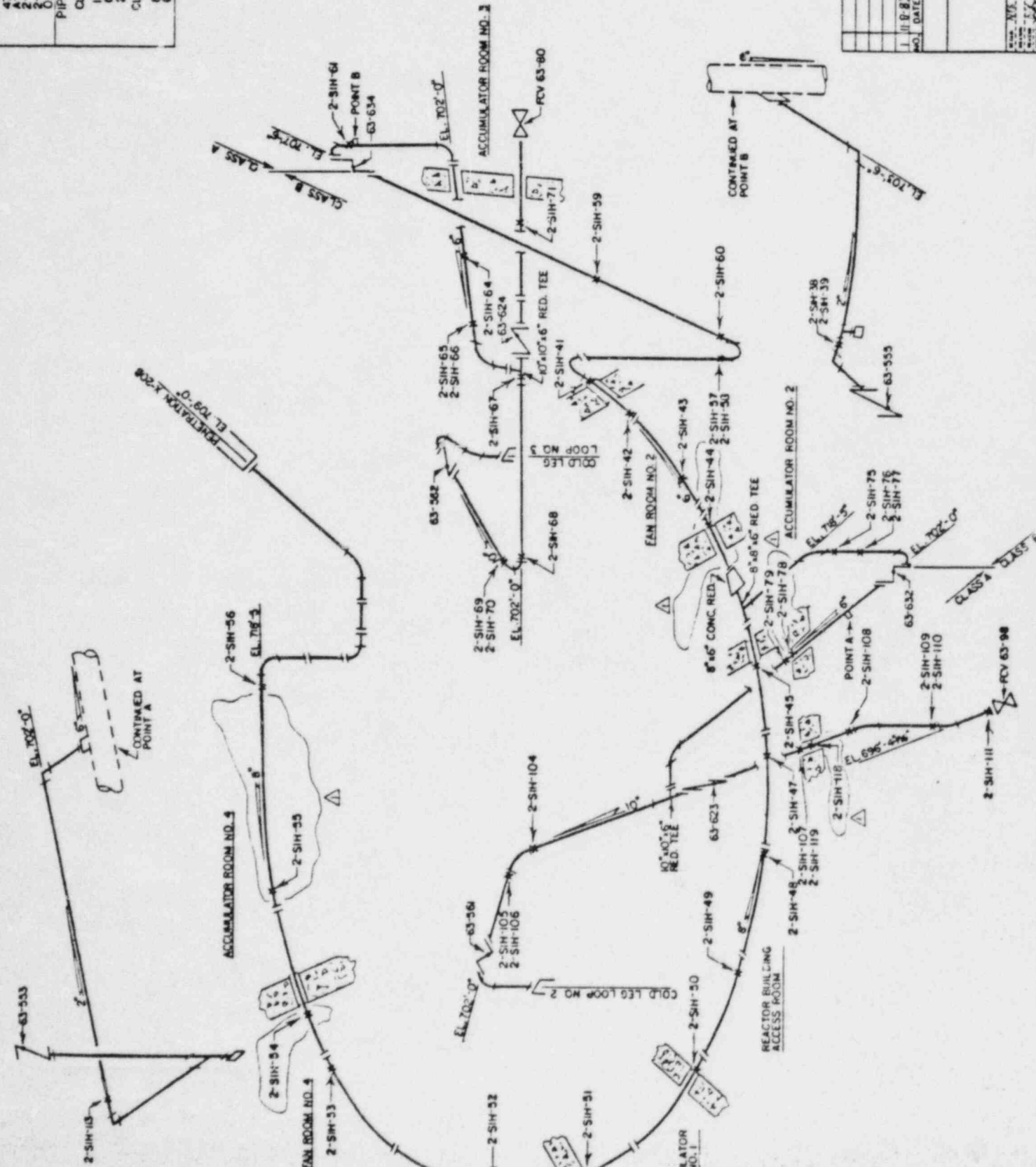


REFERENCE DRAWINGS

- 47M435 SERIES
- A75-41
- 2-SI-507-1W
- 2-SI-510-1W
- 0600102-09-02
- PIPE SCHEDULES
- CLASS A
- 10" SCH 140
- 6" SCH 160
- 2" SCH 160
- CLASS B
- 8" SCH 140
- 6" SCH 160

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NO.	DATE	REVISIONS
1	11-8-83	CORRECT NUMBER LOCATIONS AND
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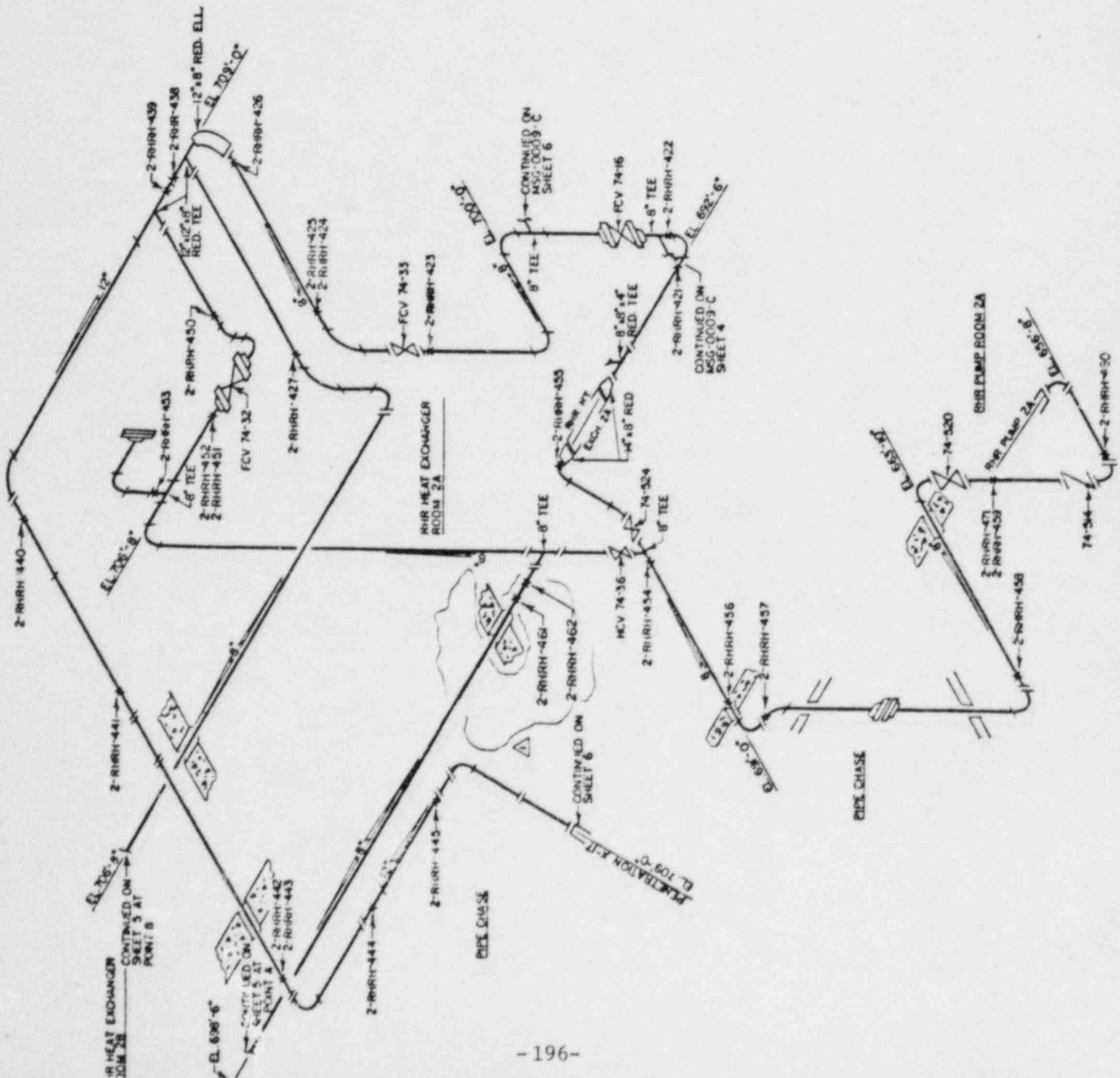
SENOG/AN NUCLEAR PLANT  
 SAFETY INJECTION SYSTEM  
 MAIN/IN LOCATIONS





REFERENCE DRAWINGS
A-7454
47K432-50
PIPE SCHEDULES
CLASS B
12" SCH 40
8" SCH 40S

NO.	DATE	REVISIONS
1	11-8-82	COMPLETE DRAWING FOR REVISIONS BY J. D. APPEL
MEMBER VALLEY AUTHORITY		
DIVISION OF NUCLEAR POWER		
SECOYAH NUCLEAR PLANT		
UNIT #2		
RESIDUAL HEAT REMOVAL SYSTEM		
PIPING LOCATIONS		
SCALE	DATE	BY
AS SHOWN		
PROJECT	NO.	MSG-0003-C-17



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NO.	DATE	REVISIONS	BY	CHKD
1		REWORK FROM WORK AND REWORK FROM REV		

THURGOOD VALLLEY AUTHORITY DIVISION OF NUCLEAR POWER	
SECURITY NUCLEAR PLANT UNIT #2	
RESIDUAL HEAT REMOVAL SYSTEM HANGEN LOCATIONS	

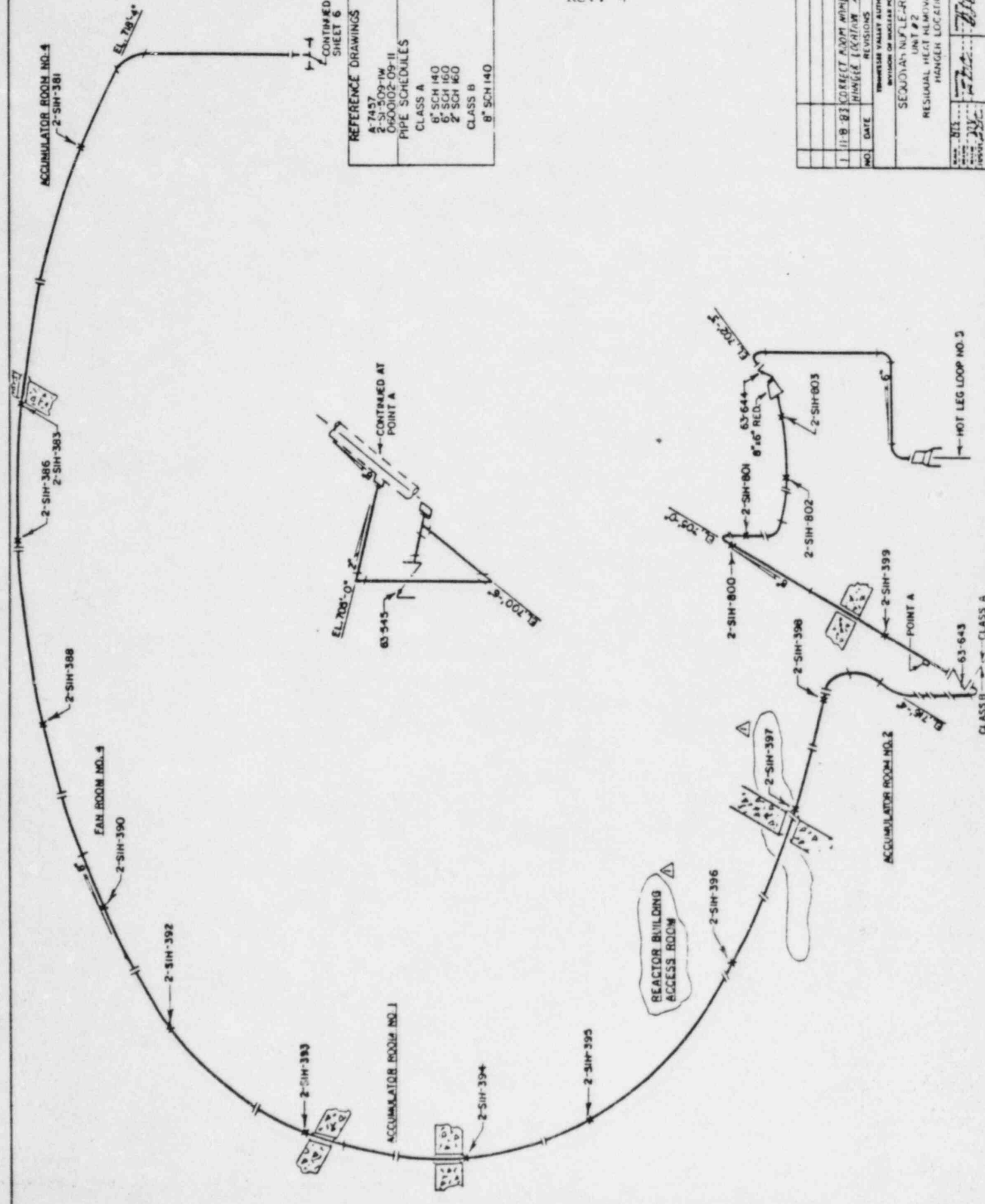
  

DATE	BY	CHKD
10/11/83	W. J. ...	...
10/11/83	...	...
10/11/83	...	...

CONTINUED FROM SHEET 6

REFERENCE DRAWINGS

A-7457
2-SI-509-1W
0600102-09-11
PIPE SCHEDULES
CLASS A
8" SCH 140
6" SCH 160
2" SCH 160
CLASS B
8" SCH 140



REFERENCE DRAWINGS

ISI-0007-C  
47K435-51  
PIPE SCHEDULES  
CLASS B  
12" SCH 40

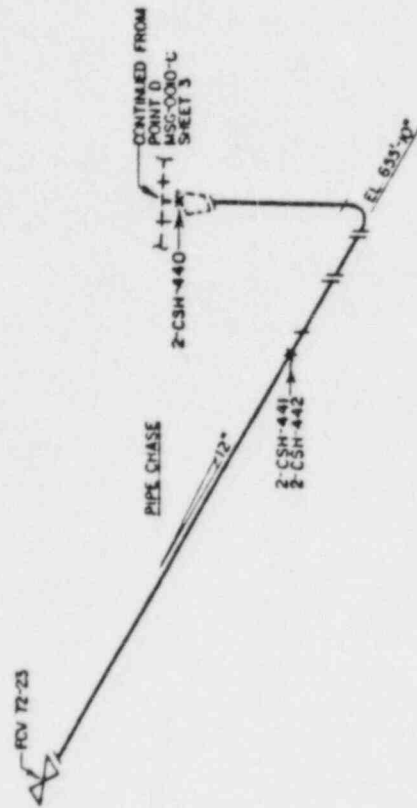
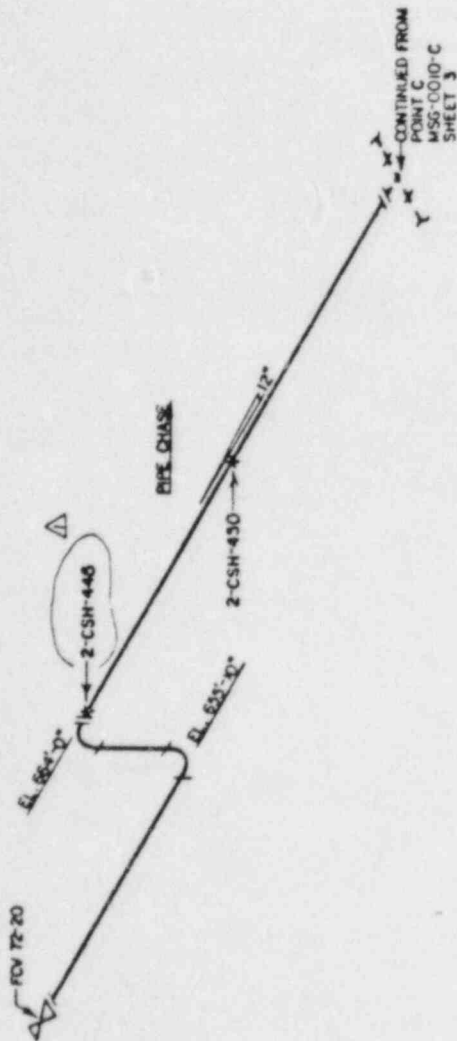
SQNP  
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NO	DATE	REVISIONS	BY	CHK
1	11-08-91	CONVERT FROM 12" SCH 40 TO 12" SCH 40	MSG	MSG

THUNDER VALLEY AUTHORITY  
DIVISION OF NUCLEAR POWER

SEDOYAH NUCLEAR PLANT  
UNIT # 2  
CONTAINMENT SPRAY SYSTEM  
HANGER LOCATIONS

Scale: AS SHOWN  
Drawing No: 101-114.2-001  
Sheet No: 101-114.2-001-C



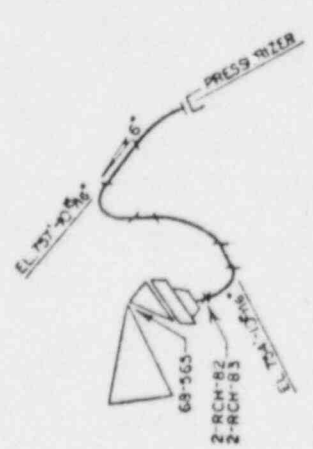
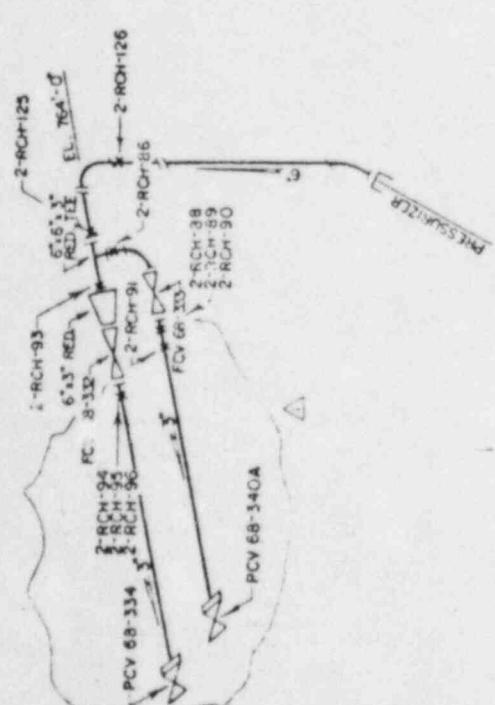
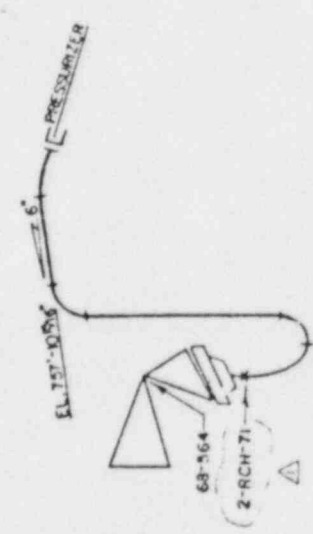
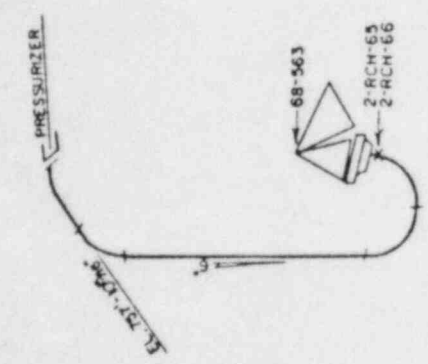
REFERENCE DRAWINGS  
 ISI-0013-C  
 000002-13-03  
 PIPE SCHEDULES  
 CLASS A  
 6" SCH 160  
 5" SCH 160

SONP  
 SI-114.2  
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NO.	DATE	REVISIONS	GROUP
1	11/8/83	CORRECT PIPING DIMENSIONS AND OMITTED ANGLES PER REVISED DRAWING	CLASS A

TRINITY VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEJOUVAH NUCLEAR PLANT  
 UNIT # 2  
 REACTOR COOLANT SYSTEM  
 HANGER LOCATIONS



REFERENCE DRAWINGS

ISI-0001-C  
 0600102-15-03  
 0600152-15-01  
 0600152-15-02

PIPE SCHEDULES

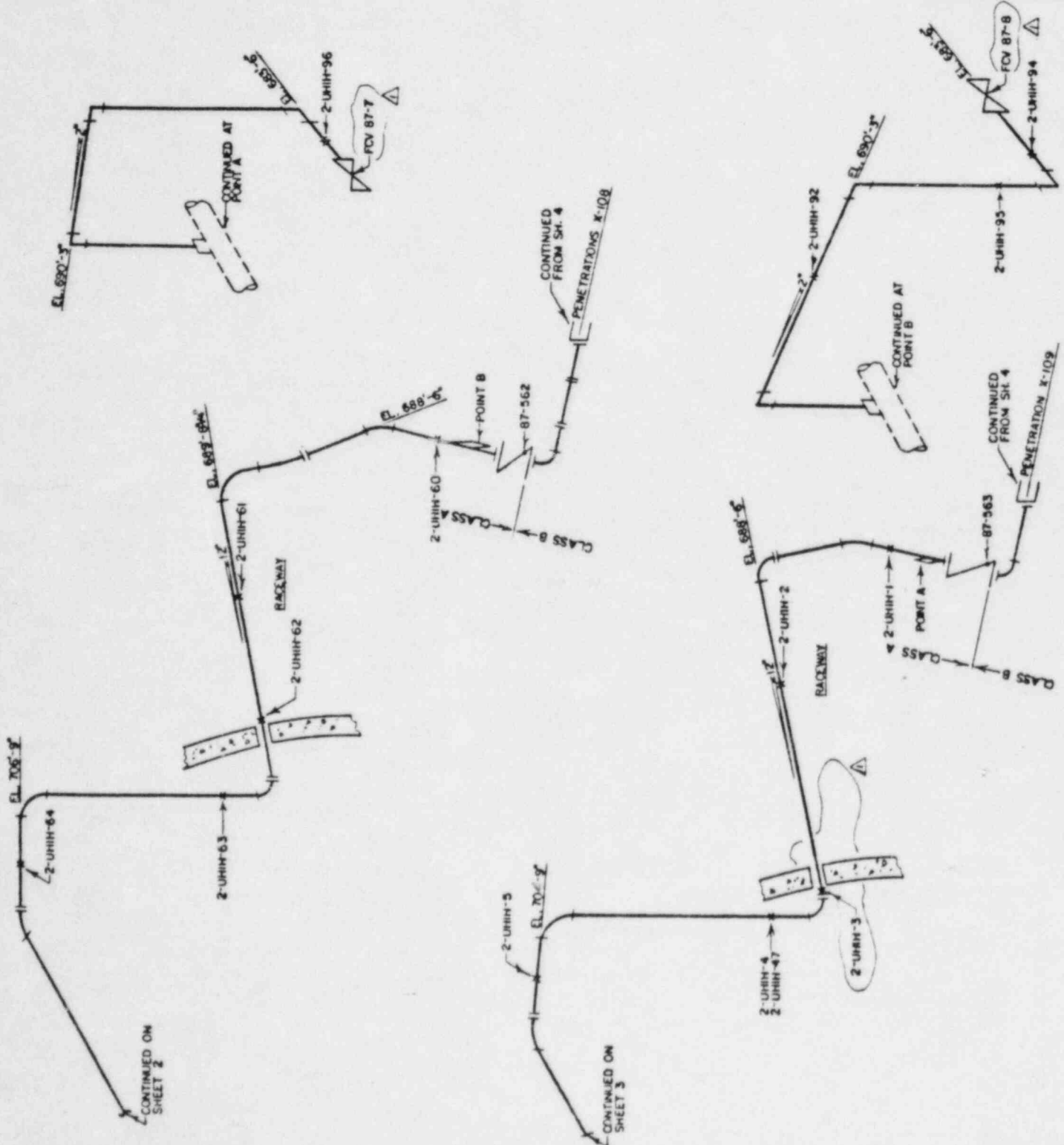
CLASS A  
 12" SCH 140  
 2" SCH 160  
 CLASS B  
 12" SCH 140

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NO.	DATE	REVISIONS	BY	APP'D
1	11/8/95	CORRECT HANGER LOCATIONS AND VALVE NUMBERS	...	...

SEQUOIA VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOIA NUCLEAR PLANT  
 UNIT #2  
 UPPER HEAD INJECTION SYSTEM  
 HANGER LOCATIONS

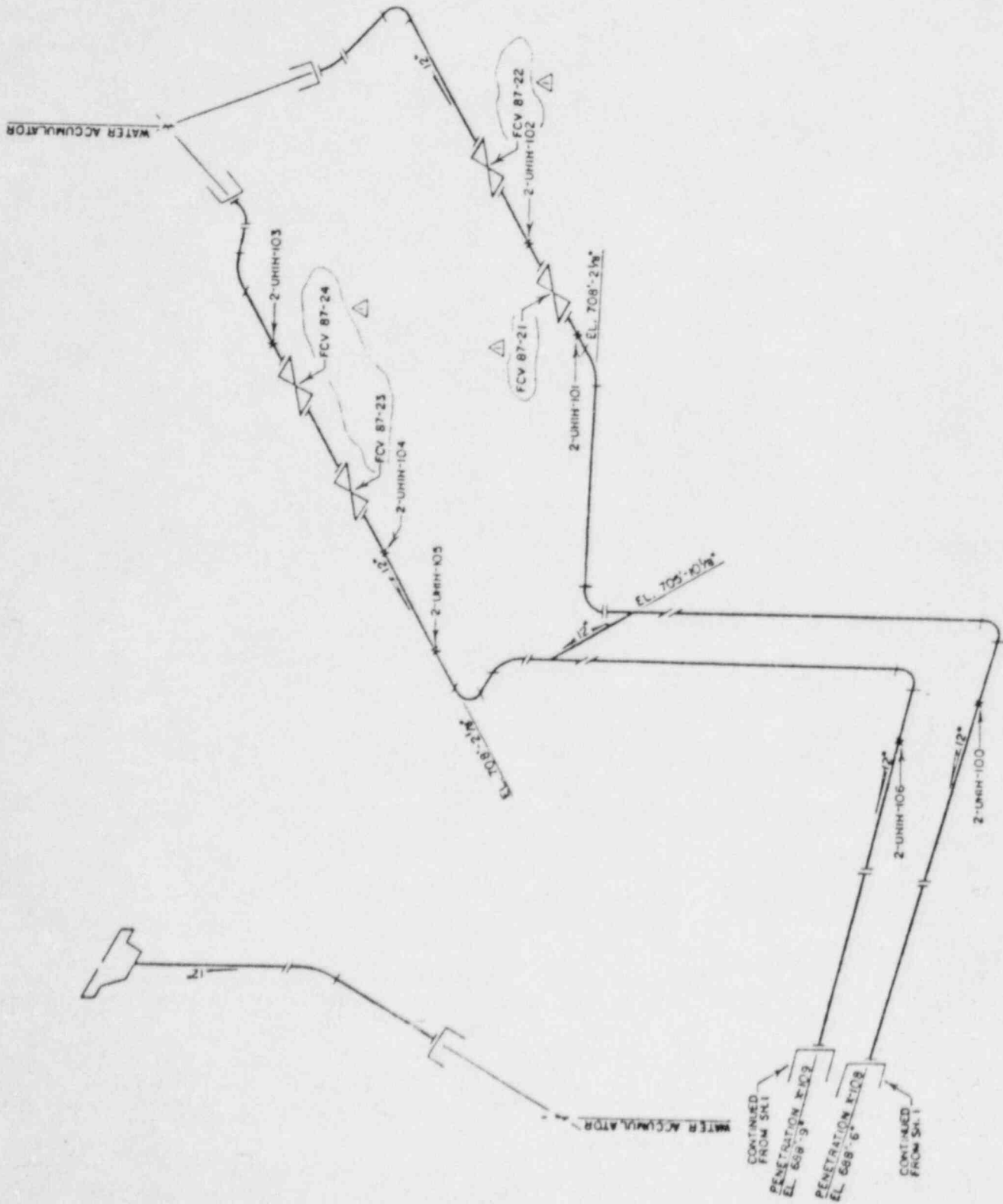


REFERENCE DRAWING  
 SI-0001-C  
 0600152-15-03  
 0600152-15-04  
 PIPE SCHEDULES  
 CLASS B  
 12" SCH 40

NO	DATE	REVISIONS	BY	APP
1	8-23	CONNECT PIPE NUMBERS #14	STL	CD
2				
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SEQUOIA VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOIA NUCLEAR PLANT  
 UNIT #2  
 UPPER HEAD INJECTION SYSTEM  
 HANGER LOCATIONS

DATE: 8/27/83  
 DRAWN BY: STL  
 CHECKED BY: STL  
 PROJECT NO: MSG-0004-C-11



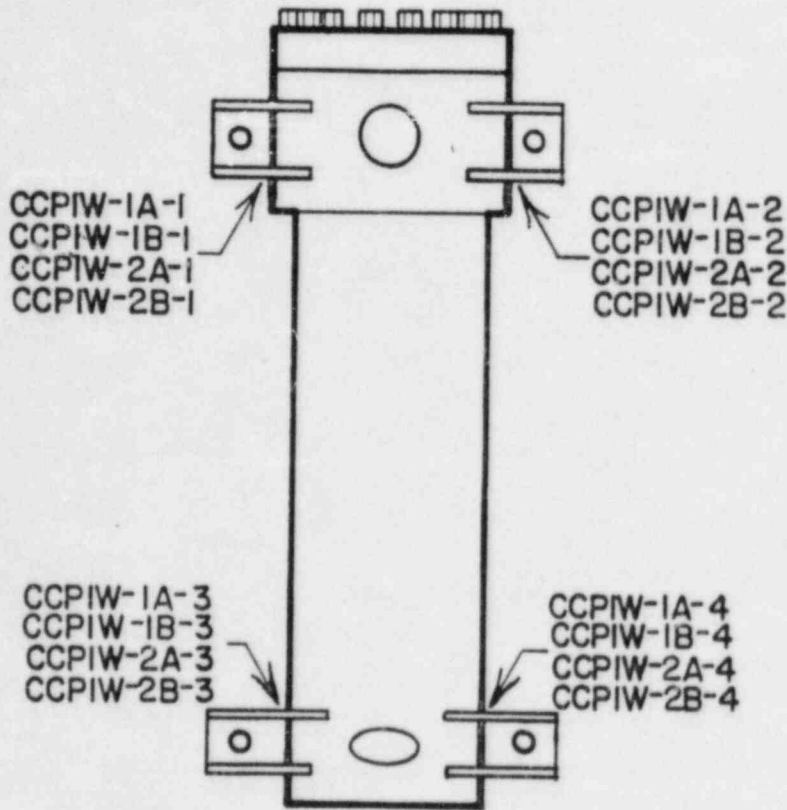
SQNP  
 SI-114.2  
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REFERENCE DRAWINGS

FC-456I9

NOTES

1. THE PUMP FEET ARE BOLTED TO A COMMON SUPPORT.
2. PUMP SUPPORT NUMBERS  
 CCPH-1A-A  
 CCPH-1B-B  
 CCPH-2A-A  
 CCPH-2B-B
3. THE PUMP FEET ARE INTERGALLY WELDED TO THE CASING.



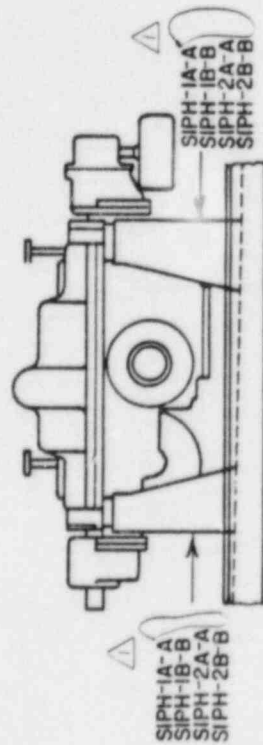
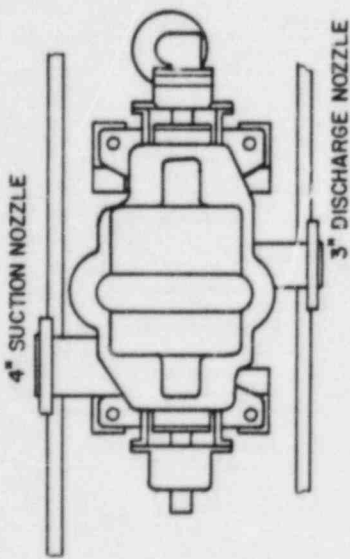
TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
 UNITS #1 & #2  
 CENTRIFUGAL CHARGING PUMP SUPPORTS

SCALE: <i>NIS</i>	SUBMITTED	APPROVED	DATE <i>3-27-84</i>
DRAWN <i>REV</i>	<i>EDC</i>	<i>AFB</i>	SHEET OF SHEETS
TRACED			ISI-0146-A
CHECKED <i>EDC</i>			

REFERENCE DRAWINGS  
 FC-45648-4  
 TVA SAFETY CLASS B

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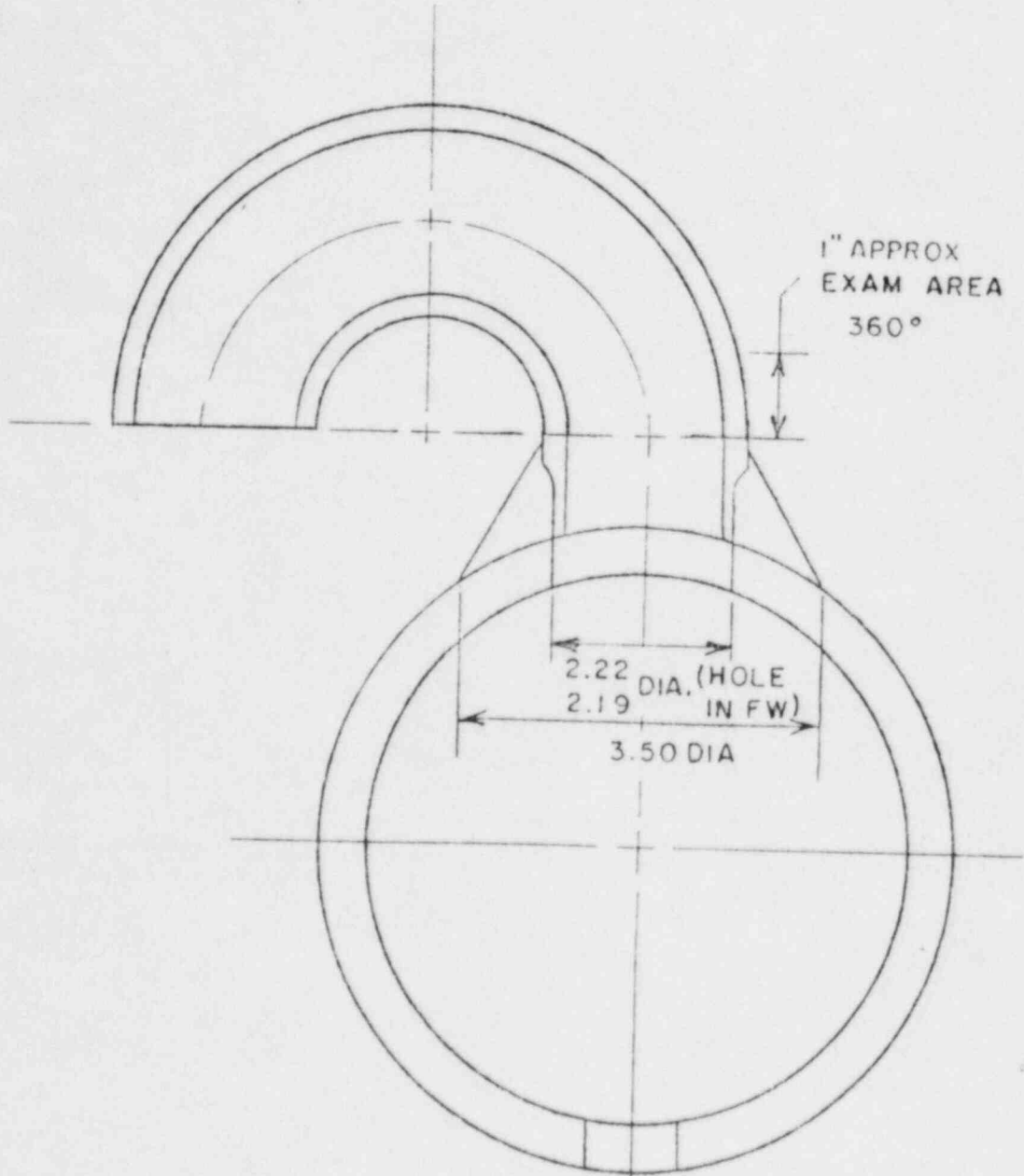


NO. DATE	1 7-25-84	DELETED NUMBERS	44	CKD/APP
REVISIONS				
TENNESSEE VALLEY AUTHORITY DIVISION OF NUCLEAR POWER				
SEQUOYAH NUCLEAR PLANT UNITS #1 & #2 SAFETY INJECTION PUMP SUPPORTS				
SCALE	AS SHOWN	DATE	3-30-84	REVISED
BY	ERL	APPROVED	JAB	REVISION
CHKD		DATE	1-08	BY
				ISI-0147-B

R/



SQNP  
 SI-114.2  
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TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
 UNITS 1 AND 2 STEAM  
 GENERATOR FEEDRINGS J-TUBES

MATERIAL  
 2" SCH 80, N.W. 0.218"

REFERENCE DRAWING  
 W 1102J70

SCALE: NONE

DRAWN *Jca*

TRACED

CHECKED *ETC*

SUBMITTED

*ETC*

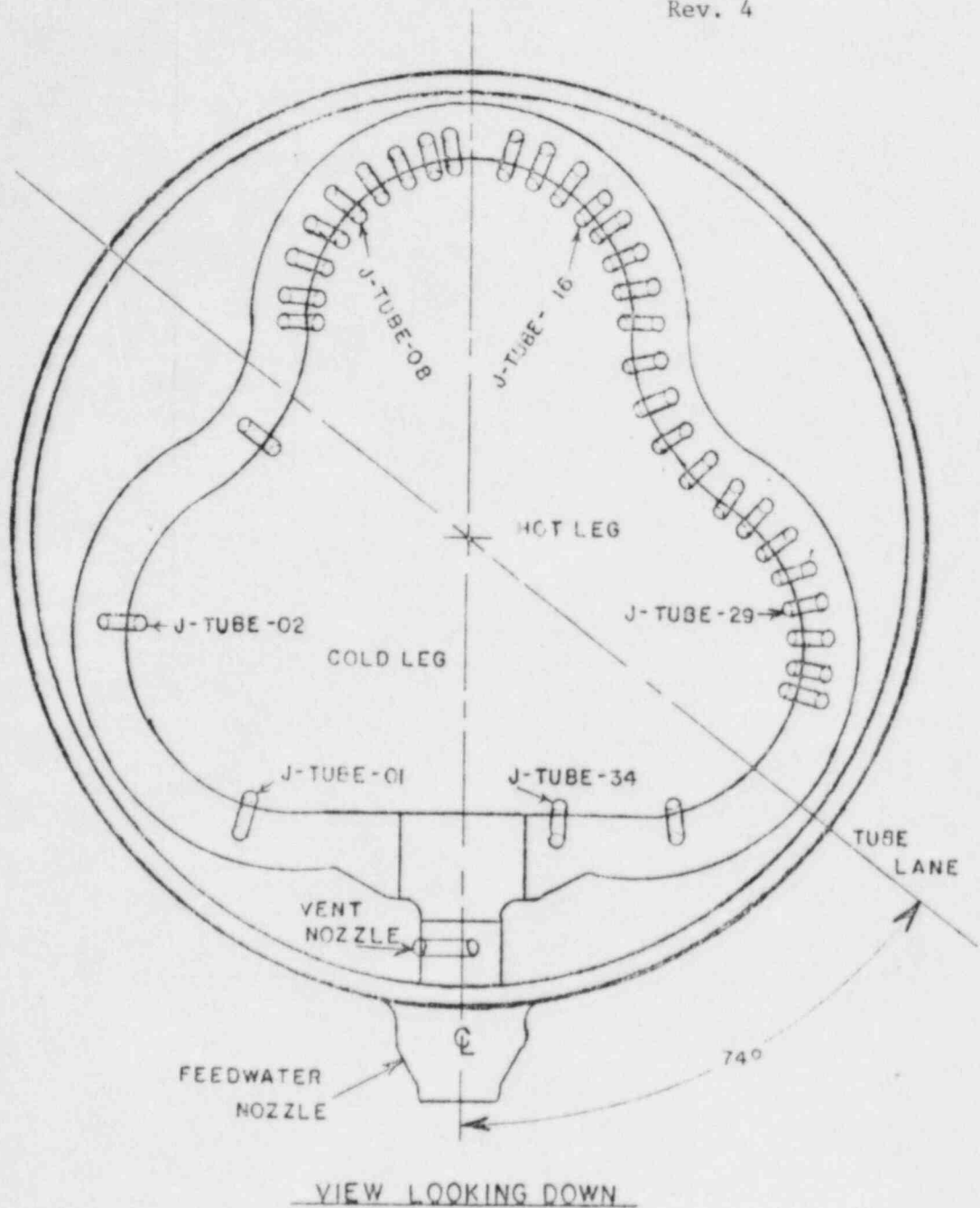
APPROVED

*ABB*

DATE 1-16-84

SHEET 1 OF 3 SHEETS

ISI-0134-A



NOTES:

1. DWG APPLIES TO:  
 UNIT-1, STM. GENS. 183  
 UNIT-2, STM. GENS. 284
2. 34 J-TUBES PER STM. GEN.

TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
 UNITS-1&2 STEAM GENERATOR  
 FEEDWATER RING J-TUBES

REFERENCE DRAWING  
 W1103J64

SCALE: NONE  
 DRAWN  
 TRACED *W.C.*  
 CHECKED

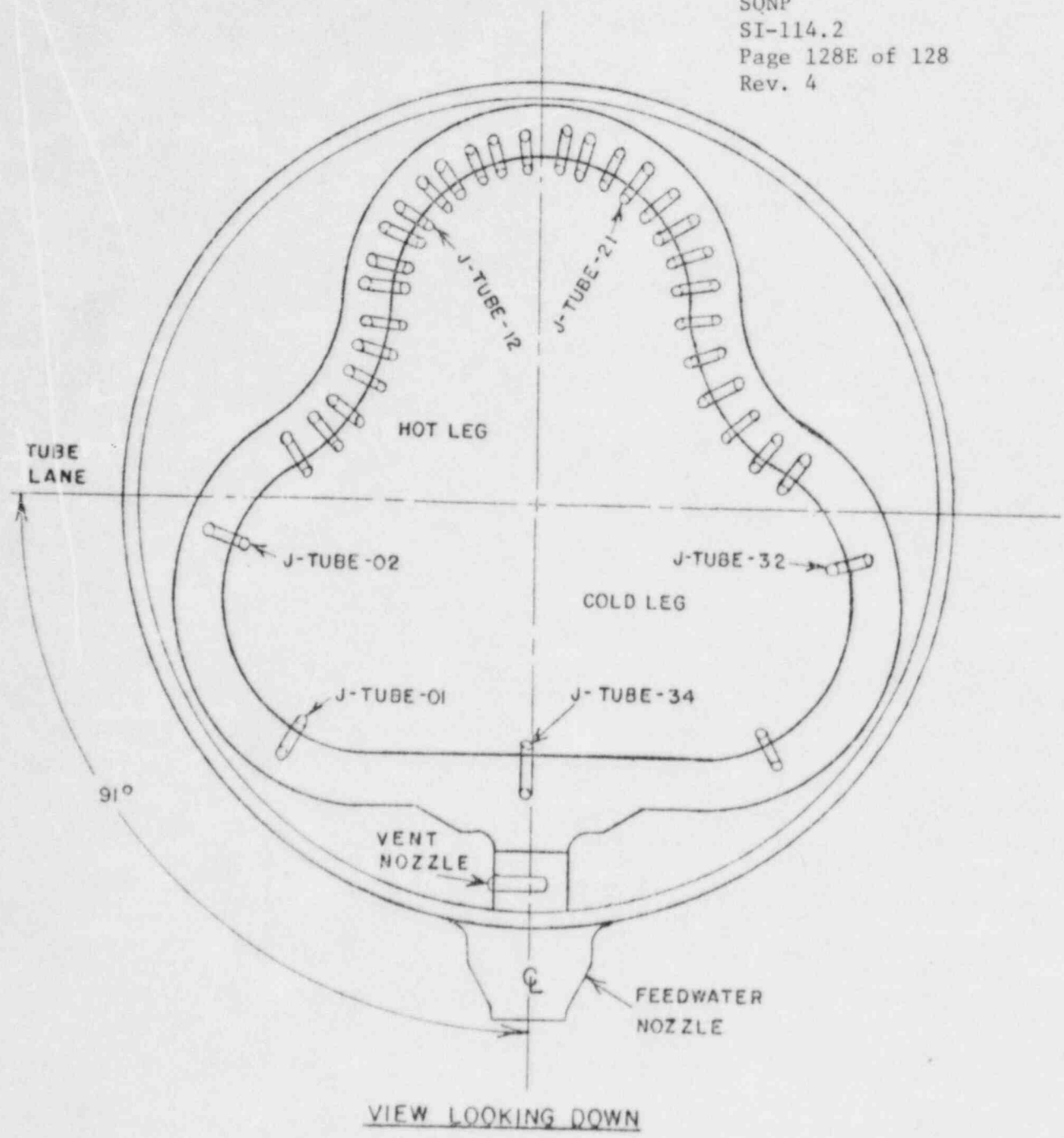
SUBMITTED

APPROVED

DATE 1-27-64

SHEET 2 OF 3 SHEETS

ISI-0134-A



NOTES:  
 1. DWG APPLIES TO:  
 UNIT-1, STM.GENS.2&4  
 UNIT-2, STM.GENS.1&3  
 2. 34 J-TUBES PER STM.GEN.

TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOYAH NUCLEAR PLANT  
 UNITS-1&2 STEAM GENERATOR  
 FEEDWATER RING J-TUBES

REFERENCE DRAWING  
 W1103J63

SCALE: NONE	SUBMITTED	APPROVED	DATE 1-27-84
DRAWN			SHEET 3 OF 3 SHEETS
TRACED <i>W.R.</i>			
CHECKED			ISI-0134-A





REFERENCE DRAWINGS

474406-77  
474555-59

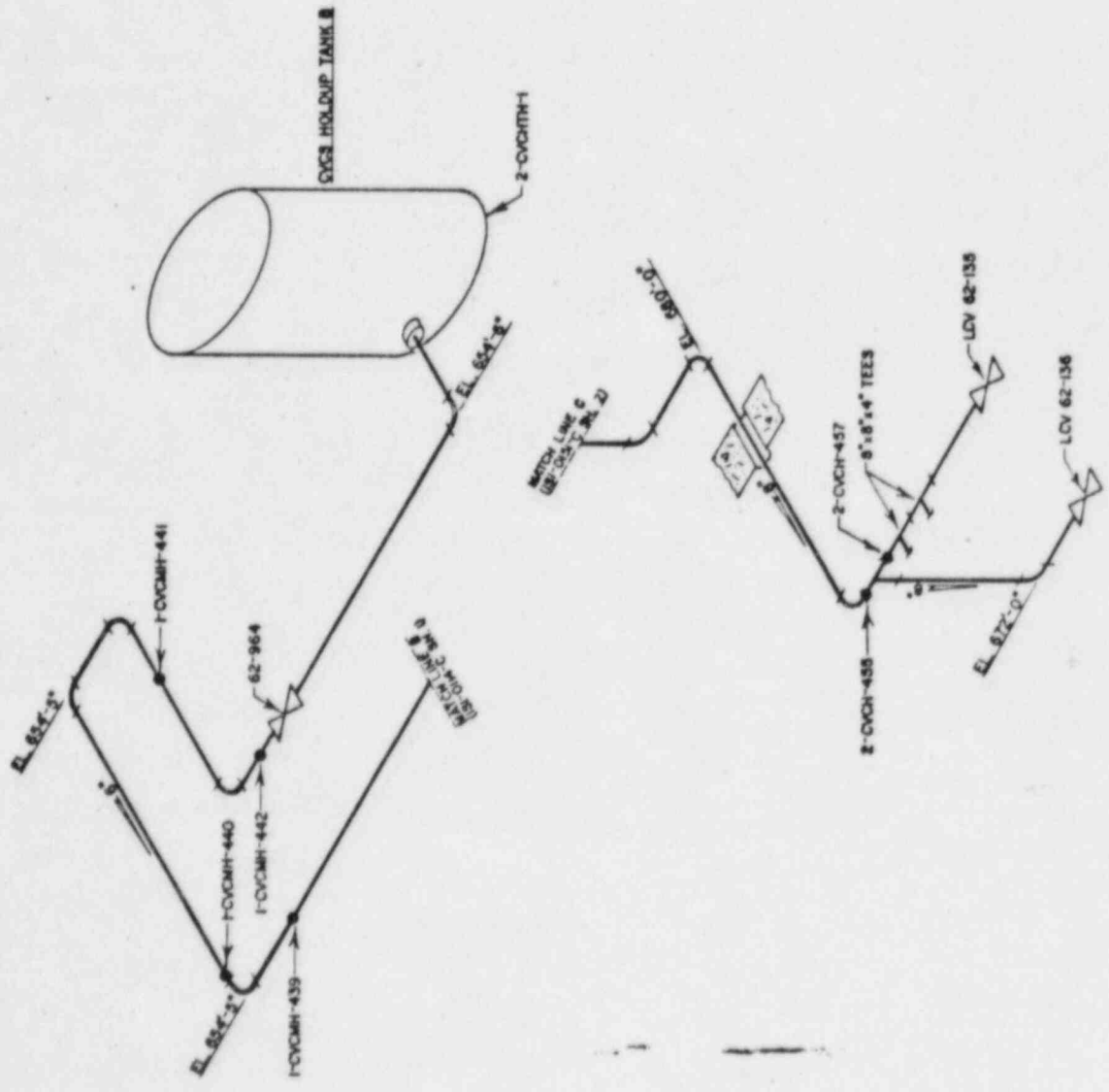
LEGEND

- RIGID SUPPORT
- TVA SAFETY CLASS C

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Rev. 4

NO.	DATE	REVISIONS	BY	APP.

TENNESSEE VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER  
 SEQUOYAH NUCLEAR PLANT  
 UNIT # 2  
 CHEMICAL AND VOLUME CONTROL SYSTEM  
 SUPPORT LOCATIONS

















REFERENCE DRAWINGS

- 47K464-33
- 47K464-34
- 47K464-55
- 47K464-56
- 47K464-57

LEGEND

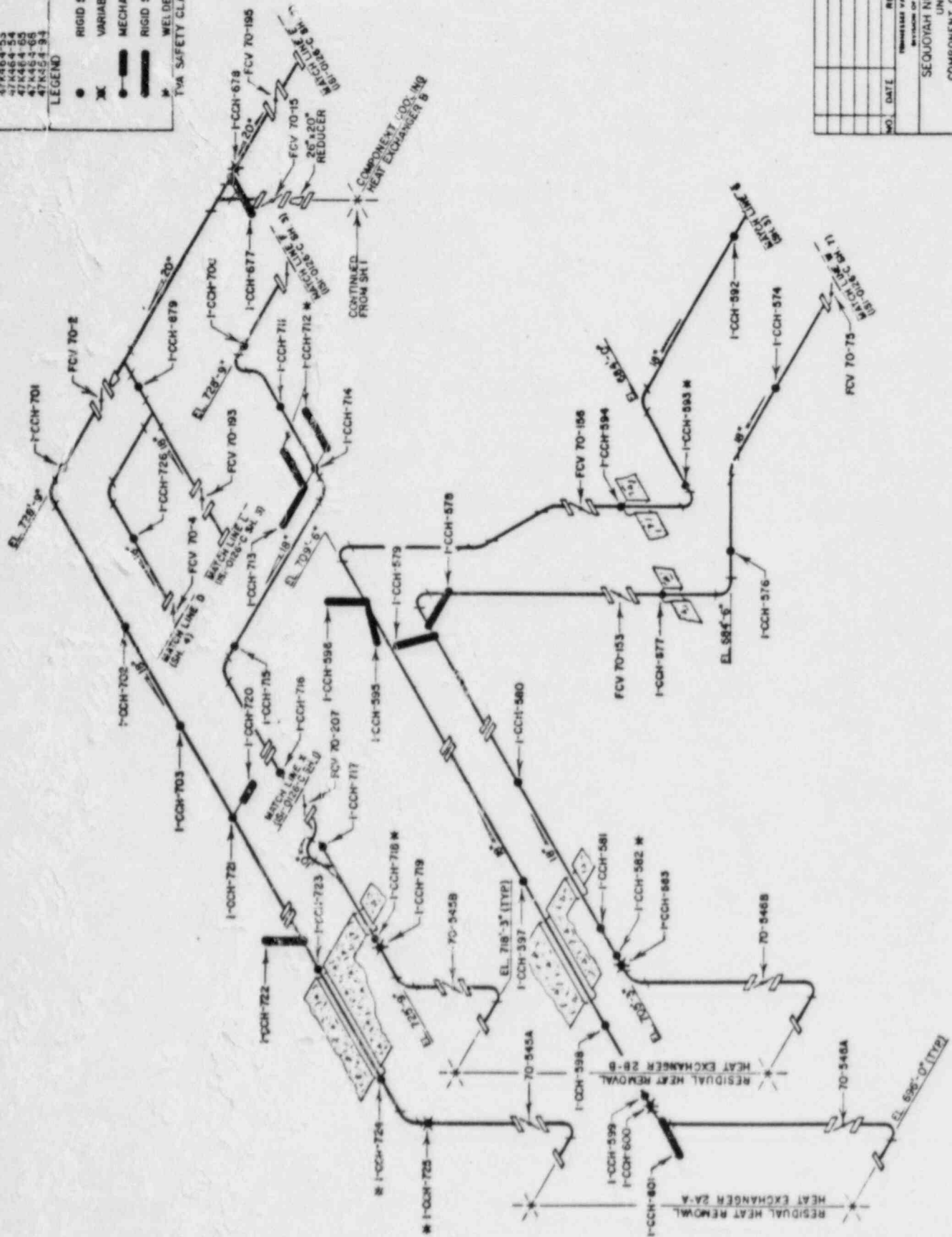
- RIGID SUPPORT
- ⊗ VARIABLE MECHANICAL SUPPORT
- MECHANICAL SHUBBER
- ▬ RIGID STRUT
- ⋈ WELDED ATTACHMENTS
- TVA SAFETY CLASS C

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NO.	DATE	REVISIONS	CR'D APP'D

UTAH STATE UNIVERSITY  
ENERGY RESEARCH CENTER  
SEQUOIA NUCLEAR PLANT  
UNIT # 2  
COMPONENT COOLING SYSTEM  
SUPPORT LOCATIONS

SI-114.2  
REV. 4  
DATE 12/23/83  
BY J.C.  
CHECKED BY J.C.  
DESIGNED BY J.C.  
DRAWN BY J.C.  
SCALE 1/8" = 1'-0"



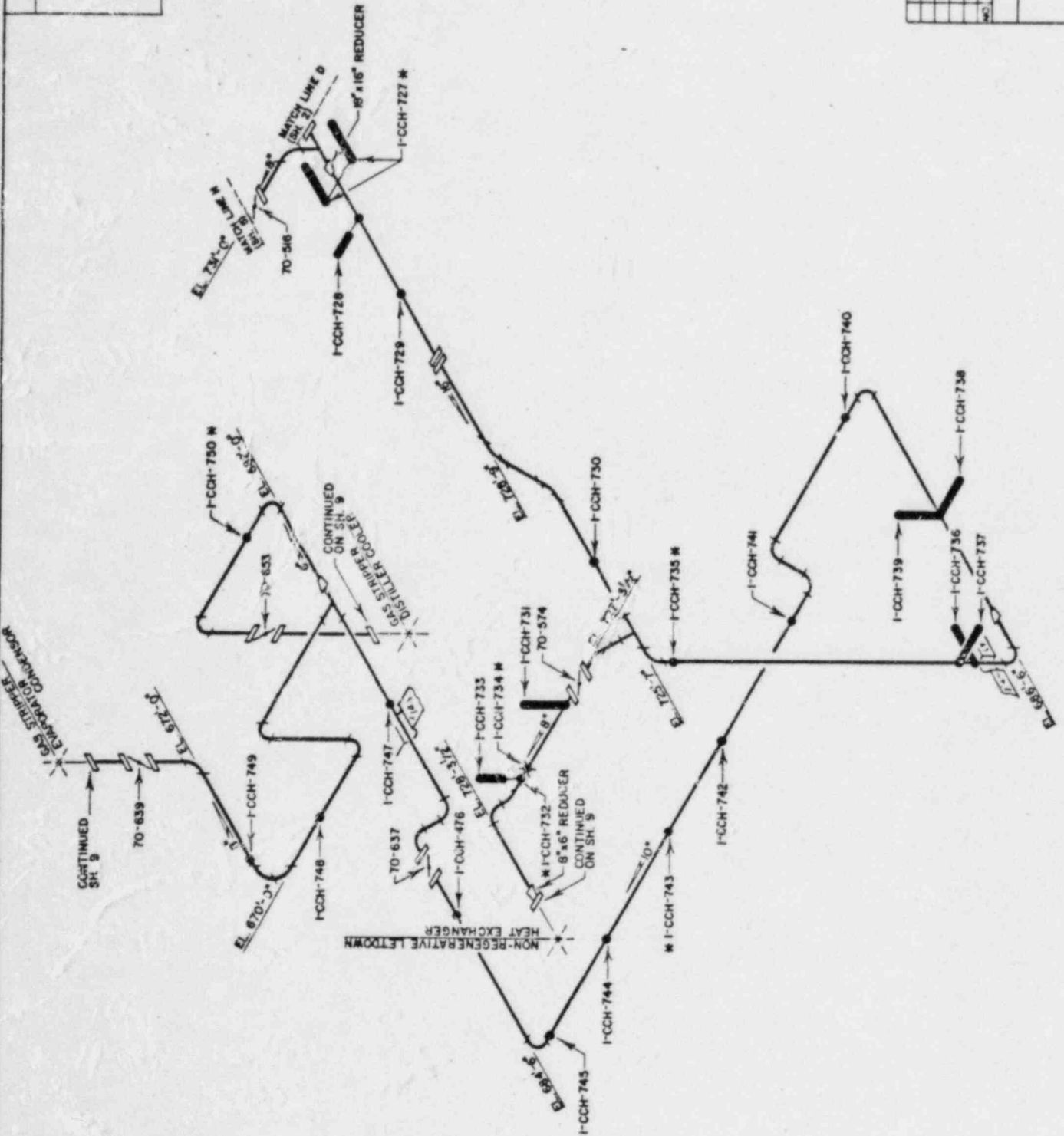


REFERENCE DRAWINGS  
47K464-5B

LEGEND

- RIGID SUPPORT
- ⊗ VARIABLE SUPPORT
- MECHANICAL SNUBBER
- RIGID STRUTT
- ⊛ WELD ATTACHMENTS
- TVA SAFETY CLASS C

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NO.	DATE	REVISIONS	BY	APP.

EMERALD VALLEY AUTHORITY  
DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
UNIT #2

SUPPORT COOLING SYSTEM  
SUPPORT LOCATION

DATE: 11/11/88  
BY: G.C.B.  
APP: [Signature]

ISF-0154-C

































REFERENCE DRAWINGS

- 47K427-63
- 47K440-58
- 47K450-33

LEGEND

- RIGID SUPPORT
- RIGID STRUTT
- WELDED ATTACHMENTS
- \* TVA SAFETY CLASS C

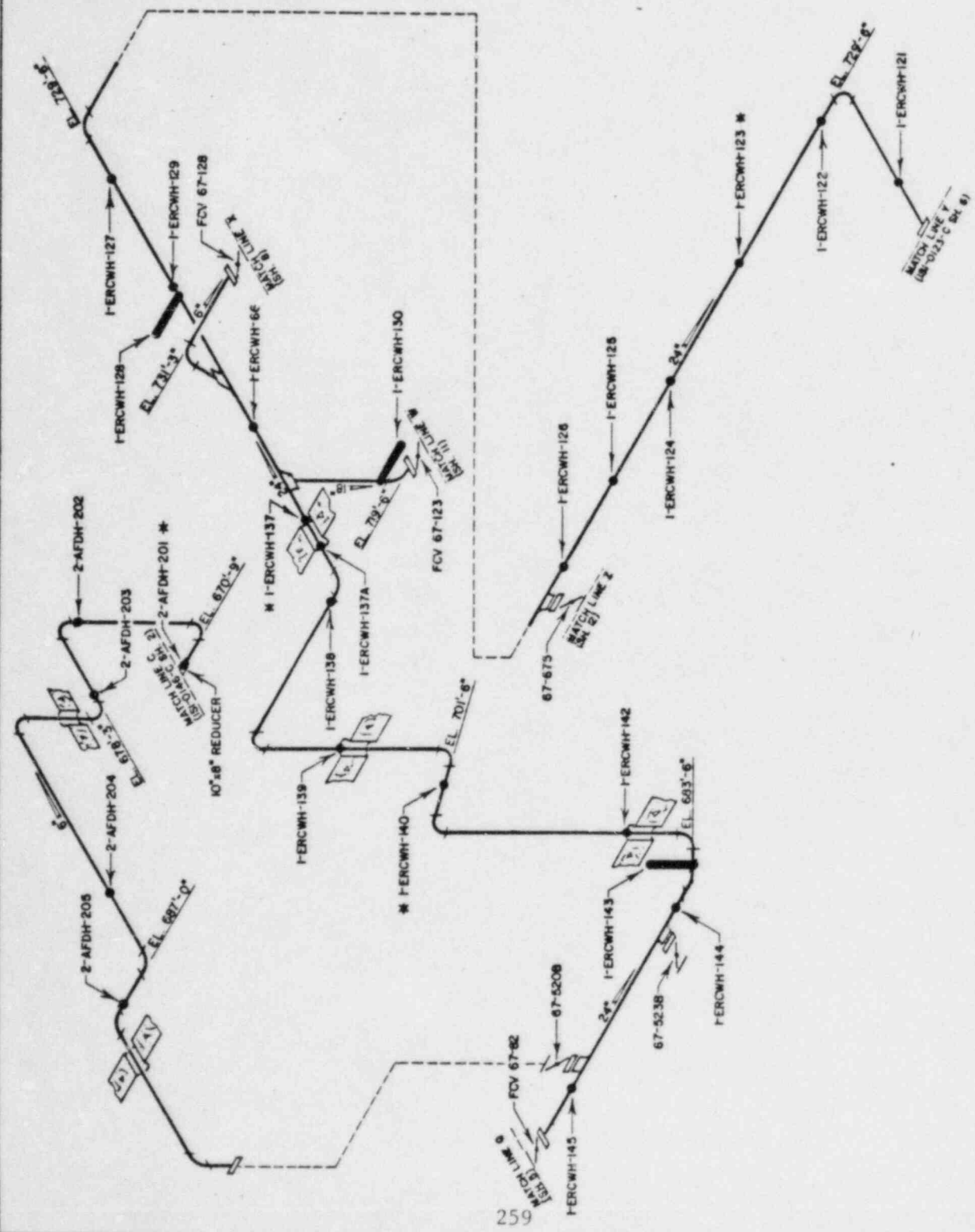
SQNP  
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NO.	DATE	REVISIONS	BY	CHK'D

SEQUOIA VALLEY AUTHORITY  
 DIVISION OF NUCLEAR POWER

SEQUOIA NUCLEAR PLANT  
 UNIT #2  
 ESSENTIAL RAW COOLING WATER SYSTEM  
 SUPPORT LOCATIONS

APPROVED: [Signature]  
 DATE: 11/10/83  
 SI-0156-C









APPENDIX A

TABLE D

Valves Subject to the Requirements of Examination  
 Category B-M-2 of Table IWB-2500-1 of Section XI

Valve No.	Code Class	Valve Cat.	Piping System	Valve Size	Valve Type	Valve Act	Valve Group No.	TVA Dwg No. (Weld Map)	Vendor Dwg No.	Vendor	Material Spec.	Valve Function	Forging/ Casting
FCV-63-67	A	B-PAS	SIS	10"	Gate	MO	1	CHM-2333-C	88926	Velan	ASTM A182		Forging
FCV-63-80	A	B-PAS	SIS	10"	Gate	MO	1	CHM-2333-C	88926	Velan	ASTM A182		Forging
FCV-63-98	A	B-PAS	SIS	10"	Gate	MO	1	CHM-2333-C	88926	Velan	ASTM A182		Forging
FCV-63-118	A	B-PAS	SIS	10"	Gate	MO	1	CHM-2333-C	88926	Velan	ASTM A182		Forging
63-560	A	AC-Act	SIS	10"	Ck	SA	2	CHM-2333-C	94-12892	Darling	ASTM A516 <sup>1</sup>	PSIV	
63-561	A	AC-Act	SIS	10"	Ck	SA	2	CHM-2333-C	94-12892	Darling	ASTM A516 <sup>1</sup>	PSIV	
63-562	A	AC-Act	SIS	10"	Ck	SA	2	CHM-2333-C	94-12892	Darling	ASTM A516 <sup>1</sup>	PSIV	
63-563	A	AC-Act	SIS	10"	Ck	SA	2	CHM-2333-C	94-12892	Darling	ASTM A516 <sup>1</sup>	PSIV	
63-622	A	AC-Act	SIS	10"	Ck	SA	2	CHM-2333-C	94-12892	Darling	ASTM A516 <sup>1</sup>	PSIV	
63-623	A	AC-Act	SIS	10"	Ck	SA	2	CHM-2333-C	94-1289	Darling	ASTM A516 <sup>1</sup>	PSIV	
63-624	A	AC-Act	SIS	10"	Ck	SA	2	CHM-2333-C	94-12892	Darling	ASTM A516 <sup>1</sup>	PSIV	
63-625	A	AC-Act	SIS	10"	Ck	SA	2	CHM-2333-C	94-12892	Darling	ASTM A516 <sup>1</sup>	PSIV	
63-640	A	AC-Act	SIS/RHR	8"	Ck	SA	2	CHM-2336-C	94-12892	Darling	ASTM A516 <sup>1</sup>	PSIV	
63-643	A	AC-Act	SIS/RHR	8"	Ck	SA	2	CHM-2336-C	94-12892	Darling	ASTM A516 <sup>1</sup>	PSIV	
63-558	A	AC-Act	SIS	6"	Ck	SA	3	CHM-2333-C	78704	Velan	ASTM A182	PSIV	Forging
63-559	A	AC-Act	SIS	6"	Ck	SA	3	CHM-2333-C	78704	Velan	ASTM A182	PSIV	Forging
63-632	A	AC-Act	SIS	6"	Ck	SA	3	CHM-2333-C	78704	Velan	ASTM A182	PSIV	Forging
63-633	A	AC-Act	SIS	6"	Ck	SA	3	CHM-2333-C	78704	Velan	ASTM A182	PSIV	Forging
63-634	A	AC-Act	SIS	6"	Ck	SA	3	CHM-2333-C	78704	Velan	ASTM A182	PSIV	Forging
63-635	A	AC-Act	SIS	6"	Ck	SA	3	CHM-2333-C	78704	Velan	ASTM A182	PSIV	Forging
63-641	A	AC-Act	SIS/RHR	6"	Ck	SA	3	CHM-2336-C	78704	Velan	ASTM A182	PSIV	Forging
63-644	A	AC-Act	SIS/RHR	6"	Ck	SA	3	CHM-2336-C	78704	Velan	ASTM A182	PSIV	Forging
68-563	A	C	RCS	6"	Re	SA	4	CHM-2334-C	H51688	Crosby	ASTM A182/ A351 <sup>2</sup>		See Note <sup>2</sup>

APPENDIX A

TABLE D

Valves Subject to the Requirements of Examination  
 Category B-M-2 of Table IWB-2500-1 of Section XI

Valve No.	Code Class	Valve Cat.	Piping System	Valve Size	Valve Type	Valve Act	Valve Group No.	TVA Dwg No. (Weld Map)	Vendor Dwg No.	Vendor	Material Spec.	Valve Function	Forging/ Casting
68-564	A	C	RCS	6"	Rel	SA	4	CHM-2334-C	H51688	Crosby	ASTM A182/ A351 <sup>2</sup>		See Note <sup>2</sup>
68-565	A	C	RCS	6"	Rel	SA	4	CHM-2334-C	H51688	Crosby		ASTM A182/ A351 <sup>2</sup>	
FCV 74-1	A	A-Act	RHR	14"	Gate	MO	5	CHM-2336-C	E-1-144831	Copes-Vulcan	ASTM A182	PSIV	Forging
FCV-74-2	A	A-Act	RHR	14"	Gate	MO	5	CHM-2336-C	E-1-144831	Copes-Vulcan	ASTM A182	PSIV	Forging
87-558	A	AC-Act	UHI	8"	Ck	SA	6	CHM-2337-C	B-148885	Copes-Vulcan	ASTM A182	PSIV	Forging
87-559	A	AC-Act	UHI	8"	Ck	SA	6	CHM-2337-C	B-148885	Copes-Vulcan	ASTM A182	PSIV	Forging
87-560	A	AC-Act	UHI	8"	Ck	SA	6	CHM-2337-C	B-148885	Copes-Vulcan	ASTM A182	PSIV	Forging
87-561	A	AC-Act	UHI	8"	Ck	SA	6	CHM-2337-C	B-148885	Copes-Vulcan	ASTM A182	PSIV	Forging
87-562	A	AC-Act	UHI	12"	Ck	SA	6	CHM-2337-C	B-148885	Copes-Vulcan	ASTM A182	PSIV	Forging
87-563	A	AC-Act	UHI	12"	Ck	SA	6	CHM-2337-C	B-148284	Copes-Vulcan	ASTM A182	PSIV	Forging

NOTES:

<sup>1</sup>Seal plate manufactured to ASTM A240 F304.

<sup>2</sup>Nozzle manufactured to ASTM A182 F316 - Forging Body manufactured to ASTM A351 CP8M - Casting

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APPENDIX C DATA  
SHEET 1

SEQUOYAH NUCLEAR PLANT  
UNIT 2, CYCLE \_\_\_\_\_  
INSERVICE INSPECTION FINAL REPORT

Reviewed by:

Quality Engineering Branch, Chief \_\_\_\_\_

Plant Superintendent \_\_\_\_\_

APPENDIX D  
NOTIFICATION OF INDICATION

PART I - FINDINGS

NOI No. \_\_\_\_\_ Plant/Unit \_\_\_\_\_  
Examination Report No. \_\_\_\_\_ Component ID \_\_\_\_\_  
Drawing No. \_\_\_\_\_

Description of Indication: (Attach Sketch/Photograph if needed)

\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

Signature of Examiner/Certification Level \_\_\_\_\_ Date \_\_\_\_\_

Signature of Field Supervisor (Contractor) \_\_\_\_\_ Date \_\_\_\_\_

NDE Section Representative \_\_\_\_\_ Date \_\_\_\_\_

PART II - DISPOSITION

\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

Disposition Prepared By \_\_\_\_\_ Date \_\_\_\_\_

Disposition Approved By \_\_\_\_\_ Date \_\_\_\_\_

PART III - VERIFICATION

Verification of Completed Corrective Action and/or Examination by NDE  
Section Representative.

\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_

Signature \_\_\_\_\_ Date \_\_\_\_\_

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SI-114.2  
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APPENDIX E  
REQUEST FOR RELIEF

REQUEST FOR RELIEF ISI-1

Components: Reactor coolant pumps (four per unit)

Class: TVA Safety Class A

Function: Circulates reactor coolant

Inspection Requirement: category B-L-2, item No. B12.20, visual examination of pump internal pressure boundary surface.

Basis for Relief:

In absence of required maintenance, disassembly of a reactor coolant pump solely to perform a visual examination of internal surfaces is impractical. This would represent an unnecessary employee exposure to high radiation and contamination areas and an excessive expense to TVA.

Time required for this major task of disassembly, examination, and reassembly would consume at least three weeks of 24-hours-per-day work. Radiation dose rates of the pump exterior will average 100 to 300 mrem/hour, and pump internal dose rates will average 10 to 20 rem/hour.

The benefit received from this major effort is minimal considering employee exposure, potential damage to safety-related equipment, and cost in dollars.

In addition, the two units at Sequoyah Nuclear Plant will operate under similar conditions. Therefore, we feel that if a pump from one of the units is disassembled for maintenance during a ten-year interval, the visual examination performed will be representative of the pump condition for each unit. This would avoid unnecessary employee exposure to the high radiation dose rates noted above. We conclude that if one pump is disassembled for maintenance during the ten-year interval, the visual examination performed satisfies examination category B-L-2 requirements for both units. Disassembly of the pump solely for visual examination is impractical.

REQUEST FOR RELIEF ISI-1

(Continued)

Alternate Inspection:

The internal surface of the reactor coolant pump casing will be visually examined whenever the surfaces are made accessible when a pump is disassembled for maintenance purposes. If during the ten-year interval a pump from either unit is not disassembled for maintenance, a pump from one unit shall be examined from the exterior. This shall be accomplished by ultrasonic thickness measurements of the pump casing.

REQUEST FOR RELIEF ISI-2

Components: Valves exceeding four-inch nominal pipe size

Class: TVA Safety Class A

Function: Various functions

Inspection Requirement: ASME Section XI, Table IWB-2500-1, examination category B-M-2, item No. B12.40, visual examination of valve internal pressure boundary surface.

Basis for Relief: During routine maintenance, visual examinations of valve body internal pressure boundary surfaces are performed and documented under existing plant administrative procedures. Most Class A valves, particularly containment isolation valves, are disassembled frequently for maintenance. In addition, the two units at Sequoyah Nuclear Plant will operate under similar conditions. If a valve from one of the units is disassembled for maintenance within a ten-year interval, we feel that the visual examination performed would be representative of both units and would be sufficient to satisfy the examination requirements for both units for that particular valve classification as defined in examination category B-M-2. We conclude that if one valve in each group of valves of the same constructional design and manufacturer that perform similar functions is disassembled from either unit during the ten-year interval, the visual examination performed satisfies examination category B-M-2 requirements of both units.

Alternate Inspection: If a valve from a particular classification has not been disassembled as the end of the inspection interval approaches, a case-by-case study will be made to determine the practicality of disassembling a valve from one of the units solely for visual examination (determine if draining the vessel would be required, etc.). If necessary, a request for relief will be issued at that time.



REQUEST FOR RELIEF ISI-3

Components: Pressure-retaining welds in piping

Class: TVA Safety Classes A and B

Function: Pressure-retaining component

Inspection Requirement: ASME Section XI, Table IWB-2500-1, examination categories B-F (item No. B5.50), B-J (item Nos. B9.10, B9.20, and B9.30), and C-F (item Nos. C5.10, C5.20, and C5.30), volumetric examination of longitudinal, circumferential, and pipe branch connection welds.

Basis for Relief: In some cases it will be impractical to inspect all welds from both sides, i.e., nonremovable hanger interference or valve and pump casings adjoining the welds. These welds will be noted on the ultrasonic examination data sheets.

Alternate Inspection: In addition to the visual examination performed during system leakage and hydrostatic pressure tests, a "best effort" ultrasonic examination will be performed. Where there is interference or problems from one direction only, consideration will be given to performing an angle beam examination for  $2T +$  (greater than two wall thicknesses from one edge of weld) from the accessible side of the weld in order to examine the entire weld cross-section.

REQUEST FOR RELIEF ISI-4

Component: Steam generator (four per unit)

Class: TVA Safety Class B

Inspection Requirement: ASME Section XI, Table IWB-2500-1, examination category C-A, item No. C1.10, volumetric examination of circumferential shell welds.

Basis for Relief: One circumferential shell weld on each generator is inaccessible due to the upper steam generator support brackets (weld Nos. SGW-D1, -D2, -D3 and -D4). See attached drawing CH-M-2345-B for weld location. Also attached drawing CH-M-2345-B for weld location. Also attached are drawings showing arrangements of the support brackets. One weld on one generator will be examined on a "best effort" basis for the baseline inspection intervals in accordance with IWC-2411 and Table IWC-2500-1.

Alternate Inspection: None

REQUEST FOR RELIEF ISI-5

Component: Reactor Pressure Vessel

Class: TVA Safety Class A

Inspection Requirement: ASME Section XI, Table IWB-2500-1, examination category B-A, item No. B1.20, 100-percent preservice baseline volumetric examination of lower head dollar weld, under conditions and with equipment and techniques equivalent to those expected to be employed during inservice inspection.

Basis for Relief: TVA will employ automated remote inspection devices to examine most of the reactor vessel welds. These examinations will be conducted from the vessel inside diameter. However, the lower head weld on each reactor pressure vessel is partially inaccessible for examination from the vessel inside diameter due to instrumentation tubes which penetrate the lower head (weld No. W01-02 - see attached drawings). Portions of the weld can be examined from one side (as permitted by T-441.4, Article 4 of Section XI) and will include 100 percent of the examination volume in accordance with IWB-3511.1 of Section V. These portions of the weld will be reexamined during the inservice intervals in accordance with the examination category B-A of Table IWB-2500-1.

Alternate Inspection: A 100-percent baseline examination of the weld will be conducted from the vessel outside diameter. This will be accomplished by performance of a manual ultrasonic examination. A remote ultrasonic examination will be conducted from the vessel inside diameter on all accessible areas of the weld.

REQUEST FOR RELIEF ISI-6

Component: Steam Generator (four per unit)

Class: TVA Safety Class A

Inspection Requirement: ASME Section XI, Table IWB-2500-1, examination category B-D, item No. B3.60, volumetric examination of nozzle inside radius section on the primary side.

Basis for Relief: Each steam generator consists of two integrally cast nozzles and two integrally cast manways. The present capability of ultrasonic testing is not sufficient to examine cast material of this thickness and achieve meaningful results. (See attached drawing).

Alternate Inspection: None

REQUEST FOR RELIEF ISI-7

Component: Reactor Coolant Loop Piping Welds (2)

Class: TVA Safety Class A

Inspection Requirement: ASME Section XI, 1WB-2500-1, examination category B-J, item No. B9.10, volumetric examination of circumferential welds.

Basis for Relief: Two circumferential shell welds in the reactor coolant loop piping (RC-23S1 and -31S1, loops 3 and 4) are located inside the reactor vessel shield wall and are inaccessible for baseline and inservice examination (see attached drawings). Both welds have undergone shop radiographic examinations. Since the baseline inspection serves as a reference to future inservice inspections and both welds will be inaccessible for inservice inspections, the shop radiographic examinations coupled with the Section III hydrostatic test will provide adequate proof of integrity of the system welds. Inservice system leakage and hydrostatic testing will prove weld integrity during the life of the plant.

Alternate Inspection: None

REQUEST FOR RELIEF ISI-8

Component: Reactor coolant pumps (four per unit)

Class: TVA Safety Class A

Inspection Requirement: ASME Section XI, IWB-2500-1, examination category B-L-1, item No. B12.10, volumetric examination of pressure-retaining welds in pump casing.

Basis for Relief: Each reactor coolant pump casing consists of a two-piece welded type 304 SST casting. The present capability of ultrasonic testing is not sufficient to examine cast material of this thickness and achieve meaningful results.

Alternate Inspection: All four welds will be surface examined for the preservice baseline, and one weld will be surface examined during each inspection interval.

SEQUOYAH NUCLEAR PLANT

REQUEST FOR RELIEF ISI-9

Component: Uncladded vessel welds in ferritic material less than two inches in thickness.

Class: TVA Safety Classes A and B

Inspection Requirement: Ultrasonic examination of welds, paragraph T-530 of ASME Section V, Article 5, 1977 Edition, Summer 1978 Addenda as referenced in paragraph IWA-2232 (c) of ASME Section XI 1977 Edition Summer 1978 Addenda.

Basis for Relief: Paragraph T-533.2(a) of Article 5 of ASME Section V requires that the basic calibration block include a basic calibration hole drilled parallel to the contact surface. However, paragraph T-533.2(b) permits the use of other calibration reflectors provided equivalent responses to that from the basic calibration hole are demonstrated.

TVA currently uses five percent notches in lieu of side drilled holes. Although the use of the five percent notch cannot be shown to be equivalent in all cases to the applicable side-drilled holes, TVA considers that examinations are technically acceptable based on the calibration requirements of paragraph III-3430 of Appendix III to the 1977 Edition, Summer 1978 Addenda of ASME Section XI. The calibration notches for ferritic material are 10%t when t is less than .312" and .104t-.009t<sup>2</sup> for material .312"-6" thick. TVA's use of five percent notches is considered equivalent to the latest approved code examination techniques.

Alternate Inspection: TVA proposes to continue the use of notches located on the I.D. and O.D. surfaces at a nominal depth of 5%t as reference reflectors.

REQUEST FOR RELIEF ISI-10

Components: Reactor vessel flange to upper shell weld

Class: TVA Safety Class A

Inspection Requirement: ASME Section XI, Table IWB-2500-1, examination category B-A, item No. B1.30, volumetric from flange face

Basis for Relief: The reactor vessel flange-to-upper shell weld is located behind the core barrel and is therefore inaccessible until the core barrel is removed. The vessel flange-to-upper shell weld is 41.9 inches below the flange face. Due to the location of the vessel flange-to-upper shell weld, TVA intends to address the weld as a reactor vessel shell weld.

We have reviewed the Sequoyah Reactor Vessel Stress Report entitled Analysis of the Main Closure Including Core Support Ledge (Document No. 30616-1105) purposely to determine a fatigue usage factor for the vessel flange to shell weld. This analysis does not provide a usage factor specifically for the weld because the analysis considers weld and base material to be homogeneous and equal in elasticity, strength, and fatigue properties. Instead, the analysis provides usage factors at critical locations.

The maximum fatigue usage factor in the vessel in the vicinity of the flange to shell weld as found in the above analysis is 0.00662 and this value can be conservatively used for the weld. We consider the value of 0.00662 to be extremely low compared to the code allowed fatigue usage factor of 1.0.

We conclude that the distance (") from the flange face to the flange-to-upper shell weld coupled with present ultrasonic techniques and the very low fatigue usage factor that the flange-to-upper shell weld should be treated as a reactor vessel shell weld.

Alternate Inspection: A remote ultrasonic examination of the weld will be conducted from the vessel inside diameter near the end of the inspection interval.



REQUEST FOR RELIEF ISI-11

Components: Feedwater Piping System and Associated Supports

Class: TVA Safety Class B

Inspection Requirement: IE Bulletin 79-13, Revision 2, dated October 16, 1979, was issued to provide inspection requirements for part of the feedwater piping system and associated supports to monitor the feedwater pipe cracking problem.

Item 1.a. requires the TVA perform radiographic (RT) examination, supplemented by ultrasonic examination as necessary to evaluate indications, of all feedwater nozzle-to-pipe welds of adjacent pipe and nozzle areas (a distance equal to at least two wall thicknesses). Evaluation shall be in accordance with ASME Section III, Subsection NC, Article NC-5000. Radiography shall be performed to the 2T penetrameter sensitivity level in lieu of Table NC-5111-1 with systems void of water. TVA met this requirement after hot functional testing for Sequoyah units 1 and 2.

Item 1.c. requires that TVA perform visual inspection of feedwater system piping supports and snubbers in containment to verify operability and conformance to design. TVA also met this requirement after hot functional testing.

Item 2.a of actions to be taken by licenses in IE Bulletin 79-13, Revision 2, dated October 16, 1979, requires during the first refueling outage that TVA perform volumetric examination of the feedwater nozzle-to-pipe welds, the feedwater piping welds to the first support, the feedwater line-to-containment penetration welds, the line one pipe in diameter downstream of auxiliary feedwater to main feedwater connection. Also, item 2.c. requires that TVA perform a visual inspection of all feedwater system piping supports and snubbers in containment to verify operability and conformance to design. These requirements were imposed before the cause of the feedwater line cracking problem was known.

REQUEST FOR RELIEF ISI-11

Basis for Relief:

TVA requests relief from examining the feedwater piping welds to the first support, the feedwater line-to-containment penetration welds, and the main feedwater line one pipe diameter downstream of the auxiliary feedwater to main feedwater connection. As required by item 2.a. TVA proposes to perform a volumetric examination on the feedwater nozzle-to-pipe welds in performing visual inspections of all feedwater system piping supports and snubbers in containment other than required by ASME Section XI.

At the time IE Bulletin 79-13 was issued, the real cause of the feedwater pipe cracking problem was not fully understood. As a result, TVA believes that a request for relief from these requirements is warranted. Our justification is as follows:

1. The Westinghouse Owners Group's efforts which are documented in WCAP-9693, Investigation of Feedwater Line Cracking in Pressurized Water Reactor Plants, dated June 1980 provided information to substantiate that the feedwater pipe cracks are fatigue failures which are caused by thermal stratification and thermal striping during low-flow rate feedwater injection. The effects of thermal stratification and striping are enhanced by temperature difference between cold feedwater and hot steam generators.
2. TVA believes there is a basis for the item 2.a. volumetric examinations of the feedwater nozzle-to-pipe welds because thermal stratification and striping are known to occur at this location; however, now that the cause of feedwater pipe cracking is known, TVA does not believe there is a basis for requiring volumetric examinations of the remaining feedwater piping in item 2.a. because the thermal and geometric conditions do not exist at these locations to support thermal stratification and striping. In addition, during a telephone conversation with NRC on September 2, 1982, TVA questioned why the IE Bulletin required RT examinations of the remaining feedwater piping.

REQUEST FOR RELIEF ISI-11

Bill Crowley, of NRC, stated that this requirement was for purposes of "upgrading" the construction radiographs of this requirement piping from 4T sensitivity to 2T sensitivity. Mr. Crowley indicated that other plants which have performed this "upgraded" RT in compliance with the IE Bulletin had found instances of flaws that required repair; however, the feedwater piping in question for Sequoyah units 1 and 2 was in accordance with ANSI B31.7, 1969 Edition with 1970 Addenda, and was RT examined in accordance with TVA process specification 3.M.2.1(d) which requires 2T sensitivity and acceptance standards identical with NC-5000 of ASME Section III. This was verified through discussions with personnel who performed the RT during construction and an independent review of the site radiographs. It is therefore TVA's position that no "upgrading" of the feedwater piping welds is required for Sequoyah units 1 and 2.

Based on the above justification, TVA concludes that the subject request for relief does not affect safe operation of the unit.

Alternate Inspection:

Perform volumetric examinations of the feedwater nozzle-to-pipe welds in accordance with item 2.a. of IE Bulletin 79-13.

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APPENDIX F  
AUGMENTED INSPECTIONS

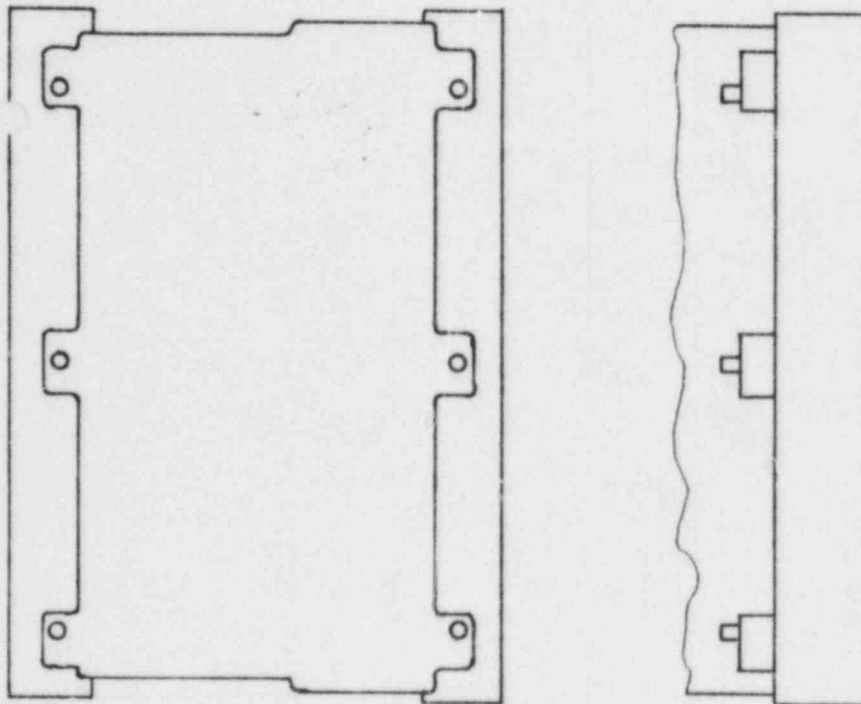
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REFERENCE DRAWINGS

P-E274165

NOTES

1. THE PUMP FEET ARE BOLTED TO A COMMON SUPPORT
2. PUMP SUPPORT NUMBERS  
1-PDPH-1C  
2-PDFH-1C



TENNESSEE VALLEY AUTHORITY  
DIVISION OF NUCLEAR POWER

SEQUOYAH NUCLEAR PLANT  
UNITS #1 & #2  
RECIPROCATING CHARGING PUMP SUPPORTS  
(POSITIVE DISPLACEMENT PUMP)

SCALE: <i>NTS</i>	SUBMITTED	APPROVED	DATE <i>8-4-84</i>
DRAWN <i>KEV</i>	<i>edc</i>	<i>[Signature]</i>	SHEET OF SHEETS
TRACED			ISI-0164-A
CHECKED <i>edc</i>			

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APPENDIX G  
SCAN PLAN DATA  
UNIT 2

12/15/83

TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2009 DIXIE, TENNESSEE  
 UNIT ? COMMERCIAL SERVICE DATE 06/01/82

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C Y :	SYSTM	EXAM AREA	R E :	T C :	S H :	M A :	NDE THICK	PROC METH	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S
01	AST	ASTH000003001	0	CE	LS	ISI0071	01 C 00.00 0.000	VT	NUT 01		RC530		LB4 03/20/83	L-AE9	A
01	AST	ASTH000003001A	0	CC	IA	ISI0071	01 S 00.00 0.000	PT	NPT 04		R0723		JEF 03/25/83	L-AE9	A
01	BIT	BITH000003001	0	CE	CS	ISI0074	01 00.00 0.000	VT	NUT 01		R0599		JEF 03/25/83	L-PC590	A
01	BIT	BIT0000003002	0	CB	NH	ISI0074	01 C 00.00 2.000	MT	NMT 02		R0529		JEF 03/25/83	L-PC590	A
01	BIT	BIT0000003003	0	CA	AW	ISI0074	01 C 00.00 2.000	UT 0	NUT 19	BNP79	RC731	CC042	LB4 03/25/83	L-PC590	A
								UT45	NUT 19	BNP79	R0732	CC043	JG3 03/25/83	L-PC590	A
								UT50	NUT 19	BNP79	PC733	CC044	JG3 03/25/83	L-PC590	A
01	BIT	MWY STUDS	0	CD	MB	ISI0074	01 C 12.50 2.500	UT 0	NUT 05	SQ 55	RC555	C0006	LB4 03/03/83	L-PC590	A
01	CS	CSF0000003001	0	CF	DW	ISI0007	01 S 12.00 0.406	PT	NPT 04		R0019		LEW 07/25/83	L-PC553	A
01	CS	CSH0000003440	0	CE	RS	MSG0011	01 S 12.00 0.406	VT	NVT 01		RC022		LEW 07/25/83	L-PC553	NOI#50002
01	CS	CSH0000003440	1	CE	RS	MSG0011	01 S 12.00 0.406	VT	MVT 01		R0597		MSE 03/25/83	L-PC553	A
01	CS	CSH0000003441	0	CE	RS	MSG0011	01 S 12.00 0.406	VT	NVT 01		RC014		LEW 07/25/83	L-PC553	A
01	CS	CSH0000003442	0	CE	RS	MSG0011	01 S 12.00 0.406	VT	NVT 01		RC013		LEW 07/25/83	L-PC553	A
01	CS	CSH0000003445	0	CE	RS	MSG0011	01 S 12.00 0.406	VT	NVT 01		R0012		LEW 07/25/83	L-PC553	A
01	CS	CSH0000003450	0	CE	RS	MSG0011	01 S 12.00 0.406	VT	NVT 01		R0015		LEW 07/25/83	L-PC553	A
01	CVC	CVCF0000032078	0	BJ	DW	ISI0009	02 S 03.00 0.439	PT	NPT 04		R0533		C6T 03/17/83	L-LC	A
01	CVC	CVCF000003210	0	BJ	DW	ISI0009	02 S 03.00 0.439	PT	NPT 04		R0532		C6T 03/17/83	L-LC	A
01	CVC	CVCF000003243	0	BJ	DW	ISI0009	02 S 03.00 0.439	PT	NPT 04		R0534		SDF 03/17/83	L-LC	A
01	CVC	CVCF000003243A	0	BJ	DW	ISI0009	02 S 03.00 0.439	PT	NPT 04		R0535		SDF 03/17/83	L-LC	A
01	CVC	CVCH000003001	0	BK2	RS	MSG0015	01 S 02.00 0.343	VT	NVT 01		R0013		JEF 07/31/83	L-4AR	A
01	CVC	CVCH000003002	0	BK2	RS	MSG0015	01 S 02.00 0.343	VT	NVT 01		R0014		JEF 07/31/83	L-4AR	A
01	CVC	CVCH000003003	0	BK2	RS	MSG0015	01 S 02.00 0.343	VT	NVT 01		R0039		JEF 07/27/83	L-RH	A
01	CVC	CVCH000003004	0	BK2	RS	MSG0015	01 S 02.00 0.343	VT	NVT 01		RC100		JEF 07/27/83	L-RH	A
01	CVC	CVCH000003005	0	BK2	RS	MSG0015	01 S 02.00 0.343	VT	NVT 01		RC101		JEF 07/27/83	L-RH	A
01	CVC	CVCH000003005	0	BK1	IA	MSG0015	01 S 02.00 0.343	PT	NPT 04		R0071		LE4 07/25/83	L-RH	A

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12/15/83

TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 CHATTANOOGA, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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SYSTEM	EXAM AREA	R E C O D E	T Y P E	S H E E T	M A T E R I A L	NDE METH	PPDC NO.	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S	
31 CVC	CVC4000000077	3	BK2	RS	MSG0015	03	S	02.00	0.343	VT	NVT	01	P0142	JEF 07/27/83 L-RW	A
31 CVC	CVC4000000078	3	BK2	RS	MSG0015	03	S	02.00	0.343	VT	NVT	01	R0143	JEF 07/27/83 L-RW	A
31 CVC	CVC4000000079	3	BK2	RS	MSG0015	03	S	02.00	0.343	VT	NVT	01	R0144	JEF 07/27/83 L-RW	A
31 CVC	CVC4000000080	3	BK2	RS	MSG0015	03	S	02.00	0.343	VT	NVT	01	R0145	JEF 07/27/83 L-RW	A
31 CVC	CVC4000000081	3	BK2	RS	MSG0015	03	S	02.00	0.343	VT	NVT	01	R0146	JEF 07/27/83 L-RW	A
01 CVC	CVC4000000082	3	BK2	RS	MSG0015	04	S	02.00	0.343	VT	NVT	01	R0147	JEF 07/27/83 L-RW	A
31 CVC	CVC4000000083	3	BK2	RS	MSG0015	04	S	02.00	0.343	VT	NVT	01	R0148	JEF 07/27/83 L-RW	A
31 CVC	CVC4000000084	3	BK2	RS	MSG0015	04	S	02.00	0.343	VT	NVT	01	R0149	JEF 07/27/83 L-RW	A
31 CVC	CVC4000000085	3	BK2	RS	MSG0015	04	S	02.00	0.343	VT	NVT	01	R0150	JEF 07/27/83 L-RW	A
31 CVC	CVC4000000086	3	BK2	RS	MSG0015	04	S	02.00	0.343	VT	NVT	01	R0151	JEF 07/27/83 L-RW	A
31 CVC	CVC4000000087	3	BK2	RS	MSG0015	04	S	02.00	0.343	VT	NVT	01	R0152	JEF 07/27/83 L-RW	A
01 CVC	CVC4000000088	3	BK2	RS	MSG0015	04	S	02.00	0.343	VT	NVT	01	R0255	MSE 08/05/83 L-3AR	A
01 CVC	CVC4000000089	3	BK2	RS	MSG0015	04	S	02.00	0.343	VT	NVT	01	R0535	MSE 03/21/83 L-LC	A
31 CVC	CVC4000000090	3	BK2	RS	MSG0015	04	S	02.00	0.343	VT	NVT	01	R0536	MSE 03/21/83 L-LC	A
31 CVC	CVC4000000101	3	BK2	RS	MSG0015	05	S	02.00	0.343	VT	NVT	01	R0217	JEF 07/31/83 L-4AR	A
31 CVC	CVC4000000102	3	BK2	RS	MSG0015	05	S	02.00	0.343	VT	NVT	01	R0153	JEF 07/27/83 L-4AR	A
31 CVC	CVC4000000103	3	BK2	RS	MSG0015	05	S	02.00	0.343	VT	NVT	01	R0154	JEF 07/27/83 L-RW	A
31 CVC	CVC4000000104	3	BK2	RS	MSG0015	05	S	02.00	0.343	VT	NVT	01	R0155	JEF 07/27/83 L-RW	A
01 CVC	CVC4000000105	3	BK2	RS	MSG0015	05	S	02.00	0.343	VT	NVT	01	R0156	JEF 07/27/83 L-RW	A
01 CVC	CVC4000000105	3	BK2	RS	MSG0015	05	S	02.00	0.343	VT	NVT	01	R0219	JEF 07/31/83 L-4AR	A
31 CVC	CVC4000000107	3	BK2	RS	MSG0015	05	S	02.00	0.343	VT	NVT	01	R0537	MSE 03/21/83 L-LC	A
31 CVC	CVC4000000108	3	BK2	RS	MSG0015	05	S	02.00	0.343	VT	NVT	01	R0538	MSE 03/21/83 L-LC	A
31 CVC	CVC4000000113	3	BK2	RS	MSG0015	05	S	02.00	0.343	VT	NVT	01	R0320	JEF 07/31/83 L-4AR	A
31 CVC	CVC4000000123	3	BK2	RS	MSG0015	03	S	02.00	0.343	VT	NVT	01	R0321	JEF 07/31/83 L-4AR	A

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12/15/83

TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER SQNP  
1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402 SI-114.2  
SECOYAH NUCLEAR PLANT Page 6 of 67  
P. O. BOX 2000 DAISSY, TENNESSEE Rev. 5  
UNIT ? COMMERCIAL SERVICE DATE 05/01/82

Y	SYSTM	EXM RNER	R	COD CAT	Y	T	S	H	A	Y	SIZE	THICK	METH	NCE MC.	PROC MC.	CAL NO.	BLCK NO.	REPT NO.	C-L NO.	I M	P	DATE	COMMENTS	R	E	S
01	CVC	CUC4000003293	3	BK2	RS	MS60012	01	S	03.00	0.438	VT	NVT	01				RO342		MSE	08/21/83	L-LC				A	
01	CVC	CUC4000003294	3	BK2	RS	MS60012	01	S	03.00	0.438	VT	NVT	01				RO343		MSE	03/21/83	L-LC				A	
01	CVC	CUC4000003300	3	BK2	RS	MS60012	01	S	03.00	0.438	VT	NVT	01				RO344		MSE	08/21/83	L-LC				A	
01	CVC	CUC4000003301	3	BK2	RS	MS60012	01	S	03.00	0.438	VT	NVT	01				RO345		MSE	03/21/83	L-LC				A	
01	CVC	CUC4000003302	3	BK2	RS	MS60012	01	S	03.00	0.438	VT	NVT	01				RO346		MSE	09/21/83	L-LC				A	
01	CVC	CUC4000003303	0	BK2	MS	MS60012	02	S	03.00	0.438	VT	NVT	01				RO347		MSE	09/21/83	L-LC				A	
01	CVC	CUC4000003307	0	BK2	RS	MS60012	02	S	02.00	0.343	VT	NVT	01				RO347		MSE	03/21/83	L-2AR				A	
01	CVC	CUC4000003342	3	BK2	RS	MS60012	02	S	03.00	0.438	VT	NVT	01				RO348		MSE	03/21/83	L-LC				A	
01	CVC	CUC4000003343	3	BK2	RS	MS60012	02	S	03.00	0.438	VT	NVT	01				PO349		MSE	08/21/83	L-LC				A	
01	CVC	CUC4000003344	3	BK2	RS	MS60012	02	S	03.00	0.438	VT	NVT	01				PO350		MSE	03/21/83	L-LC				A	
01	CVC	CUC4000003345	3	BK2	RS	MS60012	02	S	03.00	0.438	VT	NVT	01				RC351		MSE	09/21/83	L-LC				A	
01	CVC	CUC4000003346	3	BK2	RS	MS60012	02	S	03.00	0.438	VT	NVT	01				PC352		MSE	03/21/83	L-LC				A	
01	CVC	CUC4000003347	3	BK2	MS	MS60012	02	S	03.00	0.438	VT	NVT	01				RO353		MSE	09/21/83	L-LC				A	
01	CVC	CUC4000003343	3	BK2	RS	MS60012	02	S	03.00	0.438	VT	NVT	01				PO354		MSE	08/21/83	L-LC				A	
01	CVC	CUC4000003349	3	BK2	MS	MS60012	02	S	03.00	0.438	VT	NVT	01				PO355		MSE	09/21/83	L-LC				A	
01	CVC	CUC4000003350	3	BK2	RS	MS60012	02	S	03.00	0.438	VT	NVT	01				PC344		RM	03/23/83	L-LC				A	
01	CVC	CUC4000003361	3	BK2	RS	MS60012	02	S	03.00	0.438	VT	NVT	01				PO356		MSE	03/21/83	L-LC				A	
01	CVC	CUC4000003362	3	BK2	RS	MS60012	02	S	03.00	0.438	VT	NVT	01				PO357		MSE	09/21/83	L-LC				A	
01	CVC	CUC4000003363	3	BK2	MS	MS60012	02	S	03.00	0.438	VT	NVT	01				PO358		MSE	03/21/83	L-LC				A	
01	CVC	CUC4000003354	3	BK2	MS	MS60012	02	S	03.00	0.438	VT	NVT	01				RO359		MSE	03/21/83	L-LC				A	
01	CVC	CUC4000003301	3	BJ	CH	ISI0003	01	S	03.00	0.438	PT	NPT	04				PO347		RM	03/03/83	L-LC				A	
01	CVC	CUC3000001041	3	BJ	SH	MS60003	01	S	02.00	0.343	PT	NPT	04				PO379		MSE	03/15/83	L-RW				A	
01	CVC	CUC2000001045	0	EJ	SH	MS60003	01	S	02.00	0.343	PT	NPT	04				PO077		LEM	07/25/83	L-RN	NOIMS00007				R
01	CVC	CUC0000001045	1	BJ	SH	MS60003	01	S	02.00	0.343	PT	NPT	04				PO527		GKL	08/15/83	L-RW					A

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P. O. BOX 2003 BIVIS, TENNESSEE  
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Y	SYSTEM	EXAM PREP	2 E C O D E	3 C A T E	4 Y	5 M	6 A	7 S I Z E	8 T H I C K N E S S	9 M E T H O D	10 P R O C E D U R E	11 C O L O R	12 R E P T	13 N O.	14 P	15 D A T E	16 C O M M E N T S	17 R E S		
01	CVC	CVC000001045	0	BJ	SW	MSG000	C1	5	02.00	0.343	PT	NPT	04	RC579		MSE	03/13/83	L-1AR	#1A5	A
01	CVC	CVC000001047A	0	BJ	SW	MSG000	C1	5	02.00	0.343	PT	NPT	04	RC512		CGT	08/15/83	L-LC		A
01	CVC	CVC000001047B	0	BJ	SW	MSG000	C1	5	02.00	0.343	PT	NPT	04	RC511		CGT	03/15/83	L-LC		A
01	CVC	CVC000001043	0	BJ	SW	MSG000	C1	5	02.00	0.343	PT	NPT	04	RC526		CGT	08/17/83	L-LC	#1A5	A
01	CVC	CVC000001051	0	BJ	SW	MSG000	C1	5	02.00	0.343	PT	NPT	04	RC515		GDF	03/15/83	L-LC		A
01	CVC	CVC000001052	0	BJ	SW	MSG000	C1	5	02.00	0.343	PT	NPT	04	RC537		CGT	09/17/83	L-LC	#1A5	A
01	CVC	CVC000001054	0	BJ	SW	MSG000	C1	5	02.00	0.343	PT	NPT	04	RC51X		GDF	03/15/83	L-LC		A
01	CVC	CVC000001055	0	BJ	SW	MSG000	C1	5	02.00	0.343	PT	NPT	04	RC530		CGT	03/17/83	L-LC	#1A5	A
01	CVC	CVC000001057	0	BJ	SW	MSG000	C1	5	01.50	0.331	PT	NPT	04	RC514		GDF	03/15/83	L-LC		A
01	CVC	CVC000001057	0	BJ	SW	MSG000	C2	5	02.00	0.343	PT	NPT	04	RC519		MSE	08/17/83	L-PH	#1A5	A
01	CVC	CVC000001067	0	BJ	SW	MSG000	C2	5	02.00	0.343	PT	NPT	04	RC576		MSE	08/22/83	L-RH	#1A5	A
01	CVC	CVC000001070	0	BJ	SW	MSG000	C2	5	02.00	0.343	PT	NPT	04	RC540		MSE	08/17/83	L-RH	#1A5	A
01	CVC	CVC000001077	0	BJ	SW	MSG000	C2	5	02.00	0.343	PT	NPT	04	RC592		GDF	02/17/83	L-LC		A
01	CVC	CVC000001073	0	BJ	SW	MSG000	C2	5	02.00	0.343	PT	NPT	04	RC593		HJD	03/17/83	L-LC	#1A5	A
01	CVC	CVC000001095	0	BJ	SW	MSG000	C2	5	02.00	0.343	PT	NPT	04	RC594		GDF	08/17/83	L-LC		A
01	CVC	CVC000001097	0	BJ	SW	MSG000	C2	5	02.00	0.343	PT	NPT	04	RC595		GDF	08/17/83	L-LC	#1A5	A
01	CVC	CVC000001099	0	BJ	SW	MSG000	C2	5	01.50	0.331	PT	NPT	04	RC596		GDF	03/17/83	L-LC		A
01	CVC	CVC000001091	0	BJ	SW	MSG000	C2	5	01.50	0.331	PT	NPT	04	RC597		GDF	03/17/83	L-LC	#1A5	A
01	CVC	CVC000001145	0	BJ	SW	MSG000	C4	5	02.00	0.343	PT	NPT	04	RC541		MSE	03/17/83	L-RH	#1A5	A
01	CVC	CVC000001147	0	BJ	SW	MSG000	C4	5	02.00	0.343	PT	NPT	04	RC542		MSE	03/17/83	L-RH	#1A5	A
01	CVC	CVC000001143	0	BJ	SW	MSG000	C4	5	02.00	0.343	PT	NPT	04	RC070		LSH	07/26/83	L-PH		A
01	CVC	CVC000001153	0	BJ	SW	MSG000	C4	5	02.00	0.343	PT	NPT	04	RC593		CGT	09/17/83	L-LC	#1A5	A
01	CVC	CVC000001156	0	BJ	SW	MSG000	C4	5	02.00	0.343	PT	NPT	04	RC575		CGT	03/17/83	L-LC		A
01	CVC	CVC000001153	0	BJ	SW	MSG000	C4	5	02.00	0.343	PT	NPT	04	RC550		CGT	08/17/83	L-LC		A

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 P. O. BOX 2000 DAVIS, TENNESSEE  
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LINE	SYSTEM	EXAM AREA	R E C D #	T C A T	DRAWING	S H #	M A T	SIZE	THICK	METH	NDE	PRJC NO.	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I M P	DATE	COMMENTS	R E S
31	CVC	CVC0000001164	3	BJ	SW	MSG0008	04	S	02.00	0.343	PT	NPT 04		RCF01		CGT	08/17/83	L-LC #145	A
31	CVC	CVC0000001167	3	BJ	SW	MSG0008	04	S	02.00	0.343	PT	NPT 04		RCF02		CGT	08/17/83	L-LC	A
31	CVC	CVC0000001168	3	BJ	SW	MSG0008	04	S	02.00	0.343	PT	NPT 04		RCF03		CGT	08/17/83	L-LC #145	A
31	CVC	CVC0000001237A	3	BJ	SW	MSG0008	05	S	02.00	0.343	PT	NPT 04		RCF43		GDF	08/17/83	L-LC #145	A
31	CVC	CVC0000001238	3	BJ	SW	MSG0008	05	S	02.00	0.343	PT	NPT 04		RCF45		GDF	08/17/83	L-LC	A
31	CVC	CVC0000001245	3	BJ	SW	MSG0008	05	S	02.00	0.343	PT	NPT 04		RCF424		GDF	08/17/83	L-LC	A
31	CVC	CVC00000012454	3	BJ	SW	MSG0008	05	S	02.00	0.343	PT	NPT 04		RCF444		GDF	08/17/83	L-LC #145	A
31	CVC	CVC0000001249	3	BJ	SW	MSG0008	05	S	02.00	0.343	PT	NPT 04		RCF577		MSE	09/22/83	L-449 #145	A
31	CVC	CVC0000001249	3	BJ	SW	MSG0008	05	S	02.00	0.343	PT	NPT 04		RCF49		JEF	09/02/83	L-449	A
31	CVC	CVC0000001252	3	BJ	SW	MSG0008	05	S	02.00	0.343	PT	NPT 04		RCF45		MSE	08/17/83	L-RW #145	A
31	CVC	CVC0000001253	3	BJ	SW	ISI0009	05	S	02.00	0.343	PT	NPT 04		RCF78		LE4	07/25/83	L-RW NOI#SQ0008	R
31	CVC	CVC0000001253	1	BJ	SW	ISI0009	05	S	02.00	0.343	PT	NPT 04		RCF29		GKL	09/15/83	L-RW	A
31	CVC	CVC0000001256	3	BJ	SW	MSG0008	05	S	02.00	0.343	PT	NPT 04		RCF46		MSE	08/17/83	L-RW #145	A
01	CVC	CVC00010578C	3	BK2	FB	MSG0009	01	S	01.50	0.281	VT	NVT 01		RCF52		RMB	09/15/83	L-LC NOI#SQ0045	R
01	CVC	CVC00010578C	1	BK2	FB	MSG0009	01	S	01.50	0.281	VT	NVT 01		R1109		RMB	09/12/83	L-LC NOI#SQ0080 USE-A5-I S USQ083-26	A
31	CVC	CVC00012333C	3	BK2	FB	MSG0009	05	S	01.50	0.281	VT	NVT 01		RCF26		RMB	08/25/83	L-LC #145	A
31	CVC	RCH0000000017	3	BK2	RS	MSG0012	02	S	02.00	0.343	VT	NVT 01		RCF57		RMB	08/27/83	L-LC	A
31	CVC	RCH0000000018	3	BK2	RS	MSG0012	02	S	02.00	0.343	VT	NVT 01		RCF68		RMB	09/27/83	L-LC	A
31	CVC	RCH0000000019	3	BK2	RS	MSG0012	02	S	02.00	0.343	VT	NVT 01		RCF69		RMB	09/27/83	L-LC	A
31	CVC	RCH0000000020	3	BK2	RS	MSG0012	02	S	02.00	0.343	VT	NVT 01		RCF70		RMB	09/27/83	L-LC	A
31	CVC	RCH0000000021	3	BK2	RS	MSG0012	02	S	02.00	0.343	VT	NVT 01		RCF71		RMB	09/27/83	L-LC	A
31	CVC	RCH0000000022	3	BK2	RS	MSG0012	02	S	02.00	0.343	VT	NVT 01		RCF72		RMB	09/27/83	L-LC	A
31	CVC	RCH0000000033	3	BK2	RS	MSG0012	02	S	02.00	0.343	VT	NVT 01		RCF05		MSE	03/20/83	L-LC	A
31	CVC	62-0000000657	3	BK2	VB	ISI0009	02	S	02.00	0.438	VT	NVT 01		RCF19		RMB	09/15/83	L-LC	A

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 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 DAVIS, TENNESSEE  
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C Y C	SYSTM	EXAM AREA	R E #	T C P	DRAWING #	S H T	M #	SIZE	THICK	METH	NDE METH	PRGC NO.	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S			
01	CVC	62-0000000660	0	B62	W3	ISI0009	07	S	03.00	0.438	UT	NUT	01		RO553		RMB 09/15/83	L-LC NOI#SQ0046 USE-AS-I S USQD#83-27	A			
01	ELKR	ELKX0000000001	0	CA	4W	ISI0057	01	S	09.50	0.750	PT	NPT	04		RO667		HJO 09/21/83	L-RBAR PT TO SUPPLEMENT UT DUE TO SCAN LIMITATION	A			
														UT45	SQUT27	SQ	07	RO554	CO024	LB4 08/17/83	L-RBAR LMT-SCAN 4 NO SCAN S SEE R-0567 FOR PT	A
01	FD	FDF00000000007	0	CF	DW	CHM2403	01	C	16.00	0.843	RT	NRT	01		R1078		NR3 09/15/83	L-WVR IEB#79-13 WELDBASE METAL	A			
01	FD	FDF00000000010	0	CF	DW	CHM2403	01	C	16.00	0.843	RT	NRT	01		R1087		NR3 09/15/83	L-LC IEB#79-13 WELDBASE METAL	A			
01	FD	FDF00000000011	0	CF	DW	CHM2403	01	C	16.00	0.574	RT	NRT	01		R1085		NR9 09/15/83	L-LC IEB#79-13 WELDBASE METAL	A			
														UT45	SQUT27	SQ	04	R1008	CO071	FS4 09/02/83	L-LC RT EVALUATION	A
01	FD	FDF00000000021	0	CF	DW	CHM2403	01	C	16.00	0.843	RT	NRT	01		R1090		NR3 09/15/83	L-LC IEB#79-13 WELDBASE METAL	A			
														UT45	SQUT27	SQ	04	R1097	CO071	FS4 09/02/83	L-LC RT EVALUATION	A
01	FD	FDF00000000022	0	CF	DW	CHM2403	01	C	16.00	0.674	RT	NRT	01		R1088		NR3 09/15/83	L-LC IEB#79-13 WELDBASE METAL	A			
														UT45	SQUT27	SQ	04	R1077	CO071	FS4 09/02/83	L-LC RT EVALUATION	A
01	FD	FDF00000000030	0	CF	DW	CHM2403	02	C	16.00	0.843	RT	NRT	01		R1002		NR3 09/15/83	L-LC IEB#79-13 WELDBASE METAL	A			
														UT45	SQUT27	SQ	04	R1095	CO070	LB4 09/07/83	L-LC RT EVALUATION	A
01	FD	FDF00000000031	0	CF	DW	CHM2403	02	C	16.00	0.574	RT	NRT	01		R1084		NR3 09/15/83	L-LC IEB#79-13 WELDBASE METAL	A			
														UT45	SQUT27	SQ	04	R1095	CO070	LB4 09/07/83	L-LC RT EVALUATION	A
01	FD	FDF00000000040	0	CF	DW	CHM2403	02	C	16.00	0.843	RT	NRT	01		R1092		NR3 09/12/83	L-LC IEB#79-13 WELDBASE METAL	A			
01	FD	FDF00000000041	0	CF	DW	CHM2403	02	C	16.00	0.574	RT	NRT	01		R1094		NR3 09/12/83	L-LC IEB#79-13 WELDBASE METAL	A			
01	FD	FDF00000000010/11	0	CF	3M	CHM2403	01	C	16.00	0.843	RT	NRT	01		R1095		NR3 09/02/83	L-LC IEB#79-13 SPOOL PIEC E	A			
01	FD	FDF00000000021/22	0	CF	3M	CHM2403	01	C	16.00	0.843	RT	NRT	01		R1089		NR3 09/15/83	L-LC IEB#79-13 WELDBASE METAL	A			
														UT45	SQUT27	SQ	04	R1097	CO071	FS4 09/02/83	L-LC RT EVALUATION	A
01	FD	FDF000130/131	0	CF	3M	CHM2403	02	C	16.00	0.843	RT	NRT	01		R1093		NR9 09/15/83	L-LC IEB#79-13 SPOOL PIEC E	A			

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 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 DAISY, TENNESSEE  
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SYSTM	EXAM AREA	R E CAT	T Y P	S M H A	T SIZE	THICK	NDE METH	PROC NO.	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S
01 FD	FDF000140/141	0 CE	BM	SM	2403	02 C	16.00	0.843	RT	NRT	01		R1093	NR3 09/12/83 L-LC IE8879-13 SPOOL PIECE	A
01 FD	FDF000000200	0 CE	RS	MS	60013	01 C	16.00	0.843	VT	NVT	01		RC440	JCG 09/11/83 L-LC	A
01 FD	FDF000000201	0 CE	RS	MS	60015	01 C	16.00	0.843	VT	NVT	01		RC441	JCG 03/11/83 L-LC	A
01 FD	FDF000000202	0 CE	VM	MS	60016	01 C	16.00	0.843	VT	NVT	01		RC442	JCG 09/11/83 L-LC	A
01 FD	FDF000000203	0 CE	MS	MS	60015	01 C	16.00	0.843	VT	NVT	01		RC443	JCG 03/11/83 L-LC MS AND VS SUPPORT	A
01 FD	FDF000000204	0 CE	MS	MS	60015	01 C	16.00	0.843	VT	NVT	01		RC066	LBH 07/25/83 L-WVR	A
01 FD	FDF000000205	0 CE	RS	MS	60016	01 C	16.00	0.843	VT	NVT	01		RC064	LBH 07/26/83 L-WVR	A
01 FD	FDF000000206	0 CE	MS	MS	60016	01 C	16.00	0.843	VT	NVT	01		RC065	LBH 07/25/83 L-WVR	A
01 FD	FDF000000207	0 CE	RS	MS	60015	01 C	16.00	0.839	VT	NVT	01		RC063	LBH 07/25/83 L-WVR	A
01 FD	FDF000000208	0 CE	RS	MS	60015	01 C	16.00	0.838	VT	NVT	01		RC062	LBH 07/26/83 L-WVR	A
01 FD	FDF000000240	0 CE	RS	MS	60016	02 C	16.00	0.843	VT	NVT	01		RC457	JCG 03/12/83 L-LC NDI#500037	R
01 FD	FDF000000240	1 CE	RS	MS	60016	02 C	16.00	0.843	VT	NVT	01		RC357	RMB 06/27/83 L-LC	A
01 FD	FDF000000241	0 CE	RS	MS	60015	02 C	16.00	0.843	VT	NVT	01		RC468	JCG 09/12/83 L-LC	A
01 FD	FDF000000242	0 CE	MS	MS	60015	02 C	16.00	0.843	VT	NVT	01		RC459	JCG 09/12/83 L-LC	A
01 FD	FDF000000243	0 CE	MS	MS	60016	02 C	16.00	0.843	VT	NVT	01		RC470	JCG 03/12/83 L-LC MSEVS SUPPORT	A
01 FD	FDF000000244	0 CE	MS	MS	60016	02 C	16.00	0.843	VT	NVT	01		RC035	LEW 07/25/83 L-EVR	A
01 FD	FDF000000245	0 CE	IA	MS	60016	02 C	16.00	0.843	MT	NMT	02		RC705	JCG 09/25/83 L-EVR	A
01 FD	FDF000000245	0 CE	RS	MS	60015	02 C	16.00	0.843	VT	NVT	01		RC053	LEW 07/25/83 L-EVR NDI#500006	R
01 FD	FDF000000245	1 CE	RS	MS	60015	02 C	16.00	0.843	VT	NVT	01		RC701	MSE 06/25/83 L-EVR	A
01 FD	FDF000000245	0 CE	RS	MS	60015	02 C	16.00	0.843	VT	NVT	01		RC039	LEW 07/25/83 L-EVR	A
01 FD	FDF000000247	0 CE	RS	MS	60015	02 C	16.00	0.838	VT	NVT	01		RC039	LEW 07/25/83 L-EVR	A
01 FD	FDF000000281	0 CE	RS	MS	60016	02 C	16.00	0.843	VT	NVT	01		RC463	JCG 09/12/83 L-LC	A
01 FD	FDF000000282	0 CE	MS	MS	60015	02 C	16.00	0.843	VT	NVT	01		RC464	JCG 08/12/83 L-LC	A
01 FD	FDF000000285	0 CE	MS	MS	60016	02 C	16.00	0.843	VT	NVT	01		RC455	JCG 08/12/83 L-LC MSEVS SUPPORT	A

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SYSTH	EXAM AREA	R E C O R D #	C A T	T P	S M H A	DRAWING #	T SIZE	THICK	METH	NDE	PRDC NO.	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S
01 FD	FDH0000003284	0	CE	RS	MSG0015	02	C	16.00	0.343	VT	NVT 01		R0465		JCS	08/12/83	L-LC	A
01 FD	FDH0000003285	0	CE	RS	MSG0015	02	C	16.00	0.343	VT	NVT 01		R0433		JCS	08/11/83	L-EVR	A
01 FD	FDH0000003286	0	CE	RS	MSG0015	02	C	16.00	0.343	VT	NVT 01		R0414		JCS	08/10/83	L-EVR	A
01 FD	FDH0000003287	0	CC	IA	MSG0016	02	C	10.00	0.739	MT	NMT 02		R0705		JCS	03/25/83	L-EVR	A
01 FD	FDH0000003287	0	CE	RS	MSG0016	02	C	18.00	0.739	VT	NVT 01		R0415		JCS	08/10/83	L-EVR	A
01 FD	FDH0000003288	0	CC	IA	MSG0015	02	C	18.00	0.739	MT	NMT 02		R0707		JCS	08/25/83	L-EVR	A
01 FD	FDH0000003288	0	CE	RS	MSG0016	02	C	18.00	0.739	VT	NVT 01		R0416		JCS	03/10/83	L-EVR	A
01 FD	FDH0000003290	0	CE	RS	MSG0015	01	C	16.00	0.343	VT	NVT 01		R0446		JCS	08/11/83	L-LC	A
01 FD	FDH0000003291	0	CE	RS	MSG0016	01	C	16.00	0.343	VT	NVT 01		R0447		JCS	08/11/83	L-LC	A
01 FD	FDH0000003292	0	CE	VS	MSG0015	01	C	16.00	0.343	VT	NVT 01		R0448		JCS	08/11/83	L-LC NOI#SQ0035	R
01 FD	FDH0000003292	1	CE	RS	MSG0016	01	C	15.00	0.343	VT	NVT 01		R0968		RM9	03/27/83	L-LC	A
01 FD	FDH0000003293	0	CE	RS	MSG0015	01	C	16.00	0.343	VT	NVT 01		R0449		JCS	08/11/83	L-LC VS E MS SUPPORT	A
01 FD	FDH0000003294	0	CE	RS	MSG0016	01	C	16.00	0.343	VT	NVT 01		R0061		LP1	07/25/83	L-WUR	A
01 FD	FDH0000003295	0	CE	RS	MSG0015	01	C	16.00	0.343	VT	NVT 01		R0360		LB1	07/25/83	L-WUR	A
01 FD	FDH0000003295	0	CE	RS	MSG0015	01	C	16.00	0.343	VT	NVT 01		R0759		LB1	07/25/83	L-WUR	A
01 FD	FDH0000003297	0	CE	RS	MSG0016	01	C	16.00	0.739	VT	NVT 01		R0758		LB1	07/25/83	L-WUR	A
01 FD	FDH0000003293	0	CE	RS	MSG0016	01	C	18.00	0.739	VT	NVT 01		R0757		LB1	07/25/83	L-WUR	A
01 FD	FDH0000003297	0	CE	RS	MSG0016	01	C	18.00	0.739	VT	NVT 01		R0756		LB1	07/25/83	L-WUR	A
01 FD	FDS40000030054	0	CF	CH	CHM2403	01	C	16.00	0.343	UT45	SQUT27 SQ 04	R0390 C0011		JCS	03/07/83	L-WUR IEB#79-13 WELDBASE METAL	A	
01 FD	FDS40000030154	0	CF	CH	CHM2403	01	C	16.00	0.343	UT45	SQUT27 SQ 04	R0391 C0011		JCS	03/07/83	L-WUR IEB#79-13 WELDBASE METAL	A	
01 FD	FDS4000003017A	0	CF	CH	CHM2403	02	C	16.00	0.343	MT	NMT 02		R0718		JCS	07/01/83	L-EVR SURFACE EXAM TO SUPPLEMENT UT45 EXAM	A
															JCS	03/25/83	L-EVR IEB#79-13 WELDBASE METAL	A
01 FD	FDS40000030254	0	CF	CH	CHM2403	02	C	16.00	0.343	UT45	SQUT27 SQ 04	R0735 C0045		JCS	08/25/83	L-EVR IEB#79-13 WELDBASE METAL	A	

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TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 DAISY, TENNESSEE  
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SYSTM	EXAM AREA	R E C	T C O D Y	S M H A	DRAWING #	T SIZE	THICK	NOE METH	PRDC NO.	CAL BLCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S
01 FD	FDS0000000009	0 CF	DW	CHM2403	01 C	16.00	0.343	RT	NRT 01		R1091			NR3 03/02/83	L-LC IEB#79-13	A
01 FD	FDS0000000013	0 CF	DW	CHM2403	01 C	16.00	0.343	RT	NRT 01		R1081			T85 09/04/83	L-WVR IEB#79-13 WELDERASE METAL	A
											UT45 SQU27 SQ 04	R1246 C0074		LBH 03/23/83	L-WVR IEB#79-13 BASEMETAL RT EVALUATION	A
01 FD	FDS0000000021	0 CF	DW	CHM2403	02 C	16.00	0.343	RT	NRT 01		R1079			NR3 03/02/83	L-LC IEB#79-13 WELDERASE METAL	A
01 FD	FDS0000000026	0 CF	DW	CHM2403	02 C	16.00	0.343	RT	NRT 01		R1090			NR3 03/02/83	L-LC IEB#79-13 WELDERASE METAL	A
01 FD	FDS0000000115	0 CF	LW	CHM2403	01 C	19.00	0.938	MT	NMT 02		R0297			JCG 03/04/83	L-WVR	A
											UT45 SQU27 SQ 18	R0411 C0012		JCG 03/03/83	L-WVR	A
01 LHX	LHX0000000001	0 CE	DS	ISI0068	01 S	16.00	0.375	VT	NVT 01		R0517			LBH 09/15/83	L-AB714	A
01 LHX	LHX0000000001	0 CA	NW	ISI0068	01 S	16.00	0.375	VT	NVT 01		UT45 SQU27 BNP17	R0526 C0020		LBH 08/16/83	L-AB714 LMT-SCANS 354; NO PT PERFORMED; RFR	A
01 MS	MSF0000000010	0 CF	DW	ISI0015	01 C	32.00	1.380	MT	NMT 02		R0299			JCG 03/04/83	L-WVR	A
											UT45 SQU27 SQ 05	R0357 C0017		JCG 03/03/83	L-WVR	A
01 MS	MSF0000000041	0 CF	DW	ISI0015	01 C	32.00	1.270	MT	NMT 02		R0299			JCG 03/04/83	L-WVR	A
											UT45 SQU27 SQ 05	R0356 C0007		JCG 03/03/83	L-WVR	A
01 MS	MSH0000000003	0 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R1127			MSE 03/14/83	L-LC	A
01 MS	MSH0000000001	0 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R0492			JCG 03/12/83	L-LC NOI#500041	R
01 MS	MSH0000000001	1 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R1129			MSE 03/14/83	L-LC	A
01 MS	MSH0000000002	0 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R0444			JCG 03/11/83	L-LC	A
01 MS	MSH0000000003	0 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R1201			LBH 03/23/83	L-LC	A
01 MS	MSH0000000004	0 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R0494			JCG 03/12/83	L-LC	A
01 MS	MSH0000000005	0 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R0445			JCG 03/11/83	L-LC	A
01 MS	MSH0000000005	0 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R0295			RM3 07/25/83	L-WVR	A
01 MS	MSH0000000007	0 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R0281			RM3 07/25/83	L-WVR	A
01 MS	MSH0000000002	0 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R0292			RM3 07/25/83	L-WVR NDT#SQ0009	P
01 MS	MSH0000000003	1 CE	VS	MS60017	01 C	32.00	1.151	VT	NVT 01		R0399			MSE 03/25/83	L-WVR	A

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TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SECUDYAH NUCLEAR PLANT  
 P. O. BOX 2000 DAISY, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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SYSTEM	EXAM AREA	R E C O D Y	T	S H A	M	NDE	PRDC	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S		
01 MS	MSH0000000309	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0799	RM3	07/25/83	L-WVR	A	
01 MS	MSH0000000310	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0794	RM3	07/25/83	L-WVR	NOI#SQ0011	R
01 MS	MSH0000000311	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R1229	LR4	07/23/83	L-WVR	A	
01 MS	MSH0000000312	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0795	RM3	07/25/83	L-WVR	NOI#SQ0012	R
01 MS	MSH0000000313	1 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R1050	RM3	07/07/83	L-WVR	A	
01 MS	MSH0000000314	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0793	JCS	03/10/83	L-WVR	A	
01 MS	MSH0000000315	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0790	RM3	07/25/83	L-WVR	A	
01 MS	MSH0000000316	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0779	RM3	07/25/83	L-WVR	A	
01 MS	MSH0000000317	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0084	RM3	07/25/83	L-WVR	A	
01 MS	MSH0000000318	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0097	RM3	07/25/83	L-WVR	NOI#SQ0014	R
01 MS	MSH0000000319	1 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R1749	RM3	07/07/83	L-WVR	A	
01 MS	MSH0000000320	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0793	RM3	07/25/83	L-WVR	A	
01 MS	MSH0000000321	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0092	RM3	07/25/83	L-LC	A	
01 MS	MSH0000000322	0 CE	MS	MSG0017	01 C	32.00	1.151	VT	NVT	01	R0472	JCS	03/12/83	L-LC	A	
01 MS	MSH0000000323	0 CE	MS	MSG0017	02 C	32.00	1.151	VT	NVT	01	R1129	MSE	07/14/83	L-LC	A	
01 MS	MSH0000000324	0 CE	MS	MSG0017	02 C	32.00	1.151	VT	NVT	01	R0476	JCS	03/12/83	L-LC	NOI#SQ0039	R
01 MS	MSH0000000325	1 CE	MS	MSG0017	02 C	32.00	1.151	VT	NVT	01	R1150	MSE	07/14/83	L-LC	A	
01 MS	MSH0000000326	0 CE	MS	MSG0017	02 C	32.00	1.151	VT	NVT	01	R0477	JCS	03/12/83	L-LC	NOI#SQ0039	R
01 MS	MSH0000000327	1 CE	MS	MSG0017	02 C	32.00	1.151	VT	NVT	01	R1710	LR4	07/25/83	L-LC	A	
01 MS	MSH0000000328	0 CE	MS	MSG0017	02 C	32.00	1.151	VT	NVT	01	R0478	JCS	03/12/83	L-LC	A	
01 MS	MSH0000000329	0 CE	MS	MSG0017	02 C	32.00	1.151	VT	NVT	01	R1142	BSR	07/15/83	L-LC	A	
01 MS	MSH0000000330	0 CE	MS	MSG0017	02 C	32.00	1.151	VT	NVT	01	R0478	JCS	03/12/83	L-EVR	A	
01 MS	MSH0000000331	0 CE	MS	MSG0017	02 C	32.00	1.151	VT	NVT	01	R0734	LR4	07/25/83	L-EVR	A	
01 MS	MSH0000000332	0 CE	MS	MSG0017	02 C	32.00	1.151	VT	NVT	01	R0043	LR4	07/25/83	L-EVR	A	

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TENNESSEE VALLEY AUTHORITY, DIVISION OF NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2003 DAVIS, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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SYSTM	EXAM AREA	R E C O D	I F	S H F A	N D E	P R O C	C A L B L O C K	R E P T	C A L R E P T	I N P	D A T E	C O M M E N T S	R E S					
		#	C A T	D R A W I N G	#	T	S I Z E	T H I C K	M E T H	N O.	N O.	N O.	N O.	P				
01 MS	MSH0000000349	0	CC	IA	MSG0017	02	C	32.00	1.151	MT	NMT	02	R0709	JCG	09/25/83	L-EVR	A	
01 MS	MSH0000000347	0	CE	RS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0399	JCG	09/10/83	L-EVR	A	
01 MS	MSH0000000350	0	CE	MS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0340	LEW	07/25/83	L-EVR	A	
01 MS	MSH0000000351	0	CE	VS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0400	JCS	09/10/83	L-EVR	A	
01 MS	MSH0000000352	0	CE	YS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0744	LEW	07/25/83	L-EVR	A	
01 MS	MSH0000000353	0	CE	YS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0434	JCS	03/11/83	L-EVR	NOT#SQ0034	R
01 MS	MSH0000000353	1	CE	YS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R1144	MSE	09/15/83	L-EVR	A	
01 MS	MSH0000000354	0	CE	RS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0417	JCS	09/10/83	L-EVR	A	
01 MS	MSH0000000355	0	CE	RS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0419	JCS	09/10/83	L-EVR	A	
01 MS	MSH0000000300	0	CE	YS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R1131	MSE	07/14/83	L-LC	A	
01 MS	MSH0000000301	0	CE	YS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0471	JCS	09/10/83	L-LC	A	
01 MS	MSH0000000302	0	CE	VS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0473	JCS	09/10/83	L-LC	A	
01 MS	MSH0000000303	0	CE	YS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R1132	MSE	09/14/83	L-LC	A	
01 MS	MSH0000000304	0	CE	VS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0474	JCS	09/10/83	L-LC	A	
01 MS	MSH0000000305	0	CE	RS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0475	JCS	09/10/83	L-LC	A	
01 MS	MSH0000000305	0	CE	VS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0435	JCS	09/11/83	L-EVR	A	
01 MS	MSH0000000307	0	CE	YS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0342	LEW	07/25/83	L-EVR	A	
01 MS	MSH0000000306	0	CE	VS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0336	LEW	07/25/83	L-EVR	A	
01 MS	MSH0000000307	0	CC	IA	MSG0017	02	C	32.00	1.151	MT	NMT	02	R0709	JCS	09/25/83	L-EVR	A	
01 MS	MSH0000000309	0	CE	RS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0419	JCS	09/10/83	L-EVR	A	
01 MS	MSH0000000300	0	CE	VS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0733	LEW	07/25/83	L-EVR	NOT#SQ0005	R
01 MS	MSH0000000300	1	CE	VS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0700	MSE	08/25/83	L-EVR	A	
01 MS	MSH0000000391	0	CE	YS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0341	LEW	07/25/83	L-EVR	A	
01 MS	MSH0000000392	0	CE	YS	MSG0017	02	C	32.00	1.151	VT	NVT	01	R0737	LEW	07/25/83	L-EVR	A	

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TENNESSEE VALLEY AUTHORITY, DIVISION OF NUCLEAR POWER  
1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
SEQUOYAH NUCLEAR PLANT  
P. O. BOX 2000 DAVIS, TENNESSEE  
UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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SYSTEM	EXAM AREA	R E C O D Y # CAT P	D R A W I N G #	S H A P E	M A T E R I A L	N D E	P R O C NO.	C A L B L O C K NO.	C A L R E P T NO.	I N P	D A T E	C O M M E N T S	R E S	
01 MS	MSH000000433	0 CE	45	MS60017	02	C	32.00	1.151	VT	NVT	01	R0436	JCG 03/11/83 L-EUR	A
01 MS	MSH000000434	0 CE	RS	MS60017	02	C	32.00	1.151	VT	NVT	01	R0420	JCG 03/10/83 L-EUR	A
01 MS	MSH000000435	0 CE	RS	MS60017	02	C	32.00	1.151	VT	NVT	01	R0421	JCG 03/10/83 L-EUR	A
01 MS	MSH000000436	0 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R0123	RMA 03/23/83 L-LC	A
01 MS	MSH000000437	0 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R0421	JCG 03/12/83 L-LC NOI#500040	R
01 MS	MSH000000438	1 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R1135	MSE 03/14/83 L-LC	A
01 MS	MSH000000439	0 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R0440	JCG 03/12/83 L-LC	A
01 MS	MSH000000440	0 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R0493	JCG 03/12/83 L-LC NOI#500042	R
01 MS	MSH000000441	1 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R1134	MSE 03/14/83 L-LC	A
01 MS	MSH000000442	0 CE	RS	MS60017	01	C	32.00	1.151	VT	NVT	01	R0450	JCG 03/11/83 L-LC	A
01 MS	MSH000000443	0 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R0451	JCG 03/11/83 L-LC	A
01 MS	MSH000000444	0 CE	RS	MS60017	01	C	32.00	1.151	VT	NVT	01	R0452	JCG 03/11/83 L-LC	A
01 MS	MSH000000445	0 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R0479	JCG 03/12/83 L-WUR	A
01 MS	MSH000000446	0 CE	RS	MS60017	01	C	32.00	1.151	VT	NVT	01	R0100	LBH 07/27/83 L-WUR	A
01 MS	MSH000000447	0 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R0093	LBH 07/27/83 L-WUR NOI#500010	R
01 MS	MSH000000448	1 CE	45	MS60003	01	C	32.00	1.151	VT	NVT	01	R1202	LBH 03/22/83 L-WUR	A
01 MS	MSH000000449	0 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R0091	LBH 07/27/83 L-WUR	A
01 MS	MSH000000450	0 CE	RS	MS60017	01	C	32.00	1.151	VT	NVT	01	R0098	RMA 07/25/83 L-WUR NOI#500015	R
01 MS	MSH000000451	1 CE	RS	MS60017	01	C	32.00	1.151	VT	NVT	01	R1203	LBH 03/22/83 L-WUR	A
01 MS	MSH000000452	0 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R0106	RMA 07/25/83 L-WUR	A
01 MS	MSH000000453	0 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R0108	RMA 07/25/83 L-WUR NOI#500013	R
01 MS	MSH000000454	1 CE	45	MS60017	01	C	32.00	1.151	VT	NVT	01	R1249	RMA 03/09/83 L-WUR	A
01 MS	MSH000000455	0 CE	RS	MS60017	01	C	32.00	1.151	VT	NVT	01	R0289	RMA 07/25/83 L-WUR	A
01 MS	MSH000000456	0 CE	RS	MS60017	01	C	32.00	1.151	VT	NVT	01	R0097	RMA 07/25/83 L-WUR	A

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1750 CHESTNUT STREET TOWER II, DIVISION II - NUCLEAR POWER  
SECUCYAH NUCLFR PLANT, CHATTANOOGA, TENNESSEE 37402  
P. O. BOX 2000, DAVIS, TENNESSEE  
UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

SYSTEM	EXAM AREA	CDY CAT P	T S M H A	DRAWING #	T SIZE	THICK METH	NDE PRQC NO.	BLACK REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS
01 RC	RCH0000000015	0 BK1 IA	M560013 11 5	06.00	0.718	PT	NPT 04	R0204			03/20/83	L-LC
01 RC	RCH0000000015	0 BK2 CF	M560013 11 5	06.00	0.718	VT	NVT 01	R1007			09/05/83	L-LC
01 RC	RCH0000000015	0 BK2 RS	M560013 11 5	06.00	0.718	VT	NVT 01	R1008			03/05/83	L-LC
01 RC	RCH0000000025	0 BK2 RS	M560013 11 5	04.00	0.459	VT	NVT 01	R1009			09/05/83	L-LC
01 RC	RCH0000000024	0 BK2 MS	M560013 11 5	04.00	0.438	VT	NVT 01	R1010			03/05/83	L-LC
01 RC	RCH0000000025	0 BK2 US	M560013 11 5	04.00	0.458	VT	NVT 01	R1011			09/05/83	L-LC
01 RC	RCH0000000025	0 BK2 MS	M560013 11 5	04.00	0.458	VT	NVT 01	R1012			07/05/83	L-LC
01 RC	RCH0000000027	0 BK2 RS	M560013 11 5	04.00	0.458	VT	NVT 01	R1013			03/05/83	L-LC
01 RC	RCH0000000029	0 BK2 MS	M560013 11 5	04.00	0.458	VT	NVT 01	R1014			03/05/83	L-LC
01 RC	RCH0000000029	0 BK2 RS	M560013 11 5	04.00	0.458	VT	NVT 01	R1015			07/05/83	L-LC
01 RC	RCH0000000030	0 BK2 MS	M560013 11 5	04.00	0.458	VT	NVT 01	R1016			07/05/83	L-LC
01 RC	RCH0000000031	0 BK2 MS	M560013 11 5	04.00	0.459	VT	NVT 01	R1017			03/05/83	L-LC
01 RC	RCH0000000032	0 BK2 US	M560013 11 5	04.00	0.458	VT	NVT 01	R1018			09/05/83	L-LC
01 RC	RCH0000000033	0 BK2 RS	M560013 11 5	04.00	0.458	VT	NVT 01	R1019			03/05/83	L-LC
01 RC	RCH0000000034	0 BK2 RS	M560013 11 5	04.00	0.458	VT	NVT 01	R1020			07/05/83	L-LC
01 RC	RCH0000000035	0 BK2 MS	M560013 11 5	04.00	0.458	VT	NVT 01	R1021			09/05/83	L-LC
01 RC	RCH0000000035	0 BK2 RS	M560013 11 5	04.00	0.458	VT	NVT 01	R1022			03/05/83	L-LC
01 RC	RCH0000000037	0 BK2 RS	M560013 11 5	04.00	0.458	VT	NVT 01	R1023			09/05/83	L-LC
01 RC	RCH0000000039	0 BK2 RS	M560013 11 5	06.00	0.718	VT	NVT 01	R1024			03/05/83	L-LC
01 RC	RCH0000000065	0 BK2 RS	M560013 12 5	06.00	0.718	VT	NVT 01	R1025			03/27/83	L-LC
01 RC	RCH0000000055	0 BK1 IA	M560013 12 5	06.00	0.718	PT	NPT 04	R0261			03/02/83	L-LC
01 RC	RCH0000000065	0 BK2 MS	M560013 12 5	06.00	0.718	VT	NVT 01	R1027			09/27/83	L-LC
01 RC	RCH0000000071	0 BK2 RS	M560013 12 5	06.00	0.718	VT	NVT 01	R1028			03/23/83	L-LC
01 RC	RCH0000000082	0 BK2 RS	M560013 12 5	06.00	0.718	VT	NVT 01	R1029			03/27/83	L-LC



12/15/83

TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
SEQUOYAH NUCLEAR PLANT  
P. O. BOX 2002 DAVIS, TENNESSEE  
UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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SYSTEM	EXAM AREA	R E C D	T Y P E	S H A	M A	SIZE	THICK	METH	NDE	PROC	CAL	REPT	DATE	COMMENTS
01 RC	RCH0000000095	0	BK2	RS	M560013	12	5	06.00	0.717	VT	NVT	01	P1219	MSE 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	12	5	07.00	0.438	VT	NVT	01	P1220	LBM 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	RS	M560013	12	5	03.00	0.433	VT	NVT	01	P1235	MSE 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	12	5	03.00	0.438	VT	NVT	01	P1222	MSE 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	12	5	03.00	0.438	VT	NVT	01	P1223	LBM 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	12	5	05.00	0.438	VT	NVT	01	P1221	LBM 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	12	5	06.00	0.713	VT	NVT	01	P1224	MSE 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	RS	M560013	12	5	02.00	0.435	VT	NVT	01	P1242	MSE 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	12	5	03.00	0.438	VT	NVT	01	P1225	MSE 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	12	5	03.00	0.438	VT	NVT	01	P1245	MSE 09/27/83 L-LC
01 RC	RCH0000000095	0	BK1	IA	M560013	12	5	06.00	0.713	PT	NPT	04	R0252	RMS 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	12	5	06.00	0.713	VT	NVT	01	P1234	MSE 09/27/83 L-LC
01 RC	RCH0000000095	0	BK1	IA	M560013	12	5	06.00	0.718	PT	NPT	04	R0263	RMS 09/27/83 L-LC
01 RC	RCH0000000095	0	BK2	RF	M560013	12	5	05.00	0.718	VT	NVT	01	P1206	BGF 09/24/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	01	5	02.00	0.343	VT	NVT	01	R0244	MSE 09/02/83 L-LC
01 RC	RCH0000000095	0	BK2	RS	M560013	01	5	02.00	0.343	VT	NVT	01	P0245	MSE 09/02/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	01	5	02.00	0.343	VT	NVT	01	R0265	MSE 09/02/83 L-LC N31P500063 USE-A5-I A S USQ083-28
01 RC	RCH0000000095	0	BK2	MS	M560013	01	5	02.00	0.343	VT	NVT	01	R0247	MSE 09/02/83 L-LC
01 RC	RCH0000000095	0	BK2	RS	M560013	01	5	02.00	0.343	VT	NVT	01	R0248	MSE 09/02/83 L-LC
01 RC	RCH0000000095	0	BK2	RS	M560013	01	5	02.00	0.343	VT	NVT	01	P0242	MSE 09/02/83 L-LC
01 RC	RCH0000000095	0	BK2	RS	M560013	01	5	03.00	0.438	VT	NVT	01	P0250	MSE 09/02/83 L-LC
01 RC	RCH0000000095	0	BK2	RS	M560013	02	5	03.00	0.438	VT	NVT	01	P0251	MSE 09/02/83 L-LC
01 RC	RCH0000000095	0	BK2	RS	M560013	02	5	05.00	0.438	VT	NVT	01	R0252	MSE 09/02/83 L-LC
01 RC	RCH0000000095	0	BK2	MS	M560013	02	5	02.00	0.438	VT	NVT	01	R0253	MSE 09/02/83 L-LC

12/15/83

MISSISSIPPI VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
1750 CHESTNUT STREET FOWER II, CHATTANOOGA, TENNESSEE 37402  
SEQUOYAH NUCLEAR PLANT  
P. O. BOX 2000  
DUNSMITH, TENNESSEE  
UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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EXAM AREA	SYSTEM	COO Y	CAT ?	DEFINING #	S H A	SIZE	THICK	METH	NDE	PROC	CAL BLOCK NO.	REPT NO.	CAL I N P	DATE	COMMENTS	R E S	
RCH0000001141	01 RC	0	BK2	15	M560013	02	5	05.00	0.439	VT	NUT	01	RC754	MSE	09/02/83	L-LC	A
RCH0000001142	01 RC	0	BK2	15	M560013	02	5	05.00	0.439	VT	NUT	01	RC755	MSE	09/02/83	L-LC	A
RCH0000001143	01 RC	0	BK2	15	M560013	02	5	05.00	0.439	VT	NUT	01	RC756	MSE	09/02/83	L-LC	A
RCH0000001144	01 RC	0	BK2	15	M560013	02	5	05.00	0.439	VT	NUT	01	RC757	MSE	09/02/83	L-LC	A
RCH0000001145	01 RC	0	BK2	15	M560013	02	5	05.00	0.439	VT	NUT	01	RC758	MSE	09/02/83	L-LC	A
RCH0000001146	01 RC	0	BK2	15	M560013	02	5	05.00	0.439	VT	NUT	01	RC759	MSE	09/02/83	L-LC	A
RCH0000001147	01 RC	0	BK2	15	M560013	02	5	05.00	0.439	VT	NUT	01	RC760	MSE	09/02/83	L-LC	R
RCH0000001147	01 RC	1	BK2	15	M560013	02	5	05.00	0.439	VT	NUT	01	91121	RMS	09/14/83	L-LC	A
RCH0000001148	01 RC	0	BK2	15	M560013	01	5	02.00	0.343	VT	NUT	01	RC761	MSE	09/02/83	L-LC	A
RCH0000001151	01 RC	0	BK2	15	M560013	01	5	02.00	0.343	VT	NUT	01	90962	MSE	09/02/83	L-LC	A
RCH0000001155	01 RC	0	BK2	15	M560013	02	5	05.00	0.439	VT	NUT	01	RC763	MSE	09/02/83	L-LC	R
RCH0000001155	01 RC	1	BK2	15	M560013	02	5	05.00	0.439	VT	NUT	01	91122	RMS	09/14/83	L-LC	A
RCH0000001156	01 RC	0	BK2	15	M560013	01	5	02.00	0.343	VT	NUT	01	RC764	MSE	09/02/83	L-LC	A
RCH0000001181	01 RC	0	BK2	15	M560013	03	5	02.00	0.343	VT	NUT	01	90765	MDR	09/02/83	L-LC	A
RCH0000001182	01 RC	0	BK2	15	M560013	03	5	02.00	0.343	VT	NUT	01	RC766	MDR	09/02/83	L-LC	A
RCH0000001183	01 RC	0	BK2	15	M560013	03	5	02.00	0.343	VT	NUT	01	RC767	MDR	09/02/83	L-LC	A
RCH0000001184	01 RC	0	BK2	15	M560013	03	5	02.00	0.343	VT	NUT	01	RC768	MDR	09/02/83	L-LC	A
RCH0000001185	01 RC	0	BK2	15	M560013	03	5	02.00	0.343	VT	NUT	01	RC769	MDR	09/02/83	L-LC	A
RCH0000001185	01 RC	0	BK2	15	M560013	03	5	02.00	0.343	VT	NUT	01	RC770	MDR	09/02/83	L-LC	A
RCH0000001187	01 RC	0	BK2	15	M560013	04	5	02.00	0.439	VT	NUT	01	90771	MDR	09/02/83	L-LC	A
RCH0000001193	01 RC	0	BK1	14	M560013	04	5	02.00	0.439	PT	NPT	04	90729	GDF	03/11/83	L-LC	A
RCH0000001183	01 RC	0	BK2	15	M560013	04	5	02.00	0.439	VT	NUT	01	90772	MDR	09/02/83	L-LC	A
RCH0000001187	01 RC	0	BK2	15	M560013	04	5	02.00	0.439	VT	NUT	01	90773	MDR	09/02/83	L-LC	A
RCH0000001190	01 RC	0	BK2	15	M560013	04	5	02.00	0.439	VT	NUT	01	90774	MDR	09/02/83	L-LC	A

EXAM AREA	Y	S	M	H	A	THICK	METH	NO.	PRDC	BLCK	REPT	NO.	DATE	COMMENTS
RCH00000002191	0	BK2	RS	MS60013	04	S	03.00	0.438	VT	NVT	01	RC775	09/02/83	L-LC
RCH00000002192	0	BK2	MS	MS60013	04	S	03.00	0.439	VT	NVT	01	RC776	09/02/83	L-LC N018500065
RCH00000002193	1	BK2	MS	MS60013	04	S	02.00	0.439	VT	NVT	01	RC777	09/14/83	L-LC
RCH00000002194	0	BK2	RS	MS60013	04	S	02.00	0.438	VT	NVT	01	RC778	09/02/83	L-LC
RCH00000002195	0	BK2	MS	MS60013	04	S	03.00	0.438	VT	NVT	01	RC779	09/02/83	L-LC
RCH00000002197	0	BK2	MS	MS60013	04	S	03.00	0.438	VT	NVT	01	RC780	09/02/83	L-LC
RCH00000002198	0	BK2	MS	MS60013	04	S	02.00	0.443	VT	NVT	01	RC781	09/02/83	L-LC
RCH00000002199	0	BK2	MS	MS60013	04	S	02.00	0.443	VT	NVT	01	RC782	09/02/83	L-LC
RCH00000002200	0	BK2	RS	MS60013	04	S	02.00	0.443	VT	NVT	01	RC783	09/02/83	L-LC
RCH00000002215	0	BK2	MS	MS60013	04	S	03.00	0.438	VT	NVT	01	RC784	09/02/83	L-LC
RCH00000002231	0	BK2	MS	MS60013	04	S	03.00	0.438	VT	NVT	01	RC780	09/01/83	L-LC
RCH00000002232	0	BK2	MS	MS60013	04	S	03.00	0.438	VT	NVT	01	RC780	09/01/83	L-LC
RCH00000002233	0	BK2	MS	MS60013	04	S	03.00	0.438	VT	NVT	01	RC782	09/01/83	L-LC
RCH00000002234	0	BK2	MS	MS60013	04	S	03.00	0.438	VT	NVT	01	RC783	09/01/83	L-LC
RCH00000002235	0	BK2	MS	MS60013	04	S	03.00	0.438	VT	NVT	01	RC784	09/01/83	L-LC
RCH00000002236	0	BK2	MS	MS60013	04	S	02.00	0.443	VT	NVT	01	RC785	09/02/83	L-LC
RCH00000002237	0	BK2	MS	MS60013	04	S	02.00	0.443	VT	NVT	01	RC786	09/02/83	L-LC
RCH00000002238	0	BK2	MS	MS60013	04	S	02.00	0.443	VT	NVT	01	RC787	09/02/83	L-LC
RCH00000002239	0	BK2	MS	MS60013	04	S	02.00	0.443	VT	NVT	01	RC788	09/02/83	L-LC
RCH00000002240	1	BK2	RS	MS60013	04	S	02.00	0.443	VT	NVT	01	RC789	09/02/83	L-LC N018500067
RCH00000002241	0	BK2	MS	MS60013	04	S	02.00	0.443	VT	NVT	01	RC790	09/14/83	L-LC







12/15/85

TENNESSEE VALLEY AUTHORITY, DIVISION OF NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER 11 CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 CHATTANOOGA, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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Q Y :	SYSTEM	EXAM AREA	R E C O D E	T Y P E	S M H A T S I Z E	NDE METH	PRDC NO.	CAL BLOCK NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S		
01	RC	RCC0000001241	0	BJ	SW	ISI0013	09	S	02.00	0.343	PT	NPT 04	RD302	RM3 08/04/83 L-LC	A
01	RC	RCC0000001252	0	BJ	SW	ISI0013	09	S	02.00	0.343	PT	NPT 04	RD547	CGT 08/17/83 L-LC	A
01	RC	RCC0000001266	0	BJ	SW	ISI0013	09	S	02.00	0.343	PT	NPT 04	RD303	RM3 09/04/83 L-LC	A
01	RC	RCC0000001379A	0	BJ	SW	ISI0013	07	S	02.00	0.343	PT	NPT 04	RD319	CGT 03/05/83 L-LC	A
01	RC	RCC0000001382	0	BJ	SW	ISI0013	07	S	02.00	0.343	PT	NPT 04	RD320	CGT 03/05/83 L-LC	A
01	RC	RCC0000001385	0	BJ	SW	ISI0013	07	S	02.00	0.343	PT	NPT 04	RD321	CGT 03/05/83 L-LC	A
01	RC	RCC0000001397	0	BJ	SW	ISI0013	07	S	02.00	0.343	PT	NPT 04	RD322	GDF 03/05/83 L-LC	A
01	RC	RCC0000001407	0	BJ	SW	ISI0013	07	S	02.00	0.343	PT	NPT 04	RD323	GDF 08/05/83 L-LC	A
01	RC	RCC0000001444	0	BJ	SW	ISI0013	07	S	02.00	0.343	PT	NPT 04	RD324	GDF 09/05/83 L-LC	A
01	RC	RCC0000001473	0	BJ	SW	ISI0013	03	S	02.00	0.343	PT	NPT 04	RD304	RM3 08/04/83 L-LC	A
01	RC	RCC0000001433	0	BJ	SW	ISI0013	03	S	02.00	0.343	PT	NPT 04	RD305	RM3 03/04/83 L-LC	A
01	RC	RCC0000001505	0	BJ	SW	ISI0013	03	S	02.00	0.343	PT	NPT 04	RD306	RM3 03/04/83 L-LC	A
01	RC	RCC0000001502	0	BJ	SW	ISI0013	08	S	02.00	0.343	PT	NPT 04	RD307	RM3 03/04/83 L-LC	A
01	RC	RCC0000001729	0	BJ	SW	ISI0013	05	S	02.00	0.343	PT	NPT 04	RD311	TLS 08/04/83 L-LC	A
01	RC	RCC0000001733	0	BJ	SW	ISI0013	05	S	02.00	0.343	PT	NPT 04	RD312	TLS 08/04/83 L-LC	A
01	RC	RCC0000001743	0	BJ	SW	ISI0013	05	S	02.00	0.343	PT	NPT 04	RD313	TLS 08/04/83 L-LC	A
01	RC	RCC0000001745	0	BJ	SW	ISI0013	05	S	02.00	0.343	PT	NPT 04	RD314	TLS 08/04/83 L-LC	A
01	RC	RCC0000001757	0	BJ	SW	ISI0013	05	S	02.00	0.343	PT	NPT 04	RD315	TLS 08/04/83 L-LC	A
01	RC	RCC0000001751	0	BJ	SW	ISI0013	05	S	02.00	0.343	PT	NPT 04	RD510	CGT 03/15/83 L-LC	A
01	RC	63-0000000503	0	BG2	VB	ISI0013	05	S	03.00	0.438	VT	NVT 01	RD703	RM3 08/15/83 L-LC NDI#5Q0052	R
01	RC	68-0000000503	1	BG2	VB	ISI0013	05	S	03.00	0.438	VT	NVT 01	R1137	RM3 09/14/83 L-LC NDI#5Q0002	R
01	RC	68-0000000509	2	BG2	VB	ISI0013	05	S	03.00	0.438	VT	NVT 01	R1208	JDS 09/25/83 L-LC	A
01	RC	69-0000000514	0	BG2	VB	ISI0013	05	S	03.00	0.438	VT	NVT 01	RD519	RM3 03/15/83 L-LC	A
01	RC	68-0000000541	0	BG2	VB	ISI0013	04	S	03.00	0.438	VT	NVT 01	RD702	RM3 08/21/83 L-LC NDI#5Q0051	R

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SYSTEM	EXAM AREA	R E C D Y	I S H A	DRAWING #	I SITE	THICK	METH	NDE	PPCC	BLOCK	REPT	NO.	NO.	NO.	DATE	COMMENTS	R E S	
01 RC	68-000000541	1	B52	UB	ISI0013	04	S	05.00	0.439	VT	NUT	01	R1136		PM3 09/14/83	L-LC NDI#S00081 USE-85-I A S USQD#93-26	A	
01 RC	58-000000547	0	B52	UB	ISI0013	04	S	02.00	0.438	VT	NUT	01	R0570		PM3 09/21/83	L-LC	A	
01 RC	68-000000563	0	B52	UB	ISI0013	03	S	06.00	0.718	MT	NPT	03	R0715		JEF 09/23/83	L-LC B BOLTS IFR#82-02	A	
01 RC	68-000000563	0	B52	UB	ISI0013	03	S	06.00	0.718	VT	NUT	01	R0715		JEF 09/13/83	L-LC P BOLTS IFR#82-02	A	
01 RC	68-000000564	0	B42	VI	ISI0013	03	S	06.00	0.718	VT	NUT	01	R0717		LEH 09/23/83	L-LC VALVE INTERIOR	A	
01 RC	68-000000564	0	B52	UB	ISI0013	03	S	06.00	0.718	MT	NPT	03	R1082		MSE 05/07/83	L-LC IEB#82-02	R	
01 RC	68-000000564	0	B42	VI	ISI0013	03	S	06.00	0.718	VT	NUT	01	R1061		MSE 09/07/83	L-LC IEB#82-02 NOI#SQ0077	A	
01 RC	68-000000564	0	B42	VI	ISI0013	03	S	06.00	0.718	VT	NUT	01	R1072		PGF 09/07/83	L-LC VALVE INTERIOR	A	
01 RC	68-000000564	0	B42	VI	ISI0013	03	S	06.00	0.718	MT	NPT	03	R1063		ELW 09/09/83	L-LC IEB#82-02	A	
01 RC	68-000000565	0	B62	VB	ISI0013	03	S	06.00	0.718	MT	NPT	03	R0715		LDK 09/01/83	L-LC IEB#82-02	A	
01 RC	68-000000565	0	B62	VB	ISI0013	03	S	06.00	0.718	VT	NUT	01	R0716		LDK 09/01/83	L-LC IEB#82-02	A	
01 RC	68-000000565	0	B42	VI	ISI0013	03	S	06.00	0.718	VT	NUT	01	R1074		BGF 09/02/83	L-LC VALVE INTERIOR	A	
01 RCP	RCP#15LW#2155Y	0	N/A	5M	N/A	03	C	00.00	7.300	UT	0	NUT	21	R0457	CO016	F54 09/11/83	L-LC PER 261.14	A
01 RCP	RCP#15LW#2155Y	0	B61	VB	CHM2675	01	C	30.50	4.500	UT	0	NUT	05	R0710	CO040	L84 09/25/83	L-LC 41/4 FLANGE BOLTS 1 THROUGH 24	A
01 RCP	RCP#15LW#2155Y	0	B52	PB	CHM2675	01	C	06.00	2.000	VT	NUT	01	R0406		MSE 09/13/83	L-LC #1THROUGH#12 BOLTS	A	
01 RCP	RCP#15LW#2155Y	0	B52	PB	CHM2675	01	C	06.00	1.000	VT	NUT	01	R0407		MSE 03/10/83	L-LC NDI#S00032 #1THROUGH#12 BOLTS	R	
01 RCP	RCP#15LW#2155Y	1	B52	PB	CHM2675	01	C	00.00	1.000	VT	NUT	01	R0400		MSE 03/13/83	L-LC	A	
01 RCP	RCP#15LW#2155Y	0	B52	PB	CHM2675	01	C	00.00	0.575	VT	NUT	01	R0409		MSE 08/10/83	L-LC NDI#S00023 #1THROUGH #12 BOLTS	R	
01 RCP	RCP#15LW#2155Y	1	B52	PB	CHM2675	01	C	00.00	0.575	VT	NUT	01	R0410		MSE 08/13/83	L-LC	A	
01 RCP	RCP#15UP#0001	0	BK1	IA	ISI0093	01	S	00.00	0.000	PT	NPT	04	R1065		LDK 09/12/83	L-LC	A	
01 RCP	RCP#15UP#0001	0	BK2	CS	ISI0093	01	S	00.00	0.000	VT	NUT	01	R1002		LDK 09/03/83	L-LC	A	
01 RCP	RCP#15UP#0002	0	BK1	IA	ISI0093	01	S	00.00	0.000	PT	NPT	04	R1065		LDK 09/12/83	L-LC	A	
01 RCP	RCP#15UP#0002	0	BK2	CS	ISI0093	01	S	00.00	0.000	VT	NUT	01	R1002		LDK 09/05/83	L-LC	A	



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01	RCP	RCP#15UPO0003	0	BK1	IA	ISI0003	C1	S	00.00	0.000	PT	NPT	04				R1065					LDC 09/12/83	L-LC	A	
01	RCP	RCP#15UPO0003	0	BK2	CS	ISI0008	C1	S	00.00	0.000	VT	NVT	01				R1002					LDC 09/06/83	L-LC	A	
01	RCP	RCP#2FLY#4EEL	0	N/A	BM	N/A	03	C	00.00	7.000	UT	0	NUT	21	N/A		R1045	C0056				MD3 03/03/83	L-LC	PER RGL-14	A
01	RCP	RCP#2SL#1455Y	0	BG2	PB	CHM2675	C1	C	00.00	2.000	VT	NVT	01				R1237					LE4 03/11/83	L-LC	WITHROUGH#12	A
01	RCP	RCP#2SL#2455Y	0	BG2	PB	CHM2675	C1	C	00.00	1.000	VT	NVT	01				R1238					LE4 03/11/83	L-LC	WITHROUGH#12 BOLTS	A
01	RCP	RCP#2SL#3455Y	0	BG2	PB	CHM2675	C1	C	00.00	0.375	VT	NVT	01				R1239					LE4 03/11/83	L-LC	WITHROUGH#12	A
01	RCP	RCP#2SL#4455Y	0	BG2	PB	CHM2675	C1	C	00.00	0.375	VT	NVT	01				R0316					LE4 09/04/83	L-LC	WITHROUGH#12 NOT#SQ A 0027 USF-45-IS USCC#87-25	A
01	RCP	RCP#2SL#2455Y	0	BG2	PB	CHM2675	C1	C	00.00	1.000	VT	NVT	01				R0317					LE4 03/04/83	L-LC	WITHROUGH#12 BOLTS#N 02#SQ0023	R
01	RCP	RCP#2SL#2455Y	1	BG2	PB	CHM2675	C1	C	00.00	1.000	VT	NVT	01				R0329					LE4 08/05/83	L-LC		A
01	RCP	RCP#2SL#2455Y	0	BG2	PB	CHM2675	C1	C	00.00	0.375	VT	NVT	01				R0318					LE4 03/04/83	L-LC	MOI#SQ0029 WITHROUGH #12 BOLTS	R
01	RCP	RCP#2SL#2455Y	1	BG2	PR	CHM2675	C1	C	00.00	0.375	VT	NVT	01				R0328					LE4 06/05/83	L-LC		A
01	RCP	RCP#4FLY#4EEL	0	N/A	3M	N/A	03	C	00.00	7.000	UT	0	NUT	21	OC000	R0458	C0016					FSW 03/11/83	L-LC	PER RGL-14	A
01	RCP	RCP#4SL#1455Y	0	BG2	PB	CHM2675	C1	C	00.00	2.000	VT	NVT	01				R0177					RM3 07/27/83	L-LC	WITHROUGH#12 BOLTS	A
01	RCP	RCP#4SL#2455Y	0	BG2	PB	CHM2675	C1	C	00.00	1.000	VT	NVT	01				R0175					RM3 07/27/83	L-LC	WITHROUGH#12 BOLTS	A
01	RCP	RCP#4SL#3455Y	0	BG2	PB	CHM2675	C1	C	00.00	0.375	VT	NVT	01				R0175					RM3 07/27/83	L-LC	WITHROUGH#12 BOLTS	A
01	RHR	RHR#0000000000A	0	CF	CH	ISI0003	C1	S	14.00	0.439	PT	NPT	04				R0021					LE4 07/23/83	L-LC	WITHROUGH#12 BOLTS	A
01	RHR	RHR#00000010A	0	CF	CH	ISI0003	C1	S	18.00	0.552	PT	NPT	04				R1135					HJD 03/14/83	L-LC		A
01	RHR	RHR#0000000014	0	CF	CH	ISI0003	C1	S	14.00	0.439	PT	NPT	04				R1138	C0058				LB4 09/14/83	L-LC		A
01	RHR	RHR#00000000125	0	BK2	RS	M560010	C1	S	06.00	0.718	PT	NPT	04				R0365					GD3 03/03/83	L-LC		A
01	RHR	RHR#0000000002	0	BK2	RS	M560010	C1	S	14.00	1.406	VT	NVT	01				R0355	C0025				LB4 03/17/83	L-LC	LMT-SCAN 304; REF. P T EXAM R0355	A
01	RHR	RHR#0000000002	0	BK2	RS	M560010	C1	S	14.00	1.406	VT	NVT	01				R0352					JEF 03/02/83	L-44R		A
01	RHR	RHR#0000000002	0	BK2	RS	M560010	C1	S	14.00	1.406	VT	NVT	01				R1068					RM3 09/12/83	L-LC		A

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01	RHR	RHRH000003403	0	BK1	IA	MSG010	01	S	14.00	1.406	PT	NUT	04			R0426				CGT 03/11/83	L-LC	A	
01	RHR	RHRH000003403	0	BK2	RS	MSE010	01	S	14.00	1.406	VT	NUT	01			R1059				RMA 09/12/83	L-LC	A	
01	RHR	RHRH000003404	0	BK2	RS	MSE010	01	S	14.00	1.406	VT	NUT	01			R1070				RMA 03/12/83	L-LC	A	
01	RHR	RHRH000003405	0	BK2	RS	MSE010	01	S	14.00	1.406	VT	NUT	01			R1071				RMA 03/12/83	L-LC	A	
01	RHR	RHRH000003405	0	BK2	RS	MSE010	01	S	14.00	1.406	VT	NUT	01			R1072				RMA 03/12/83	L-LC	A	
01	RHR	RHRH000003407	0	BK1	IA	MSE010	01	S	14.00	1.406	PT	NPT	04			00427				CGT 03/11/83	L-LC	A	
01	RHR	RHRH000003407	0	BK2	RS	MSE010	01	S	14.00	1.406	VT	NUT	01			R1107				RMA 09/13/83	L-LC	NOT#SQ0078	R
01	RHR	RHRH000003407	1	BK2	RS	MSE010	01	S	14.00	1.406	VT	NUT	01			R1235				LE4 09/23/83	L-LC	A	
01	RHR	RHRH000003400	0	CE	US	MSE001	03	S	14.00	0.438	VT	NUT	01			R0001				LE4 07/22/83	L-RPR2A	A	
01	RHR	RHRH000003401	0	CE	RS	MSE010	03	S	14.00	0.438	VT	NUT	01			R0002				LE4 07/22/83	L-RPR2A	A	
01	RHR	RHRH000003402	0	CE	RS	MSE010	03	S	14.00	0.438	VT	NUT	01			R0054				JEF 07/25/83	L-RPR2A	A	
01	RHR	RHRH000003403	0	CE	RS	MSE010	03	S	14.00	0.438	VT	NUT	01			R0155				JEF 07/25/83	L-RPR2A	A	
01	RHR	RHRH000003404	0	CE	RS	MSE010	03	S	14.00	0.438	VT	NUT	01			R031E				LE4 07/22/83	L-PC553	NOT#SQ0024	R
01	RHR	RHRH000003404	1	CE	RS	MSE010	03	S	14.00	0.438	VT	NUT	01			R1209				LE4 09/25/83	L-PC553	A	
01	RHR	RHRH000003405	0	CE	RS	MSE010	03	S	14.00	0.438	VT	NUT	01			R0703				LE4 07/22/83	L-PC553	A	
01	RHR	RHRH000003407	0	CE	US	MSE010	03	S	14.00	0.438	VT	NUT	01			R0004				LE4 07/22/83	L-RPR2B	A	
01	RHR	RHRH000003408	0	CE	RS	MSE010	03	S	14.00	0.438	VT	NUT	01			R0205				LE4 07/22/83	L-RPR2B	NOT#SQ0001	R
01	RHR	RHRH000003409	1	CE	RS	MSE010	03	S	14.00	0.438	VT	NUT	01			R030R				JCF 03/25/83	L-RPR2B	A	
01	RHR	RHRH000003409	0	CE	RS	MSE010	03	S	14.00	0.438	VT	NUT	01			R0052				JEF 07/25/83	L-RPR2B	A	
01	RHR	RHRH000003410	0	CE	RS	MSE010	03	S	14.00	0.438	VT	NUT	01			R0351				JEF 07/25/83	L-RPR2B	A	
01	RHR	RHRH000003411	0	CE	RS	MSE010	03	S	14.00	0.562	VT	NUT	01			R0253				LE4 03/02/83	L-PC553	A	
01	RHR	RHRH000003412	0	CE	RS	MSE010	03	S	14.00	0.562	VT	NUT	01			R0005				LE4 07/22/83	L-PC553	A	
01	RHR	RHRH000003414	0	CE	US	MSE010	02	S	14.00	0.438	VT	NUT	01			R0435				JEF 03/13/83	L-PC553	A	
01	RHR	RHRH000003415	0	CE	RS	MSE010	02	S	14.00	0.438	VT	NUT	01			R056R				JEF 09/13/83	L-PC553	A	



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01	RHR	RHR4000003446	0	CE	RS	MSG0010	05	S	08.00	0.322	VT	NUT	01	P0587				JEF 03/09/83	L-RHR23	A	
01	RHR	RHR4000003447	0	CE	RS	MSG0010	05	S	08.00	0.322	VT	NUT	01	R0575				JEF 08/19/83	L-RHR23	NOI#SQ0047	R
01	RHR	RHR4000003447	1	CE	RS	MSG0010	05	S	08.00	0.322	VT	NUT	01	R1145				MSE 07/15/83	L-RHR28		A
01	RHR	RHR4000003448	0	CE	MS	MSG0010	05	S	08.00	0.322	VT	NUT	01	R0576				JEF 08/19/83	L-RHR28		A
01	RHR	RHR4000003449	0	CE	RS	MSG0010	05	S	08.00	0.322	VT	NUT	01	R0577				JEF 08/19/83	L-RHR29		A
01	RHR	RHR4000003450	0	CE	RS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0700				RM9 07/23/83	L-RHR24		A
01	RHR	RHR4000003451	0	CE	RS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0701				RM9 07/24/83	L-RHR24	NOI#SQ0003	R
01	RHR	RHR4000003451	1	CE	RS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0745				JEF 07/27/83	L-RHR24		A
01	RHR	RHR4000003452	0	CE	MS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0702				RM9 07/24/83	L-RHR24	NOI#SQ0004	R
01	RHR	RHR4000003453	1	CE	RS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0702				RM9 07/23/83	L-RHR24		A
01	RHR	RHR4000003454	0	CE	RS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0701				RM9 07/23/83	L-RHR24		A
01	RHR	RHR4000003455	0	CE	RS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0702				RM9 07/23/83	L-RHR24		A
01	RHR	RHR4000003456	0	CE	RS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0703				RM9 07/23/83	L-RHR24		A
01	RHR	RHR4000003457	0	CE	MS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0516				JEF 07/15/83	L-PC590		A
01	RHR	RHR4000003458	0	CE	MS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0505				JEF 08/20/83	L-PC553		A
01	RHR	RHR4000003459	0	CE	RS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0594				JEF 03/17/83	L-RP24		A
01	RHR	RHR4000003460	0	CE	RS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0457				JEF 09/11/83	L-RP24		A
01	RHR	RHR4000003461	0	CE	MS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0328				LE4 07/24/83	L-RHR28		A
01	RHR	RHR4000003462	0	CE	RS	MSG0010	04	S	08.00	0.322	VT	NUT	01	R0404				JEF 08/19/83	L-RHR28		A
01	RHR	RHR4000003463	0	CE	MS	MSG0010	05	S	08.00	0.322	VT	NUT	01	R0727				LEW 07/24/83	L-RHR28		A
01	RHR	RHR4000003464	0	CE	RS	MSG0010	05	S	08.00	0.322	VT	NUT	01	R0705				RM3 07/29/83	L-RHR28		A
01	RHR	RHR4000003465	0	CE	RS	MSG0010	05	S	08.00	0.322	VT	NUT	01	R0725				LE4 07/24/83	L-RHR29		A
01	RHR	RHR4000003465	0	CE	RS	MSG0010	05	S	08.00	0.322	VT	NUT	01	R0206				RM9 07/29/83	L-RHR29		A

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01 RHR	RHR4000000457	D	CE	YS	MSG0010	05	S	06.00	0.322	VT	NUT 01	R0492		JEF 08/15/83	L-PC590	A	
01 RHR	RHR4000000469	D	CE	YS	MSG0010	05	S	08.00	0.322	VT	NUT 01	RC585		JEF 03/19/83	L-RPR28	NOIHS00048	R
01 RHR	RHR4000000457	I	CE	YS	MSG0010	05	S	08.00	0.322	VT	NUT 01	P1175		LDC 07/17/83	L-RP22A		A
01 RHR	RHR4000000470	D	CE	YS	MSG0010	05	S	08.00	0.322	VT	NUT 01	FC399		JEF 08/07/83	L-RPR28		A
01 RHR	RHR4000000471	D	CE	YS	MSG0010	04	S	09.00	0.322	VT	NUT 01	R0596		JEF 03/19/83	L-RP22A		A
01 RHR	RHR4000000472	D	CE	YS	MSG0010	05	S	06.00	0.322	VT	NUT 01	RC587		JEF 08/17/83	L-RPR28		A
01 RHR	RHR50000002011	D	CF	CH	ISI0003	01	S	14.00	0.438	PT	NPT 04	R0769		JEF 07/25/83	L-RPR2A		A
01 RHR	RHR50000002015	D	CF	CH	ISI0003	01	S	14.00	0.438	PT	NPT 04	R0759		JEF 07/25/83	L-RPR2A		A
01 RHR	RHR50000002027	D	CF	CH	ISI0003	01	S	14.00	0.438	PT	NPT 04	R0349		JEF 07/25/83	L-RPR28		A
01 RHR	RHR50000002031	D	CF	CH	ISI0003	01	S	14.00	0.438	PT	NPT 04	RC350		JEF 07/25/83	L-RPR2A		A
01 RHR	RHR50000002064	D	CF	CH	ISI0003	01	S	14.00	0.438	PT	NPT 04	R0048		JEF 07/25/83	L-RPR28		A
01 RHR	RHR50000002065	D	BJ	CH	ISI0003	07	S	14.00	0.438	PT	NPT 04	R0767		JEF 07/25/83	L-RPR2A		A
01 RHR	RHR50000002091	D	BJ	CH	ISI0003	05	S	06.00	0.716	PT	NPT 04	R0583		GDF 08/15/83	L-LC		A
01 RHR	RHR50000002095	D	BJ	CH	ISI0003	08	S	03.00	0.343	PT	NPT 04	R0724		FS4 08/27/83	L-LC		A
01 RHR	RHR5000000215	D	CF	LW	ISI0003	01	S	18.00	0.562	PT	NPT 04	R0251		MSE 09/23/83	L-1A9		A
01 RHR	RHR5000000215	D	CF	LW	ISI0003	01	S	14.00	0.438	PT	NPT 04	RC250		LEW 03/02/83	L-PC555		A
01 RHR	RHR50000002215	D	CF	LW	ISI0003	01	S	18.00	0.562	PT	NPT 04	R0341		LEW 09/02/83	L-PC553		A
01 RHR	RHR50000002215	D	CF	LW	ISI0003	01	S	18.00	0.562	PT	NPT 04	R0341		JCS 03/05/83	L-PC553		A
01 RHR	RHR50000002115	D	BJ	SW	ISI0003	03	S	02.00	0.343	PT	NPT 04	R0247		JEF 07/25/83	L-RPR28		A
01 RHR	SIR00000002320	D	CE	RS	MSG0010	05	S	12.00	0.406	VT	NUT 01	R0256		MSE 09/25/83	L-1A9		A
01 RHR	SIR00000002321	D	CE	RS	MSG0010	07	S	08.00	0.412	VT	NUT 01	R0237		JEF 07/31/83	L-4A9		A
01 RHR	SIR00000002322	D	CE	RS	MSG0010	06	S	09.00	0.312	VT	NUT 01	R0559		MSE 09/13/83	L-4A9		A



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01 RHR	SIF0000000003	0	BK2 RS	MS60010	05 S	09.00	0.312	VT	NVT 01		R0765		RMB	09/27/83	L-LC	A	
01 RHR	SIH0000000009	0	BK2 RS	MS60010	05 S	06.00	0.718	VT	NVT 01		R0765		RMB	03/27/83	L-LC	A	
01 RHR	74-0000000001	0	B62 VB	ISI0003	07 S	14.00	1.406	MT	NVT 01		R0585		LEW	03/20/83	L-LC IER#82-02	A	
								VT	NVT 01		R0583		LEW	03/27/83	L-LC IER#82-02 NOI#SQ0049	R	
01 RHR	74-0000000001	1	B62 VB	ISI0003	07 S	14.00	1.406	VT	NVT 01		R0596		LEW	08/27/83	L-LC IER#82-02	A	
01 RHR	74-0000000003	0	B62 VB	ISI0003	07 S	14.00	1.406	VT	NVT 01		R1108		RMB	09/13/83	L-LC NOI#SQ0079	R	
01 RHR	74-0000000002	1	B62 VB	ISI0003	07 S	14.00	1.406	VT	NVT 01		R1715		LRM	09/27/83	L-LC	A	
01 RHRHX	RHR4000000014A	0	CB	NW	CHM2404	01 S	00.00	1.000	PT	NPT 04		R0743		JEF	07/27/83	L-RHX2A	A
									UT 0	NUT 23	SC 15	R0783	C0052	LRM	08/23/83	L-RHX2A	A
									UT 45	NUT 23	SC 15	R0786	C0055	LRM	08/23/83	L-RHX2A LMT-SCAN 3,5,6	A
01 RHRHX	RHR4000000015A	0	CA	SH	CHM2404	01 S	00.00	0.000	UT 0	NUT 23	SC 15	R0784	C0052	LRM	08/23/83	L-RHX2A	A
									UT 45	NUT 23	SC 15	R0787	C0055	LRM	08/23/83	L-RHX2A LMT- NO SCAN 4	A
01 RHRHX	RHR4000000017A	0	CA	NW	CHM2404	01 S	00.00	1.000	UT 0	NUT 23	SC 15	R0785	C0052	LRM	03/23/83	L-RHX2A	A
									UT 45	NUT 23	SC 15	R0788	C0055	LRM	09/23/83	L-RHX2A LMT-SCAN 4	A
01 RHRHX	RHR4000000018A	0	CE	CS	CHM2404	01 S	00.00	1.000	VT	NVT 01		R0204		RMB	07/27/83	L-RHX2A	A
01 RHRP	RHRPH-00002A4	0	CE	CS	ISI0099	01	00.00	0.000	VT	NVT 01		R0792		RMB	05/05/83	L-RP2A NOI#SQ0023	R
01 RHRP	RHRPH-00002A4	1	CE	CS	ISI0099	01	00.00	0.000	VT	NVT 01		R1714		MSE	09/27/83	L-RP2A	A
01 RHRP	RHRPH-00002B9	0	CE	CS	ISI0099	01	00.00	0.000	VT	NVT 01		R1179		MSE	07/27/83	L-RP2A #135	A
01 RHX	RHX4000000001	0	CE	CS	ISI0065	03 S	00.00	0.000	VT	NVT 01		R0124		LDC	08/21/83	L-RBAR NOI#SQ0050	R
01 RHX	RHX4000000001	1	CE	CS	ISI0065	03 S	00.00	0.000	VT	NVT 01		R1313		LRM	07/25/83	L-RBAR	A
01 RHX	RHX4000000001A	0	CC	IA	ISI0065	03 S	00.00	0.000	PT	NPT 04		R0459		CGT	03/12/83	L-RBAR .375 THK LUGS(3)	A
01 RHX	RHX4000000002	0	CE	CS	ISI0066	03 S	00.00	0.000	VT	NVT 01		R0722		LDC	08/29/83	L-RBAR NOI#SQ0054	R
01 RHX	RHX4000000002	1	CE	CS	ISI0065	03 S	00.00	0.000	VT	NVT 01		R1712		LRM	09/25/83	L-RBAR	A
01 RHX	RHX3000000001	0	CA	NW	ISI0065	01 S	09.63	0.739	PT	NPT 04		R0460		TLS	08/12/83	L-RBAR	A
01 RHX	RHX0000000002	0	CA	FW	ISI0065	01 S	09.67	0.738	PT	NPT 04		R1179		WJD	09/15/83	L-RBAR	A
01 RHX	RHX0000000003	0	CA	TW	ISI0065	01 S	09.26	0.756	PT	NPT 04		R1140		TLS	09/15/83	LMT-BEST EFFORT PT SEE RE A QJEST FOR RELIEF	A

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TENNESSEE VALLEY AUTHORITY, DIVISION OF NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 DAVIS, TENNESSEE  
 UNIT 3 COMMERCIAL SERVICE DATE 06/01/82

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SYSTH	EXAM AREA	R S CAT	T Y P	S M H A	W T	SIZE	THICK	METH	NDE	PROC NO.	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S
01 RHX	RHX0000000004	0 CA	MW	ISI0056	01 S	09.26	0.756	PT	NPT	04		R0451		WJD	02/12/83	L-RBAR	A
01 RV	CHNUT#0000001	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	03/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000003	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0499		JCG	08/15/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000004	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0499		JCG	08/15/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000006	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0499		JCG	08/15/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000007	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0499		JCG	08/15/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000008	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0499		JCG	08/15/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000009	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0499		JCG	08/15/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000015	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	08/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000018	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	08/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000020	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	08/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000021	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	03/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000022	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0597		JCG	08/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000023	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	03/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000024	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	03/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000025	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	08/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000026	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	03/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000028	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	08/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000027	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	03/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000030	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	08/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000031	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	03/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000032	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0597		JCG	08/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000033	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	08/17/83	L-UC TVA IMPOSED INSP	A
01 RV	CHNUT#0000035	0 B61	CB	CHM2341	01 C	00.00	7.000	MT	NMT	03		R0550		JCG	08/17/83	L-UC TVA IMPOSED INSP	A

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TENNESSEE VALLEY AUTHORITY, DIVISION OF NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 DAVIS, TENNESSEE  
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Y	SYSTEM	EXAM AREA	R E C O D	T Y	S M H A	D R A W I N G	T S I Z E	T H I C K	N O E M E T H	P R O C N O.	C A L B L O C K N O.	R E P T N O.	C A L R E P T N O.	I N P	D A T E	C O M M E N T S	R E S
01	RV	CHNUT#0000035	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO550	JCG 08/17/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000037	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO499	JCG 08/15/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000038	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO587	JCG 03/19/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000039	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO499	JCG 08/15/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000040	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO587	JCG 08/19/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000044	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO587	JCG 08/19/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000045	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO597	JCG 08/19/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000046	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO597	JCG 03/19/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000047	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO587	JCG 03/19/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000051	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO587	JCG 03/19/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000052	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO597	JCG 08/17/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000055	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO587	JCG 08/17/83	L-UC TVA IMPOSED INSP	A
01	RV	CHNUT#0000060	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO550	JCG 08/17/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000001	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO594	LDC 03/23/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000004	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO596	LDC 03/24/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000005	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO594	LDC 03/23/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000006	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO594	LDC 03/23/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000009	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO596	LDC 03/24/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000010	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO574	LDC 03/23/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000012	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO574	LDC 08/23/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000013	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO594	LDC 03/23/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000014	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO594	LDC 03/23/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000016	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO574	LDC 03/23/83	L-UC TVA IMPOSED INSP	A
01	RV	CHSTUD#0000017	0	B61	CB	CHM2341	01	C	00.00	7.000	MT	NMT	03	RO596	LDC 08/24/83	L-UC TVA IMPOSED INSP	A

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TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SECOYAH NUCLEAR PLANT  
 P. O. BOX 2000 DRTSY, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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Y	SYSTM	EXAM AREA	R S C D Y	T C A T	S H	M A	N D E	P R O C	C A L B L O C K	R E P T	C A L R E P T	I N	D A T E	C O M M E N T S	R E S	
01	SG	H#41WCBLT3	1	B32	MB	MSG0002	01	C	00.00	1.990	VT	NUT	01	R1057	MSE 09/09/83 L-LC NOT#SQ0075 16BOLTS IEB#02-02 BOLTING RMVD.	R
01	SG	H#41WCBLT3	2	B32	MB	MSG0002	01	C	00.00	1.980	VT	NUT	01	R1059	MSE 09/09/83 L-LC 16BOLTS IEB#02-02 4 A REPLACEMENT BOLTS	A
01	SG	SGH00000000A1	3	B3	TW	CHM2345	01	C	00.00	5.700	UT	0	NUT 19 SQ 49 R0309 C0059 FSW 04/23/83 L-LC	A	A	
														UT45 NUT 19 SQ 49 R0324 C0056 JCG 03/23/83 L-LC LMT-SCAN 3 E 4	A	
														UT50 NUT 19 SQ 49 R0325 C0057 JCG 03/30/83 L-LC LMT-SCAN 3 E 4	A	
01	SG	SSW00000000B1	0	CA	TW	CHM2345	01	C	00.00	0.000	UT	0	NUT 19 SQ 40 R0789 C0053 FSW 08/23/83 L-LC 12 FT STARTING @ 0;	A	A	
														LMT-SCAN 2	A	
														L-LC 12 FT STARTING @ 0;	A	
														LMT-SCAN 3	A	
														L-LC 12 FT STARTING @ 0;	A	
														LMT-SCAN 3	A	
01	SG	SGH00000000E1	0	CA	SH	CHM2345	01	C	00.00	3.580	UT	0	NUT 19 SQ 40 R0779 C0049 FSW 08/25/83 L-LC	A	A	
														UT45 NUT 19 SQ 40 R0712 C0038 FSW 03/23/83 L-LC LMT-SCAN 4	A	
														UT50 NUT 19 SQ 40 R0714 C0039 FSW 03/24/83 L-LC LMT-SCAN 4	A	
01	SG	SGH00000000F1	0	CA	HW	CHM2345	01	C	00.00	4.000	UT	0	NUT 19 WB 39 R0770 C0048 FSW 03/25/83 L-LC 15FT START @ 0 DEG.	A	A	
														CH LMT- SCAN 2	A	
														L-LC 15 FT. START @ 0 DEG	A	
														CH LMT- SCAN 4	A	
														L-LC 15 FT. START @ 0 DEG	A	
														CH LMT- SCAN 4	A	
01	SI	RHRH00000000B	0	BK2	RS	MSG0007	12	S	02.00	0.343	VT	NUT	01	R0373	RM4 09/31/83 L-LC	A
01	SI	RHRH000000007	0	BK2	RS	MSG0009	12	S	02.00	0.343	VT	NUT	01	R0374	RM9 03/31/83 L-LC	A
01	SI	RHRH000000010	0	BK2	RS	MSG0009	12	S	02.00	0.343	VT	NUT	01	R0375	RM9 03/31/83 L-LC	A
01	SI	RHR4000000011	0	BK2	RS	MSG0007	12	S	02.00	0.343	VT	NUT	01	R0376	RM3 03/31/83 L-LC	A
01	SI	SIF0000000121B	0	BJ	DW	ISI0002	04	S	06.00	0.718	PT	NPT	04	R0748	JEF 09/02/83 L-4AR	A
														UT45 SOUT27 SQ 01 R0510 C0001 FSW 03/04/83 L-4AR	A	
01	SI	SIF0000000129	0	BJ	DW	ISI0002	04	S	10.00	1.000	PT	NPT	04	R0716	RM3 03/26/83 L-LC	A
														UT45 SOUT27 SQ 39 R1046 C0057 MDR 09/03/83 L-LC LMT-BEST EFFORT 505	A	
														SCANS ONLY R0716 FOR PT	A	
01	SI	SIF0000000182	0	BJ	DW	ISI0002	05	S	06.00	0.718	PT	NPT	04	R0717	MSE 08/25/83 L-2AR	A
														UT45 SOUT27 SQ 01 R0774 C0046 LBN 03/27/83 L-2AR	A	
01	SI	SIF0000000179	0	BJ	DW	ISI0002	05	S	06.00	0.718	PT	NPT	04	R0590	CGT 09/13/83 L-LC	A

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TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 DAISY, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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SYSTM	EXAM AREA	R E C O D Y C A T P	T D R A W I N G S	S H A T S I Z E	M A T E R I A L	NDC METH	PRDC NO.	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S
01 SI	SIF0000000179	J BJ	CH ISI0002	05 S	06.00	0.718	UT45	SQUT27	SQ 01	R0911	C0061	FSW 03/31/83	L-LC	A
01 SI	SIF0000000201	J BJ	CH ISI0002	07 S	03.00	0.438	PT	NPT 04		R0245		JEF 08/02/83	L-4AR	A
01 SI	SIH0000000001	0 BK2	RS MSG0007	03 S	10.00	1.000	VT	NVT 01		R0180		JEF 07/23/83	L-1AR	A
01 SI	SIH0000000002	J BK1	IA MSG0007	03 S	10.00	1.000	PT	NPT 04		R1109		BGR 09/22/83	L-LC	A
01 SI	SIH0000000002	J BK2	RS MSG0007	03 S	10.00	1.000	VT	NVT 01		R0746		LEW 08/29/83	L-LC	A
01 SI	SIH0000000003	J BK2	RS MSG0007	03 S	10.00	1.000	VT	NVT 01		R1064		LEW 08/23/83	L-LC	A
01 SI	SIH0000000004	J BK2	RS MSG0007	03 S	10.00	1.000	VT	NVT 01		R0747		LEW 09/23/83	L-LC	A
01 SI	SIH0000000005	J BK2	RS MSG0007	03 S	10.00	1.000	VT	NVT 01		R0748		LEW 03/23/83	L-LC	A
01 SI	SIH0000000006	J BK2	RS MSG0007	03 S	10.00	1.000	VT	NVT 01		R0749		LEW 03/23/83	L-LC	A
01 SI	SIH0000000007	J BK2	RS MSG0007	03 S	06.00	0.718	VT	NVT 01		R1101		RMB 09/13/83	L-LC	A
01 SI	SIH0000000003	J BK2	RS MSG0007	03 S	06.00	0.718	VT	NVT 01		R0191		JEF 07/23/83	L-1AR	A
01 SI	SIH0000000007	0 BK2	RS MSG0007	03 S	06.00	0.718	VT	NVT 01		R0192		JEF 07/23/83	L-1AR	A
01 SI	SIH0000000010	J CE	RS MSG0009	03 S	06.00	0.718	VT	NVT 01		R1174		BGR 07/17/83	L-4FR	A
01 SI	SIH0000000011	0 CE	RS MSG0009	03 S	06.00	0.713	VT	NVT 01		R0791		LEW 03/23/83	L-4FR NOI#SQ0053	R
01 SI	SIH0000000011	1 CE	RS MSG0009	03 S	06.00	0.713	VT	NVT 01		R1207		BGR 07/17/83	L-4FR NOI#SQ0087 USE-AS- IS USQ0183-28	A
01 SI	SIH0000000012	J CE	RS MSG0009	03 S	06.00	0.718	VT	NVT 01		R0792		LEW 03/23/83	L-4FR	A
01 SI	SIH0000000013	J CE	RS MSG0009	03 S	06.00	0.718	VT	NVT 01		R0793		LEW 03/23/83	L-4FR	A
01 SI	SIH0000000014	J CE	RS MSG0009	03 S	06.00	0.718	VT	NVT 01		R0794		LEW 03/23/83	L-4FR	A
01 SI	SIH0000000015	0 CE	RS MSG0009	03 S	06.00	0.718	VT	NVT 01		R0795		LEW 08/23/83	L-4FR	A
01 SI	SIH0000000016	0 CE	RS MSG0009	03 S	06.00	0.718	VT	NVT 01		R0796		LEW 03/23/83	L-4FR	A
01 SI	SIH0000000017	0 CE	RS MSG0009	03 S	06.00	0.712	VT	NVT 01		R0322		JEF 07/31/83	L-4AR	A
01 SI	SIH0000000019	J CC	IA MSG0007	02 S	08.00	0.812	PT	NPT 04		R0722		MSE 03/23/83	L-4AR	A
01 SI	SIH0000000019	J CE	RS MSG0009	02 S	08.00	0.812	VT	NVT 01		R0323		JEF 07/31/83	L-4AR	A
01 SI	SIH0000000019	0 CE	RS MSG0009	02 S	08.00	0.812	VT	NVT 01		R0324		JEF 07/31/83	L-4AR	A

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TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
1750 CHESTNUT STREET TOWER II, CHATTANOOGA, TENNESSEE 37402  
SEQUOYAH NUCLEAR PLANT  
P. O. BOX 2000  
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SYST#	EXAM AREA	R	E	C	Y	F	S	M	A	NCE	TRICK	METH	NC.	PROC	NO.	REPT	NO.	CAL	I	DATE	COMMENTS	
		#	CAT	P	CPA	R	Y	SIZE	VT	VT	VT	VT	VT	VT	VT	VT	VT	VT	VT	VT	VT	VT
01 SI	SIH0000000020	0	BK2	RS	MSG000	02	S	06.00	0.718	VT	NVT	01	NVT	01	R1102	RM3	09/13/83	L-LC			A	
01 SI	SIH0000000021	0	BK2	RS	MSG000	02	S	06.00	0.718	VT	NVT	01	NVT	01	R1103	RM3	09/13/83	L-LC			A	
01 SI	SIH0000000022	0	BK1	IA	MSE000	02	S	10.00	1.000	PT	VPT	04	VPT	01	R0171	MSE	09/30/83	L-LC			A	
01 SI	SIH0000000022	0	BK2	RS	MSG000	02	S	10.00	1.000	VT	NVT	01	NVT	01	R0750	LEH	09/23/83	L-LC			A	
01 SI	SIH0000000023	0	BK2	RS	MSG000	02	S	10.00	1.000	VT	NVT	01	NVT	01	R0751	LEH	03/23/83	L-LC			A	
01 SI	SIH0000000024	0	BK1	IA	MSE000	02	S	10.00	1.000	PT	NPT	04	NPT	01	R0972	MSE	03/30/83	L-LC			A	
01 SI	SIH0000000024	0	BK2	RS	MSG000	02	S	10.00	1.000	VT	NVT	01	NVT	01	R0752	LEH	03/28/83	L-LC			A	
01 SI	SIH0000000025	0	BK2	RS	MSG000	02	S	10.00	1.000	VT	NVT	01	NVT	01	R0225	JEF	07/31/83	L-4AR			A	
01 SI	SIH0000000026	0	BK2	RS	MSG000	02	S	10.00	1.000	VT	NVT	01	NVT	01	R0226	JEF	07/31/83	L-4AR			A	
01 SI	SIH0000000027	0	BK2	RS	MSG000	02	S	10.00	1.000	VT	NVT	01	NVT	01	R0227	JEF	07/31/83	L-4AR			A	
01 SI	SIH0000000031	0	BK2	US	MSG000	02	S	07.00	0.343	VT	NVT	01	NVT	01	R0104	JEF	07/29/83	L-1AR	WT#S00017		R	
01 SI	SIH0000000031	1	BK2	US	MSG000	03	S	07.00	0.343	VT	NVT	01	NVT	01	R1047	RM3	09/01/83	L-1AR			A	
01 SI	SIH0000000032	0	BK2	RS	MSG000	02	S	02.00	0.343	VT	NVT	01	NVT	01	R0103	JEF	07/23/83	L-1AR			A	
01 SI	SIH0000000033	0	BK2	US	MSG000	02	S	02.00	0.343	VT	NVT	01	NVT	01	R1147	MSE	03/15/83	L-3AR			A	
01 SI	SIH0000000033	0	BK	MS	MSG000	02	S	02.00	0.343	VT	NVT	01	NVT	01	R1148	MSE	09/15/83	L-3AR			A	
01 SI	SIH0000000041	0	CE	RS	MSG000	05	S	06.00	0.718	VT	NVT	01	NVT	01	R0550	MSE	09/13/83	L-LC			A	
01 SI	SIH0000000042	0	CE	RS	MSG000	05	S	06.00	0.718	VT	NVT	01	NVT	01	R0794	MSE	08/04/83	L-2FR			A	
01 SI	SIH0000000043	0	CE	RS	MSG000	05	S	06.00	0.718	VT	NVT	01	NVT	01	R0795	MSE	08/04/83	L-2FR			A	
01 SI	SIH0000000044	0	CE	RS	MSG000	05	S	06.00	0.718	VT	NVT	01	NVT	01	R0572	MSE	08/22/83	L-2FR			A	
01 SI	SIH0000000045	0	CE	RS	MSG000	05	S	08.00	0.912	VT	NVT	01	NVT	01	R0361	MSE	08/13/83	L-RBAR			A	
01 SI	SIH0000000047	0	CE	RS	MSG000	05	S	08.00	0.912	VT	NVT	01	NVT	01	R0562	MSE	09/13/83	L-RBAR			A	
01 SI	SIH0000000047	0	CE	RS	MSG000	05	S	08.00	0.912	VT	NVT	01	NVT	01	R0563	MSE	09/13/83	L-RBAR			A	
01 SI	SIH0000000050	0	CE	RS	MSG000	05	S	08.00	0.912	VT	NVT	01	NVT	01	R0554	MSE	03/19/83	L-RBAR			A	
01 SI	SIH0000000050	0	CE	RS	MSG000	05	S	08.00	0.912	VT	NVT	01	NVT	01	R0555	MSE	09/19/83	L-RBAR			A	

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TENNESSEE VALLEY AUTHORITY, DIVISION, NUCLEAR POWER  
1750 CHESTNUT STREET TOWER II, CHATTANOOGA, TENNESSEE 37402  
SEQUOYAH NUCLEAR PLANT  
P. O. BOX 2009  
01151, TENNESSEE  
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C	Y	EXAM	R	T	S	H	M	PRG	THCK	METH	NO.	PLDCK	REPT	NO.	DATE	COMMENTS	R
Y	EXAM	AREA	ECOD	Y	H	A	P	NOE	SIZE			NO.	NO.				E
C	SYSTEM		Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	Y	S
01	SI	SIH0000000051	0	CE	RS	M56000	05	S	08.00	0.912	VT	NVT	01	R0797	LEW 08/23/83	L-1AR	A
01	SI	SIH0000000052	0	CE	RS	M56000	05	S	09.00	0.912	VT	NVT	01	R0559	MSE 03/13/83	L-4FR	A
01	SI	SIH0000000053	0	CE	RS	M56000	05	S	08.00	0.912	VT	NVT	01	R0584	MSE 09/09/83	L-4FR	A
01	SI	SIH0000000054	0	CE	RS	M56000	05	S	09.00	0.912	VT	NVT	01	R0596	MSE 03/03/83	L-4FR	A
01	SI	SIH0000000055	0	CE	RS	M56000	05	S	09.00	0.912	VT	NVT	01	R0340	JEF 07/31/83	L-4AR NDIRSQ0022	R
01	SI	SIH0000000055	1	CE	RS	M56000	05	S	05.00	0.912	VT	NVT	01	R1051	RM8 09/01/83	L-4AR	A
01	SI	SIH0000000055	0	CE	RS	M56000	05	S	09.00	0.912	VT	NVT	01	R0028	JEF 07/31/83	L-4AR	A
01	SI	SIH0000000057	0	CE	RS	M56000	05	S	06.00	0.718	VT	NVT	01	R0337	MSE 03/03/83	L-3AR	A
01	SI	SIH0000000053	0	CE	RS	M56000	05	S	05.00	0.718	VT	NVT	01	R0366	MSE 03/03/83	L-3AR	A
01	SI	SIH0000000057	0	CE	RS	M56000	05	S	05.00	0.718	VT	NVT	01	R0309	MSE 03/03/83	L-3AR NDIRSQ0026	R
01	SI	SIH0000000059	1	CE	RS	M56000	05	S	06.00	0.718	VT	NVT	01	R1237	LB4 03/23/83	L-3AR	A
01	SI	SIH0000000059	0	CE	RS	M56000	05	S	06.00	0.718	VT	NVT	01	R0376	MSE 03/03/83	L-3AR	A
01	SI	SIH0000000061	0	BK2	RS	M56000	05	S	05.00	0.718	VT	NVT	01	R0280	MSE 03/03/83	L-3AR	A
01	SI	SIH0000000064	0	BK2	RS	M56000	05	S	06.00	0.718	VT	NVT	01	R0377	RM3 03/31/83	L-LC	A
01	SI	SIH0000000065	0	BK2	RS	M56000	05	S	06.00	0.718	VT	NVT	01	PC753	LE4 03/23/83	L-LC	A
01	SI	SIH0000000065	0	E	RS	M56000	05	S	06.00	0.718	VT	NVT	01	PC754	LE4 03/23/83	L-LC	A
01	SI	SIH0000000067	0	BK2	RS	M56000	05	S	10.00	1.000	VT	NVT	01	R1100	LE4 03/23/83	L-LC	A
01	SI	SIH0000000068	0	BK2	RS	M56000	05	S	10.00	1.000	VT	NVT	01	R1104	RM3 09/13/83	L-LC	A
01	SI	SIH0000000069	0	BK2	RS	M56000	05	S	10.00	1.000	VT	NVT	01	PC755	LE4 03/27/83	L-LC	A
01	SI	SIH0000000070	0	BK2	RS	M56000	05	S	10.00	1.000	VT	NVT	01	RM507	MSE 06/20/83	L-LC	A
01	SI	SIH0000000071	0	BK2	RS	M56000	05	S	10.00	1.000	VT	NVT	01	PC756	LE4 03/27/83	L-LC	A
01	SI	SIH0000000075	0	CE	RS	M56000	05	S	06.00	0.718	VT	NVT	01	R0508	MSE 03/23/83	L-2AR	A
01	SI	SIH0000000075	0	CE	RS	M56000	05	S	06.00	0.718	VT	NVT	01	P0792	LEW 03/23/83	L-2AR	A
01	SI	SIH0000000077	0	CE	RS	M56000	05	S	06.00	0.718	VT	NVT	01	P0609	MSE 03/20/83	L-2AR	A





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TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
1753 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
SECOYAH NUCLEAR PLANT  
P. O. BOX 2000  
DAISY, TENNESSEE  
UNIT 2 COMMERCIAL SERVICE DATE 05/01/82

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SYSTM	EXAM AREA	R	E	C	D	T	S	M	A	NDE	PRJC	BLOCK	REPT	NO.	NO.	P	DATE	COMMENTS
		NO.	NO.	NO.	NO.	NO.	NO.	NO.	NO.	NO.	NO.	NO.	NO.	NO.	NO.	NO.		
01 SI	SIH0000001161	3	BK2	RS	MSG000	07	S	01.50	0.281	VT	MUT	01	R0173				MSE 03/03/83 L-4FR	A
01 SI	SIH0000001162	3	BK2	RS	MSG000	07	S	01.50	0.281	VT	MUT	01	R0229				JEF 07/31/83 L-4AR	A
01 SI	SIH0000001163	3	BK2	RS	MSG000	07	S	01.50	0.281	VT	MUT	01	R0230				JEF 07/31/83 L-4AR	A
01 SI	SIH0000001164	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0188				JEF 07/23/83 L-RW	A
01 SI	SIH0000001165	3	BK2	RS	MSG000	07	S	03.00	0.438	VT	MUT	01	R0239				JEF 07/31/83 L-4AR N1M5Q0021	R
01 SI	SIH0000001166	1	BK2	RS	MSG000	07	S	03.00	0.438	VT	MUT	01	R117E				LN4 07/20/83 L-4AR	A
01 SI	SIH0000001167	3	BK2	RS	MSG000	07	S	03.00	0.438	VT	MUT	01	R0231				JEF 07/31/83 L-4AR	A
01 SI	SIH0000001168	3	BK2	RS	MSG000	07	S	03.00	0.438	VT	MUT	01	R0233				JEF 07/31/83 L-4AR	A
01 SI	SIH0000001169	3	BK2	RS	MSG000	07	S	03.00	0.438	VT	MUT	01	R0234				JEF 07/31/83 L-4AR	A
01 SI	SIH0000001170	3	BK2	RS	MSG000	11	S	01.50	0.281	VT	MUT	01	R0232				JEF 07/31/83 L-4AR	A
01 SI	SIH0000001171	3	BK2	RS	MSG000	11	S	01.50	0.281	VT	MUT	01	R1375				RH3 07/12/83 L-LC	A
01 SI	SIH0000001172	3	BK2	RS	MSG000	11	S	01.50	0.281	VT	MUT	01	R0302				LN4 08/23/83 L-LC	A
01 SI	SIH0000001173	3	BK2	RS	MSG000	11	S	01.50	0.281	VT	MUT	01	R0203				LF4 08/23/83 L-LC	A
01 SI	SIH0000001174	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0204				LF4 03/23/83 L-LC	A
01 SI	SIH0000001175	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0235				JEF 07/31/83 L-4AR	A
01 SI	SIH0000001176	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0189				JEF 07/23/83 L-RW	A
01 SI	SIH0000001177	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0190				JEF 07/23/83 L-RW	A
01 SI	SIH0000001178	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0237				MSE 03/03/83 L-5AR	A
01 SI	SIH0000001179	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0238				MSE 03/03/83 L-5AR	A
01 SI	SIH0000001180	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0239				MSE 03/03/83 L-5AR	A
01 SI	SIH0000001181	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0240				MSE 03/03/83 L-5AR	A
01 SI	SIH0000001182	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0189				RH4 07/23/83 L-RW	A
01 SI	SIH0000001183	3	BK2	RS	MSG000	07	S	02.50	0.375	VT	MUT	01	R0191				JEF 07/23/83 L-RW	A
01 SI	SIH0000001184	3	BK2	RS	MSG000	07	S	01.50	0.281	VT	MUT	01	R0105				LF4 08/23/83 L-LC	A

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1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
SEQUOYAH NUCLEAR PLANT  
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SYSTM	EXAM AREA	COO Y	EXAM AREA	DRWING	S	M	A	THICK	METH	NO.	PRDC	NO.	REPT	NO.	DATE	COMMENTS	R E S
01 SI	SIH0000002211	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	RC106	LE4	09/23/83	L-LC	A	
01 SI	SIH0000002212	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R1105	RM3	09/15/83	L-LC	A	
01 SI	SIH0000002213	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R1075	RM9	09/12/83	L-LC	A	
01 SI	SIH0000002214	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R0614	MSE	09/23/83	L-2AR	A	
01 SI	SIH0000002215	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R0390	RM3	03/31/83	L-2AR	A	
01 SI	SIH0000002216	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R0791	MSE	03/04/83	L-2FR	A	
01 SI	SIH0000002217	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R0792	MSE	03/04/83	L-2FR	A	
01 SI	SIH0000002218	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R0793	MSE	03/04/83	L-2FR	A	
01 SI	SIH0000002219	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R0508	MSE	03/03/83	L-2FR	N3I#SQ025	
01 SI	SIH0000002220	1	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R1177	LE4	09/23/83	L-2FR	A	
01 SI	SIH0000002221	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R0799	MSE	03/04/83	L-2FR	A	
01 SI	SIH0000002222	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R0790	MSE	03/04/83	L-2FR	A	
01 SI	SIH0000002223	0	BK2 RS	MSG0007	07	S	01.50	0.281	VT	NUT	01	R0771	MSE	03/03/83	L-3AR	A	
01 SI	SIH0000002224	0	BK2 RS	MSG0007	10	S	01.50	0.281	VT	NUT	01	R0307	LE4	03/23/83	L-LC	A	
01 SI	SIH0000002225	0	BK2 RS	MSG0007	10	S	01.50	0.281	VT	NUT	01	R0308	LE4	03/23/83	L-LC	A	
01 SI	SIH0000002226	0	BK2 RS	MSG0007	10	S	01.50	0.281	VT	NUT	01	R0309	LE4	03/23/83	L-LC	A	
01 SI	SIH0000002227	0	BK2 RS	MSG0007	10	S	01.50	0.281	VT	NUT	01	R0310	LE4	03/23/83	L-LC	A	
01 SI	SIH0000002228	0	BK2 RS	MSG0007	10	S	01.50	0.281	VT	NUT	01	R0772	MSE	03/03/83	L-3AR	A	
01 SI	SIH0000002229	0	BK2 RS	MSG0007	07	S	02.50	0.375	VT	NUT	01	R0274	MSE	03/03/83	L-3AR	A	
01 SI	SIH0000002230	0	BK2 RS	MSG0007	12	S	02.00	0.343	VT	NUT	01	R0381	RM3	03/31/83	L-LC	A	
01 SI	SIH0000002231	0	BK2 RS	MSG0007	12	S	02.00	0.343	VT	NUT	01	R0382	RM3	03/31/83	L-LC	A	
01 SI	SIH0000002232	0	BK2 RS	MSG0007	12	S	02.00	0.343	VT	NUT	01	R0383	RM6	03/31/83	L-LC	A	
01 SI	SIH0000002233	0	BK2 RS	MSG0007	12	S	02.00	0.343	VT	NUT	01	R0304	RM3	03/31/83	L-LC	A	

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TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 DAIKY, TENNESSEE  
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EXAM AREA	R E C O D Y	T	S H A	M	NDE	PROC	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S
SI SIH0000003364	0 BK2 RS	MSG0009	12 S	02.00	0.343	VT	NVT 01		R0395	RM3	03/31/83	L-LC	A
SI SIH0000003365	0 BK2 RS	MSG0009	12 S	02.00	0.343	VT	NVT 01		R0386	RM3	03/31/83	L-LC	A
SI SIH0000003366	0 BK2 RS	MSG0009	12 S	02.00	0.343	VT	NVT 01		R0387	RM3	03/31/83	L-2FR	A
SI SIH0000003413	0 CE VS	MSG0009	05 S	16.00	0.500	VT	NVT 01		R0403	JEF	03/13/83	L-RHX2A	A
SI SIH0000003415	0 CE RS	MSG0009	05 S	16.00	0.500	VT	NVT 01		R0401	JEF	03/13/83	L-RHX2A	A
SI SIH0000003421	0 CE RS	MSG0007	01 S	08.00	0.322	VT	NVT 01		R0570	JEF	03/13/83	L-RHX2B	A
SI SIH0000003422	0 CE RS	MSG0007	01 S	08.00	0.322	VT	NVT 01		R0582	JEF	03/09/83	L-RHX2B	A
SI SIH0000003423	0 CE RS	MSG0007	01 S	08.00	0.322	VT	NVT 01		R0583	JEF	03/09/83	L-RHX2B	A
SI SIH0000003424	0 CC IA	MSG0007	01 S	09.00	0.322	PT	NPT 04		R0556	JEF	03/13/83	L-RHX2B	A
SI SIH0000003424	0 CE RS	MSG0009	01 S	08.00	0.322	VT	NVT 01		R0570	JEF	03/13/83	L-RHX2B	A
SI SIH0000003425	0 CE RS	MSG0009	01 S	08.00	0.322	VT	NVT 01		R0552	JEF	03/03/83	L-PC590	A
SI SIH0000003426	0 CE RS	MSG0009	01 S	08.00	0.322	VT	NVT 01		R0353	JEF	03/03/83	L-PC590	A
SI SIH0000003427	0 CE RS	MSG0009	01 S	08.00	0.322	VT	NVT 01		R0493	JEF	03/15/83	L-PC590	A
SI SIH0000003428	0 CE RS	MSG0009	01 S	09.00	0.322	VT	NVT 01		R0494	JEF	03/15/83	L-PC590	A
SI SIH0000003429	0 CE RS	MSG0009	01 S	08.00	0.312	VT	NVT 01		R0495	JEF	03/15/83	L-PC590	A
SI SIH0000003430	0 CE RS	MSG0009	05 S	08.00	0.322	VT	NVT 01		R0571	JEF	03/13/83	L-RHX2B	A
SI SIH0000003431	0 CE RS	MSG0009	05 S	08.00	0.322	VT	NVT 01		R0329	LB1	07/24/83	L-PC590	A
SI SIH0000003446	0 CE RS	MSG0009	05 S	16.00	0.500	VT	NVT 01		R0402	JEF	03/13/83	L-RHX2A	A
SI SIH0000003450	0 CE RS	MSG0009	05 S	08.00	0.322	VT	NVT 01		R049F	JEF	03/15/83	L-PC590	A
SI SIH0000003451	0 CE RS	MSG0009	06 S	08.00	0.322	VT	NVT 01		R0730	LF4	07/24/83	L-PC590	A
SI SIH0000003452	0 CE RS	MSG0009	05 S	08.00	0.322	VT	NVT 01		R0521	JEF	03/15/83	L-RHX2A	A
SI SIH0000003453	0 CE VS	MSG0009	05 S	08.00	0.322	VT	NVT 01		R0520	JEF	03/15/83	L-RHX2A NDI8500043	R
SI SIH0000003453	1 CE VS	MSG0009	05 S	08.00	0.322	VT	NVT 01		R1147	MSE	03/15/83	L-RHX2A	A
SI SIH0000003454	0 CE VS	MSG0009	04 S	09.00	0.322	VT	NVT 01		R0522	JEF	03/15/83	L-RHX2A	A

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TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. Box 2000 DAVIS, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 05/01/82

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Y	SYSTEM	EXPH AREA	E CAT	T P	S H	M C	NDE THICK	PROC METH	NO.	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S
01	SI	SIH0000000455	0	CE	RS	M560007	04 S 08.00 0.322	UT	NVT 01		RC57		JEF	03/15/83	L-RHX2A	A
01	SI	SIH0000000455	0	CE	RS	M560007	04 S 08.00 0.322	UT	NVT 01		RC57		JEF	08/15/83	L-RHX2A	A
01	SI	SIH0000000457	0	CE	RS	M560007	04 S 08.00 0.322	UT	NVT 01		PC525		JEF	03/15/83	L-RHX2A	A
01	SI	SIH0000000458	0	CE	RS	M560007	04 S 08.00 0.322	UT	NVT 01		RD497		JEF	03/15/83	L-PC590	A
01	SI	SIH0000000459	0	CE	RS	M560007	04 S 08.00 0.322	UT	NVT 01		RD498		JEF	03/15/83	L-PC590	A
01	SI	SIH0000000460	0	CE	RS	M560007	04 S 09.00 0.322	UT	NVT 01		RD496		JEF	03/03/83	L-PC590	A
01	SI	SIS0000000015	0	CF	DW	ISI0002	01 S 08.00 0.312	PT	NPT 04		RC743		LE4	03/02/83	L-PC590	A
01	SI	SIS0000000017	0	CF	DW	ISI0002	01 S 08.00 0.312	PT	NPT 04		RC744		LE4	03/02/83	L-PC590	A
								UT45	SQUT27	SQ 02	RC590	CO216	FS4	03/13/83	L-PC590	A
01	SI	SIS0000000019	0	CF	DW	ISI0002	01 S 08.00 0.312	PT	NPT 04		RC506		MSE	09/15/83	L-PC590	A
								UT45	SQUT27	SQ 02	RC591	CO216	FS4	08/13/83	L-PC590	A
01	SI	SIS0000000021	0	CF	DW	ISI0002	02 S 08.00 0.322	PT	NPT 04		RC507		MSE	03/15/83	L-PC590	A
01	SI	SIS0000000025	0	CF	DW	ISI0002	02 S 08.00 0.312	PT	NPT 04		RC508		MSE	03/15/83	L-PC590	A
								UT45	SQUT27	SQ 02	RC592	CO216	FS4	03/13/83	L-PC590	A
01	SI	SIS0000000026	0	BJ	DW	ISI0002	05 S 05.00 0.718	PT	NPT 04		RC719		MSE	03/25/83	L-1AR	A
								UT45	SQUT27	SQ 01	RC775	CO046	LM4	03/27/83	L-1AR	A
01	SI	SIS0000000028	0	BJ	DW	ISI0002	05 S 02.00 0.343	PT	NPT 04		RC719		MSE	08/25/83	L-1AR	A
01	SI	SIS0000000029	0	BJ	DW	ISI0002	05 S 06.00 0.718	PT	NPT 04		RC348		GDF	03/03/83	L-LC	A
								UT45	SQUT27	SQ 01	RC555	CO025	LM4	03/17/83	L-LC	A
01	SI	SIS0000000027	0	BJ	DW	ISI0002	05 S 06.00 0.718	PT	NPT 04		RC720		MSE	03/25/83	L-3AR	A
								UT45	SQUT27	SQ 01	RC776	CO046	LM4	03/27/83	L-3AR	A
01	SI	SIS0000000028	0	BJ	DW	ISI0002	05 S 05.00 0.718	PT	NPT 04		PC591		WJD	03/17/83	L-LC	A
								UT45	SQUT27	SQ 01	RC599	CO034	FS4	03/27/83	L-LC	A
01	SI	SIS0000000030	0	BJ	DW	ISI0002	05 S 06.00 0.718	PT	NPT 04		RC721		RM3	03/25/83	L-LC	A
								UT45	SQUT27	SQ 01	RC715	CO034	FS4	09/27/83	L-LC	A
01	SI	SIS0000000024	0	BJ	DW	ISI0002	07 S 03.00 0.438	PT	NPT 04		RC746		JEF	03/02/83	L-4AR	A
01	SI	SIS0000000022	0	BJ	DW	ISI0002	07 S 03.00 0.438	PT	NPT 04		RC747		JEF	03/02/83	L-4AR	A
01	SI	SIS0000000125	0	BJ	DW	ISI0002	03 S 01.50 0.201	PT	NPT 04		RC749		RM3	03/03/83	L-LC	A

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1750 CHESTNUT STREET TOWER II, CHATTANOOGA, TENNESSEE 37402  
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SYSTH	EXAM AREA	Z	Y	CODE	Y	Y	S	M	H	A	DRAWING	N	T	SIZE	THICK	HT	NO.	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I	N	P	DATE	COMMENTS	R	E	S
01 SI	SI0000001127	0	BJ	SW	ISI0002	C0	S	01.50	0.291	PT	NPT	04						R0350		R0350	GDF			09/03/83	L-LC	A		
01 SI	SI0000001131	0	BJ	SW	ISI0002	C3	S	01.50	0.291	PT	NPT	04						R0351		R0351	GDF			03/03/83	L-LC	A		
01 SI	SI0000001135	0	BJ	SW	ISI0002	C3	S	01.50	0.291	PT	NPT	04						R1113		R1113	MSE			09/14/83	L-LC	A		
01 SI	SI0000001915	0	BJ	SW	ISI0002	C7	S	01.50	0.291	PT	NPT	04						R0316		R0316	WJD			09/23/83	L-LC	A		
01 SI	SI0000001920	0	BJ	SW	ISI0002	C7	S	01.50	0.291	PT	NPT	04						R0317		R0317	WJD			09/23/83	L-LC	A		
01 SI	SI0000001923	0	BJ	SW	ISI0002	C7	S	01.50	0.291	PT	NPT	04						R0318		R0318	GDF			03/23/83	L-LC	A		
01 SI	SI0000001923	0	BJ	SW	ISI0002	C7	S	01.50	0.291	PT	NPT	04						R0319		R0319	MSE			03/23/83	L-2AR	A		
01 SI	SI0000001923	0	BJ	SW	ISI0002	C7	S	01.50	0.291	PT	NPT	04						R0375		R0375	MSE			08/05/83	L-3AR	A		
01 SI	SI0000001937	0	BJ	SW	ISI0002	10	S	01.50	0.291	PT	NPT	04						R0520		R0520	GDF			03/23/83	L-LC	A		
01 SI	SI0000001991	0	BJ	SW	ISI0002	10	S	01.50	0.291	PT	NPT	04						R0521		R0521	GDF			08/23/83	L-LC	A		
01 SI	SI0000001995	0	BJ	SW	ISI0002	10	S	01.50	0.291	PT	NPT	04						R0522		R0522	GDF			09/20/83	L-LC	A		
01 SI	SI0000001701	0	BJ	SW	ISI0002	11	S	01.50	0.291	PT	NPT	04						R0422		R0422	GDF			03/13/83	L-LC	A		
01 SI	SI0000001705	0	BJ	SW	ISI0002	11	S	01.50	0.291	PT	NPT	04						PC423		PC423	GDF			09/13/83	L-LC	A		
01 SI	SI0000001913	0	BJ	SW	ISI0002	11	S	01.50	0.291	PT	NPT	04						PC549		PC549	GDF			03/17/83	L-LC	A		
01 SI	SI0000211439C	0	B62	FR	ISI0002	03	S	01.50	0.291	VT	NUT	01						PC707		PC707	JEF			07/29/83	L-1AR	A		
01 SI	65-0000002067	0	B62	W3	ISI0002	04	S	10.00	1.000	VT	NUT	01						R0312		R0312	JEF			07/31/83	L-4AR	A		
01 SI	65-0000002113	0	B62	W3	ISI0002	05	S	10.00	1.000	VT	NUT	01						S0390		S0390	MSE			08/09/83	L-1AR	A		
01 SI	65-00000021560	0	B62	W3	ISI0002	05	S	10.00	1.000	VT	NUT	01						R1741		R1741	RM3			08/17/83	L-LC	A		
01 SI	65-00000023563	0	B62	W3	ISI0002	04	S	10.00	1.000	VT	NUT	01						PC442		PC442	RM3			03/11/83	L-LC	A		
01 SI	65-0000002622	0	B62	W3	ISI0002	05	S	10.00	1.000	VT	NUT	01						PC453		PC453	RM3			03/11/83	L-LC	A		
01 SI	65-0000002625	0	B62	W8	ISI0002	04	S	10.00	1.000	VT	NUT	01						R0431		R0431	RM3			09/11/83	L-LC	A		
01 SI	65-0000002653	0	B62	W8	ISI0002	05	S	06.00	0.718	VT	NUT	01						RC208		RC208	JEF			07/23/83	L-1AR	A		
01 UHI	UHI0000000001	0	B42	RS	M560016	C1	S	12.00	1.125	VT	NUT	01						R0327		R0327	JEF			09/30/83	L-RW	A		
01 UHI	UHI0000000002	0	B42	RS	M560016	C1	S	12.00	1.125	VT	NUT	01						R0328		R0328	JEF			03/30/83	L-RW	A		

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TENNESSEE VALLEY AUTHORITY, DIVISION OF NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOIAH NUCLEAR PLANT  
 P. O. BOX 2000 DAISSY, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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SYSTM	EXAM AREA	CODE	TYP	S	M	H	A	ND	PR	COL	REP	COL	I	DATE	COMMENTS	RES		
		CAT	P	DRAWING	F	T	SIZE	THICK	METH	NO.	BLOCK NO.	REPT NO.	REPT NO.	N P				
01 UHI	UHI4000000003	0	BK2	RS	MSG0014	01	S	12.00	1.125	VT	NVT	01	R0329	JEF	08/30/83	L-RW	A	
01 UHI	UHI4000000004	0	BK2	RS	MSG0014	01	S	12.00	1.125	VT	NVT	01	R0330	JEF	09/30/83	L-LC	A	
01 UHI	UHI4000000005	0	BK2	RS	MSG0014	01	S	12.00	1.125	VT	NVT	01	R0331	JEF	03/30/83	L-LC	A	
01 UHI	UHI4000000006	0	BK2	VS	MSG0014	03	S	09.00	0.912	VT	NVT	01	R0332	JEF	08/30/83	L-LC	A	
01 UHI	UHI4000000007	0	BK2	MS	MSG0014	03	S	08.00	0.912	VT	NVT	01	R0333	JEF	03/30/83	L-LC	A	
01 UHI	UHI4000000008	0	BK2	RS	MSG0014	03	S	08.00	0.912	VT	NVT	01	R0334	JEF	08/30/83	L-LC	A	
01 UHI	UHI4000000009	0	BK2	MS	MSG0014	03	S	08.00	0.912	VT	NVT	01	R0335	JEF	09/30/83	L-LC	A	
01 UHI	UHI4000000010	0	BK2	MS	MSG0014	03	S	08.00	0.912	VT	NVT	01	R0308	JEF	03/31/83	L-LC	NDI#SQ0056	R
01 UHI	UHI4000000010	1	BK2	MS	MSG0014	03	S	08.00	0.912	VT	NVT	01	R1114	BGR	09/14/83	L-LC	A	
01 UHI	UHI4000000011	0	BK2	MS	MSG0014	03	S	09.00	0.912	VT	NVT	01	R0309	JEF	09/31/83	L-LC	A	
01 UHI	UHI4000000012	0	BK2	MS	MSG0014	03	S	08.00	0.912	VT	NVT	01	R0300	JEF	09/31/83	L-LC	A	
01 UHI	UHI4000000013	0	BK2	MS	MSG0014	03	S	09.00	0.912	VT	NVT	01	R0391	JEF	03/31/83	L-LC	NDI#SQ0057	R
01 UHI	UHI4000000015	1	BK2	MS	MSG0014	03	S	08.00	0.912	VT	NVT	01	R1179	LBH	09/20/83	L-LC	A	
01 UHI	UHI4000000014	0	BK2	MS	MSG0014	03	S	08.00	0.912	VT	NVT	01	R0302	JEF	09/31/83	L-UC	A	
01 UHI	UHI4000000015	0	BK2	MS	MSG0014	03	S	08.00	0.906	VT	NVT	01	R1328	JNS	09/26/83	L-UC	A	
01 UHI	UHI4000000016	0	BK2	MS	MSG0014	03	S	08.00	0.906	VT	NVT	01	R1190	BGR	07/21/83	L-UC	A	
01 UHI	UHI4000000017	0	BK2	VS	MSG0014	03	S	08.00	0.906	VT	NVT	01	R1331	BBH	07/23/83	L-UC	A	
01 UHI	UHI4000000018	0	BK2	MS	MSG0014	03	S	08.00	0.906	VT	NVT	01	R1197	BGF	09/20/83	L-UC	NDI#SQ0085 USE-AS-IS USQ0033-28	A
01 UHI	UHI4000000019	0	BK2	MS	MSG0014	03	S	08.00	0.905	VT	NVT	01	R1191	BGF	09/17/83	L-UC	A	
01 UHI	UHI4000000022	0	BK2	MS	MSG0014	03	S	08.00	0.906	VT	NVT	01	R1173	BGF	09/19/83	L-UC	A	
01 UHI	UHI4000000023	0	BK2	MS	MSG0014	03	S	08.00	0.906	VT	NVT	01	R1172	BGF	09/19/83	L-UC	A	
01 UHI	UHI4000000024	0	BK2	RS	MSG0014	01	S	08.00	0.912	VT	NVT	01	R0336	JEF	09/30/83	L-LC	A	
01 UHI	UHI4000000025	0	BK2	RS	MSG0014	03	S	08.00	0.912	VT	NVT	01	R0337	JEF	09/30/83	L-LC	A	
01 UHI	UHI4000000026	0	BK2	VS	MSG0014	03	S	08.00	0.912	VT	NVT	01	R0338	JEF	03/30/83	L-LC	A	

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TENNESSEE VALLEY AUTHORITY, DIVISION OF NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 CHATSWORTH, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 05/01/82

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SYSTM	EXAM AREA	R E C O D E	T Y P E	S M P L E	H A Z A R D	SIZE	THICK	METH	NDE PROC NO.	CAL BLOCK NO.	REPT NO.	CAL REPT NO.	I N P	DATE	COMMENTS	R E S		
01 UHI	UHI4000000027	0	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0839	JEF	09/30/83	L-LC	A	
01 UHI	UHI4000000028	0	BK2	RS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0840	JEF	09/30/83	L-LC	A	
01 UHI	UHI4000000029	0	BK2	VS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0841	JEF	03/30/83	L-LC	A	
01 UHI	UHI4000000030	0	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0842	JEF	09/30/83	L-LC	A	
01 UHI	UHI4000000031	0	BK2	RS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0843	JEF	09/30/83	L-LC	A	
01 UHI	UHI4000000032	0	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0844	JEF	09/30/83	L-LC	A	
01 UHI	UHI4000000033	0	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0893	JEF	03/31/83	L-LC	A	
01 UHI	UHI4000000034	0	BK1	IA	MSG0014	C3	S	03.00	0.912	PT	NPT	04	R0717	JEF	09/01/83	L-LC	A	
01 UHI	UHI4000000034	0	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0894	JEF	08/31/83	L-LC	A	
01 UHI	UHI4000000035	0	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0895	JEF	03/31/83	L-LC	NDI#SQ0058	R
01 UHI	UHI4000000035	1	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	P1115	BGR	09/14/83	L-LC	A	
01 UHI	UHI4000000035	0	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0896	JEF	03/31/83	L-LC	A	
01 UHI	UHI4000000037	0	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0708	JEF	03/31/83	L-LC	A	
01 UHI	UHI4000000038	0	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R0797	JEF	03/31/83	L-UC	NDI#SQ0059	R
01 UHI	UHI4000000038	1	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	P1116	BGR	07/14/83	L-UC	A	
01 UHI	UHI4000000037	0	BK2	MS	MSG0014	C3	S	08.00	0.312	VT	NVT	01	R1132	LHM	09/21/83	L-UC	A	
01 UHI	UHI4000000040	0	BK2	VS	MSG0014	C3	S	08.00	0.906	VT	NVT	01	P1185	BGF	09/23/83	L-UC	A	
01 UHI	UHI4000000041	0	BK2	MS	MSG0014	C3	S	08.00	0.906	VT	NVT	01	P1185	GR9	07/19/83	L-UC	A	
01 UHI	UHI4000000042	0	BK2	MS	MSG0014	C3	S	08.00	0.906	VT	NVT	01	P1197	BGF	09/23/83	L-UC	A	
01 UHI	UHI4000000045	0	BK2	MS	MSG0014	C3	S	08.00	0.906	VT	NVT	01	P1198	GR9	09/19/83	L-UC	A	
01 UHI	UHI4000000045	0	BK2	MS	MSG0014	C3	S	08.00	0.906	VT	NVT	01	P1198	BGF	09/19/83	L-UC	NDI#SQ0086 USE-AS-I S USQD#92-28	A
01 UHI	UHI4000000047	0	BK2	RS	MSG0014	C1	S	12.00	1.125	VT	NVT	01	R0845	JEF	09/30/83	L-LC	A	
01 UHI	UHI4000000060	0	BK2	RS	MSG0014	C1	S	12.00	1.125	VT	NVT	01	R0846	JEF	09/30/83	L-RW	A	
01 UHI	UHI4000000061	0	BK2	RS	MSG0014	C1	S	12.00	1.125	VT	NVT	01	R0847	JEF	08/30/83	L-RW	A	

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1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
SCOVILLE NUCLEAR PLANT  
P. O. BOX 2000  
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C	Y	SYSTEM	EXAM AREA	Q	Y	S	M	A	DRAGING	T	SIZE	TRICK METH	NDE	PFOC	NO.	CEL	GLCK	REPT	NO.	CEL	REPT	NO.	I	N	P	DATE	COMMENTS	R	E	S
01	UHI	UHIH0000000062	0	BK2	RS	M5G0014	C1	S	12.00	1.125	VT	NVT	01			R0948						JEF	09/30/83	L-RW					A	
01	UHI	UHIH0000000063	0	BK2	RS	M5G0014	C1	S	12.00	1.125	VT	NVT	01			R0949							JEF	09/30/83	L-LC					A
01	UHI	UHIH0000000064	0	BK2	RS	M5G0014	C1	S	12.00	1.125	VT	NVT	01			R0950							JEF	09/30/83	L-LC					A
01	UHI	UHIH0000000065	0	BK2	RS	M5G0014	C2	S	12.00	1.125	VT	NVT	01			R0951							JEF	09/30/83	L-LC					A
01	UHI	UHIH0000000066	0	BK2	RS	M5G0014	C2	S	09.00	0.912	VT	NVT	01			R0952							JEF	09/30/83	L-LC					A
01	UHI	UHIH0000000067	0	BK2	RS	M5G0014	C2	S	09.00	0.912	VT	NVT	01			R0953							JEF	03/30/83	L-LC					A
01	UHI	UHIH0000000068	0	BK2	RS	M5G0014	C2	S	09.00	0.912	VT	NVT	01			R0954							JEF	09/31/83	L-LC					A
01	UHI	UHIH0000000069	0	BK2	RS	M5G0014	C2	S	09.00	0.912	VT	NVT	01			R0955							JEF	09/31/83	L-LC					A
01	UHI	UHIH0000000070	0	BK2	RS	M5G0014	C2	S	09.00	0.912	VT	NVT	01			R0956							JEF	09/31/83	L-LC					A
01	UHI	UHIH0000000071	0	BK2	RS	M5G0014	C2	S	09.00	0.912	VT	NVT	01			R0957							JEF	09/31/83	L-LC					A
01	UHI	UHIH0000000072	0	BK2	RS	M5G0014	C2	S	09.00	0.912	VT	NVT	01			R0958							JEF	09/31/83	L-LC					A
01	UHI	UHIH0000000073	1	BK2	RS	M5G0014	C2	S	09.00	0.912	VT	NVT	01			R0959							JEF	09/31/83	L-LC					A
01	UHI	UHIH0000000074	0	BK2	RS	M5G0014	C2	S	09.00	0.905	VT	NVT	01			R1117							MOR	09/14/83	L-LC					A
01	UHI	UHIH0000000075	0	BK2	RS	M5G0014	C2	S	09.00	0.905	VT	NVT	01			R1118							LRM	09/29/83	L-UC					A
01	UHI	UHIH0000000076	0	BK2	RS	M5G0014	C2	S	09.00	0.905	VT	NVT	01			R1119							GRB	09/13/83	L-UC					A
01	UHI	UHIH0000000077	0	BK2	RS	M5G0014	C2	S	09.00	0.905	VT	NVT	01			R1120							JNS	09/25/83	L-UC					A
01	UHI	UHIH0000000078	0	BK2	RS	M5G0014	C2	S	09.00	0.905	VT	NVT	01			R1121							BJM	09/21/83	L-UC					A
01	UHI	UHIH0000000079	0	BK2	RS	M5G0014	C2	S	09.00	0.905	VT	NVT	01			R1122							BJM	09/21/83	L-UC					A
01	UHI	UHIH0000000080	0	BK2	RS	M5G0014	C1	S	05.00	0.525	VT	NVT	01			R0954							GRB	09/13/83	L-UC					A
01	UHI	UHIH0000000081	0	BK2	RS	M5G0014	C1	S	02.00	0.343	VT	NVT	01			R0955							JEF	09/30/83	L-RW					A
01	UHI	UHIH0000000082	0	BK2	RS	M5G0014	C1	S	02.00	0.343	VT	NVT	01			R0956							JEF	09/30/83	L-RW					A
01	UHI	UHIH0000000083	0	BK2	RS	M5G0014	C1	S	02.00	0.343	VT	NVT	01			R0957							JEF	09/30/83	L-RW					A
01	UHI	UHIH0000000084	0	CE	RS	M5G0014	C4	S	12.00	1.125	VT	NVT	01			R0946							RM9	07/25/83	L-REF					A
01	UHI	UHIH0000000085	0	CC	IA	M5G0014	C4	S	12.00	1.125	PT	NVT	04			R0916							LRM	07/25/83	L-REF					A

12/15/80

TENNESSEE VALLEY AUTHORITY, DIVISION OF NUCLEAR POWER  
1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
SEQUOYAH NUCLEAR PLANT  
P. O. BOX 2000  
UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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SYSTEM	EXAM AREA	R	F	S	H	DRAWING #	T	SIZE	THICK	METH	NO.	PRIC	NO.	CAL	NO.	PEPT	NO.	I	NO.	DATE	COMMENTS	R
SYSTEM	EXAM AREA	CODE	CLT	Y	M	Y	Y	Y	Y	Y	Y	Y	Y	NO.	NO.	NO.	NO.	NO.	NO.	NO.	NO.	NO.
01 UHI	UHI40000001101	0	CE	MS	MS60014	04	S	12.00	1.125	VT	NUT	01		RC007				LBM	07/21/83	I-4E9	A	
01 UHI	UHI40000001102	0	CC	IA	MS60014	04	S	12.00	1.125	PT	NPT	04		RC325				LBM	09/05/83	L-4E9	A	
01 UHI	UHI40000001103	0	CE	RS	MS60014	04	S	12.00	1.125	VT	NUT	01		RC009				LBM	07/23/83	L-4E9	A	
01 UHI	UHI40000001104	0	CE	VS	MS60014	04	S	12.00	1.125	VT	NUT	01		RC009				LBM	07/23/83	L-4E9	A	
01 UHI	UHI40000001104	0	CC	IA	MS60014	04	S	12.00	1.125	PT	NPT	04		RC326				LBM	09/05/83	L-4E9	A	
01 UHI	UHI40000001104	0	CE	RS	MS60014	04	S	12.00	1.125	VT	NUT	01		RC010				LBM	07/23/83	L-4E9	A	
01 UHI	UHI40000001105	0	CC	IA	MS60014	04	S	12.00	1.125	PT	NPT	04		RC327				LBM	09/05/83	L-4E9	A	
01 UHI	UHI40000001105	0	CE	MS	MS60014	04	S	12.00	1.125	VT	NUT	01		RC011				LBM	07/23/83	L-4E9	A	
01 UHI	UHI40000001105	0	CE	RS	MS60014	04	S	12.00	1.125	VT	NUT	01		RC045				RH3	07/25/83	L-4E9	A	
01 UHI	UHI40000001130	0	BK2	MS	MS60014	02	S	05.00	0.525	VT	NUT	01		R1171				BGF	09/13/83	L-UC	A	
01 UHI	UHI40000001131	0	BK2	RS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC359				JEF	09/29/83	L-LC	NOT#SQ0055	
01 UHI	UHI40000001131	1	BK2	RS	MS60014	02	S	09.00	0.712	VT	NUT	01		R1119				BGR	09/14/83	L-LC	A	
01 UHI	UHI40000001132	0	BK2	RS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC359				JEF	09/30/83	L-LC	A	
01 UHI	UHI40000001133	0	BK2	MS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC366				JEF	09/30/83	L-LC	A	
01 UHI	UHI40000001134	0	BK2	VS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC350				JEF	09/30/83	L-LC	A	
01 UHI	UHI40000001135	0	BK2	RS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC061				JEF	09/30/83	L-LC	A	
01 UHI	UHI40000001135	0	BK2	RS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC362				JEF	09/30/83	L-LC	A	
01 UHI	UHI40000001137	0	BK2	VS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC065				JEF	09/30/83	L-LC	A	
01 UHI	UHI40000001138	0	BK2	MS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC367				JEF	09/30/83	L-LC	A	
01 UHI	UHI40000001139	0	BK2	MS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC064				JEF	09/30/83	L-LC	A	
01 UHI	UHI40000001140	0	BK2	MS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC063				JEF	09/30/83	L-LC	NOT#SQ0061	
01 UHI	UHI40000001140	1	BK2	MS	MS60014	02	S	09.00	0.712	VT	NUT	01		R1117				BGR	09/14/83	L-LC	A	
01 UHI	UHI40000001141	0	BK2	VS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC064				JEF	09/31/83	L-LC	A	
01 UHI	UHI40000001142	0	BK2	MS	MS60014	02	S	09.00	0.712	VT	NUT	01		RC065				JEF	09/31/83	L-LC	A	

12/15/81

TENNESSEE VALLEY AUTHORITY, DIVISION NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II CHATTANOOGA, TENNESSEE 37402  
 SEQUOYAH NUCLEAR PLANT  
 P. O. BOX 2000 DAVIS, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 05/01/82

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EXAM AREA	Q	T	S	M	NDE	PPDC	CAL	I	CAL	I	DATE	COMMENTS	R				
SYSTM	EXAM AREA	Q CAT	T P	DRAWING	H A	Y SIZE	THICK	METH	NO.	REPT NO.	REPT NO.	P	DATE	COMMENTS	R		
01 UHI	UHI4000003143	0	BK2	MS	MSG0014	02	S	08.00	0.312	VT	NVT	01	R0905	JEF 08/31/83 L-LC	NOI#500762	R	
01 UHI	UHI4000003143	1	BK2	MS	MSG0014	02	S	08.00	0.312	VT	NVT	01	R1120	MDR 09/14/83 L-LC		A	
01 UHI	UHI4000003144	0	BK2	MS	MSG0014	02	S	09.00	0.912	VT	NVT	01	R0707	JEF 08/31/83 L-UC		A	
01 UHI	UHI4000003145	0	BK2	MS	MSG0014	02	S	08.00	0.706	VT	NVT	01	R1192	BGF 09/17/83 L-UC		A	
01 UHI	UHI4000003146	0	BK2	VS	MSG0014	02	S	08.00	0.906	VT	NVT	01	R1193	BGF 09/23/83 L-UC		A	
01 UHI	UHI4000003147	0	BK2	MS	MSG0014	02	S	08.00	0.905	VT	NVT	01	R1194	BGF 09/17/83 L-UC		A	
01 UHI	UHI4000003148	0	BK2	MS	MSG0014	02	S	08.00	0.706	VT	NVT	01	R1149	BGF 09/19/83 L-UC		A	
01 UHI	UHI4000003149	0	BK2	MS	MSG0014	02	S	08.00	0.905	VT	NVT	01	R1150	BGF 09/17/83 L-UC		A	
01 UHI	UHI4000003152	0	BK2	MS	MSG0014	02	S	05.00	0.525	VT	NVT	01	R1195	GRB 09/17/83 L-UC	NOI#500083 USE-AS-IS US00#83-28	A	
01 UHI	UHI4000003153	0	BK2	MS	MSG0014	02	S	05.00	0.525	VT	NVT	01	R1196	GRB 09/17/83 L-UC	NOI#500084 USE-AS-IS US00#83-28	A	
01 UHI	UPIF0000003017	0	CF	CW	ISI0001	01	S	12.00	1.125	PT	NPT	04	R0073	LEW 07/25/83 L-RW		A	
													UT45 SQU27 SQ 03 R0564 C0027	FS4 03/21/83 L-RW		A	
01 UHI	UPIF0000003019	0	BJ	CW	ISI0001	01	S	12.00	1.125	PT	NPT	04	R0076	LE4 07/27/83 L-RW		A	
													UT45 SQU27 SQ 03 R0334 C0004	EW5 03/05/83 L-RW		A	
01 UHI	UPIF0000003021	0	BJ	CW	ISI0001	01	S	12.00	1.125	PT	NPT	04	R0203	MSE 08/03/83 L-LC		A	
													UT45 SQU27 SQ 03 R0342 C0004	EW5 08/03/83 L-LC		A	
01 UHI	UPIF0000003023	0	BJ	CW	ISI0001	01	S	12.00	1.125	PT	NPT	04	R0429	MSE 08/11/83 L-LC		A	
													UT45 SQU27 SQ 03 R0553 C0027	FS4 08/21/83 L-LC		A	
01 UHI	UPIF0000003043	0	CF	CW	ISI0001	02	S	12.00	1.125	PT	NPT	04	R0171	LEW 07/27/83 L-RW		A	
													UT45 SQU27 SQ 03 R0337 C0004	EW5 08/05/83 L-RW		A	
01 UHI	UPIF0000003045	0	BJ	CW	ISI0001	02	S	12.00	1.125	PT	NPT	04	R0173	LE4 07/27/83 L-RW		A	
													UT45 SQU27 SQ 03 R0338 C0004	EW5 03/05/83 L-RW		A	
01 UHI	UPIF0000003880	0	BG2	FB	ISI0001	01	S	09.00	0.905	PT	NPT	07	R1163	L0X 09/17/83 L-UC	IEB#82-02	A	
													VT NVT 01	R1155	L0X 09/17/83 L-UC	IEB#82-02	A
01 UHI	UPIF0000003180	0	BG2	FB	ISI0001	01	S	08.00	0.706	PT	NPT	07	R1164	L0X 09/17/83 L-UC	IEB#82-02	A	
													VT NVT 01	R1156	L0X 09/17/83 L-UC	IEB#82-02	A
01 UHI	UPIF0000003680	0	BG2	FB	ISI0001	01	S	08.00	0.905	PT	NPT	07	R1166	L0X 09/17/83 L-UC	IEB#82-02	A	

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12/15/83

TENNESSEE VALLEY AUTHORITY, DIVISION OF NUCLEAR POWER  
 1750 CHESTNUT STREET TOWER II, CHATTANOOGA, TENNESSEE 37402  
 SECURIAH NUCLEAR PLANT  
 P. O. BOX 2000, DAISY, TENNESSEE  
 UNIT 2 COMMERCIAL SERVICE DATE 06/01/82

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E Y S	SYSTM	EXAM AREA	R E C #	T C O D E P	S M H A T	D R A W I N G	S I Z E	T H I C K	M E T H	N O F	P R O C N O.	C A L B L O C K N O.	R E P T N O.	C A L R E P T N O.	I N P	D A T E	C O M M E N T S	R E S		
01	UHI	UPIF00000368C	0	B62	FB	ISI0001	01	S	08.00	0.706	VT	NUT	01	R1159	LDK	09/17/83	L-UC	IEB#82-02	A	
01	UHI	UPIF00000538C	0	B62	FB	ISI0001	02	S	08.00	0.706	PT	NPT	07	R1158	LDK	09/15/83	L-UC	IEB#82-02	A	
											VT	NUT	01	R1159	LDK	09/15/83	L-UC	IEB#82-02	A	
01	UHI	UPIF00000550C	0	B62	FB	ISI0001	02	S	08.00	0.706	PT	NPT	07	R1157	LDK	09/15/83	L-UC	IEB#82-02	A	
											VT	NUT	01	R1160	LDK	09/15/83	L-UC	IEB#82-02	A	
01	UHI	UPIF00000308C	0	B62	FB	ISI0001	02	S	08.00	0.706	PT	NPT	07	R1189	LDK	09/15/83	L-UC	IEB#82-02	A	
											VT	NUT	01	P1161	LDK	09/15/83	L-UC	IEB#82-02	A	
01	UHI	UPIF00000638C	0	B62	FB	ISI0001	02	S	08.00	0.706	PT	NPT	07	R1170	LDK	09/15/83	L-UC	IEB#82-02	A	
											VT	NUT	01	R1162	LDK	09/15/83	L-UC	IEB#82-02	A	
01	UHI	UPI5000000018B	0	BJ	BW	ISI0001	01	S	02.00	0.343	PT	NPT	04	R0072	LEW	07/25/83	L-RW		A	
01	UHI	UPI5000000019	0	BJ	BW	ISI0001	01	S	12.00	1.125	PT	NPT	04	R0275	LEW	07/25/83	L-RW		A	
											UT45	SQUT27	SQ	03	R0335	C0004	EWS	03/05/83	L-RW	A
01	UHI	UPI5000000020	0	BJ	BW	ISI0001	01	S	12.00	1.125	PT	NPT	04	R0172	LEW	07/27/83	L-RW		A	
											UT45	SQUT27	SQ	03	R0336	C0004	EWS	08/05/83	L-RW	A
01	UHI	UPI5000000023	0	BJ	BW	ISI0001	01	S	12.00	1.125	PT	NPT	04	R0430	MSE	09/11/83	L-LC		A	
											UT45	SQUT27	SQ	03	R0562	C0027	FSW	09/21/83	L-LC	A
01	UHI	UPI5000000051	0	BJ	BW	ISI0001	02	S	12.00	1.125	PT	NPT	04	R1052	RMB	09/09/83	L-RW		A	
											UT45	SQUT27	SQ	02	R1066	C0059	MOR	09/13/83	L-RW	A
01	UHI	UPI5000000053	0	BJ	BW	ISI0001	02	S	12.00	1.125	PT	NPT	04	R0174	LEW	07/27/83	L-RW		A	
											UT45	SQUT27	SQ	03	R0339	C0304	EWS	08/05/83	L-RW	A
01	UHI	UPI5000000054	0	BJ	BW	ISI0001	02	S	12.00	1.125	PT	NPT	04	R0254	MSE	08/03/83	L-LC		A	
											UT45	SQUT27	SQ	03	R0340	C0004	EWS	03/05/83	L-LC	A
01	UHI	UPI5000000055	0	BJ	BW	ISI0001	02	S	12.00	1.125	PT	NPT	04	R0589	MSE	09/19/83	L-LC		A	
											UT45	SQUT27	SQ	03	R0651	C0027	FSW	03/21/83	L-LC	A
01	UHI	UPI3000001272	0	BJ	BW	ISI0001	01	S	02.00	0.343	PT	NPT	04	R0074	LEW	07/25/83	L-RW		A	
01	UHI	UPIF00000318C	0	B62	FB	ISI0001	01	S	08.00	0.706	PT	NPT	07	R1165	LDK	09/17/83	L-UC	IEB#82-02	A	
											VT	NUT	01	R1157	LDK	09/17/83	L-UC	IEB#82-02	A	
01	UHI	87-0000000553	0	B62	VB	ISI0001	01	S	08.00	0.706	PT	NPT	07	R0318	JEF	09/29/83	L-UC	IEB#82-02	A	
											VT	NUT	01	R0319	JEF	08/23/83	L-UC	IEB#82-02	A	
01	UHI	87-0000000559	0	B62	VB	ISI0001	01	S	08.00	0.706	VT	NUT	01	R1151	LDK	09/17/83	L-UC		A	
01	UHI	87-0000000560	0	B62	VB	ISI0001	02	S	08.00	0.706	VT	NUT	01	R1152	LDK	09/17/83	L-UC		A	

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Owner: Tennessee Valley Authority  
Division of Nuclear Power  
1750 Chestnut Street Tower II  
Chattanooga, Tennessee 37401

Plant Unit: 2

Owner Certificate of  
Authorization: Not Required

Plant: Sequoyah Nuclear Plant  
P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
1 June 1982

National Board Number for Unit:  
No Number Assigned

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The unit 2, cycle 1 in-service examination at Sequoyah Nuclear Plant included a total of 87 "Notification of Indication" (NOI) reports during the duration of examination. All NOIs were dispositioned in accordance with SI-114.2, section 17.0 and satisfactorily reinspected with the exception of 15 examinations which were dispositioned with an Unreviewed Safety Question Determination (USQD). Components were reviewed by Field Services, Mechanical Engineering Group to use in an "as is" condition. USQDs are made as an attachment to applicable NOIs.

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Owner: Tennessee Valley Authority  
 Division of Nuclear Power  
 1750 Chestnut Street Tower II  
 Chattanooga, Tennessee 37401

Plant Unit: 2

Owner Certificate of  
 Authorization: Not Required

Plant: Sequoyah Nuclear Plant  
 P. O. Box 2000  
 Daisy, Tennessee

Commercial Service Date:  
 1 June 1982

National Board Number for Unit:  
 No Number Assigned

PART

SEQUOYAH NUCLEAR PLANT

UNIT 2, CYCLE 1 - SUMMARY OF INDICATIONS

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Indication	Piping	Supports	Vessels	Pumps	Total
Arc Strike		2			2
Bolt Engagement		1			1
Broken Parts		1			1
Cleanliness	9	1	4		4
Defective Bolting	1		5	5	11
Hydraulic Fluid Low/Leaking		4			4
Inconsistent Bolting	1	1			2
Linear Indication	1				1
Loose Parts		33			33
Missing Parts		6			6
Out-of-Range Settings		10			10
Rounded Indications	1				1
Weld Separation		1			1
Total	13	60			87
Percent	15%	69%	10%	6%	100%

Owner: Tennessee Valley Authority  
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Plant Unit: 2

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Plant: Sequoyah Nuclear Plant  
P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
1 June 1982

National Board Number for Unit:  
No Number Assigned

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SUMMARY OF NOTIFICATIONS

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Notification Report No.	Discrepancy	Disposition Complete
SQ-001	Hanger 2-RHRH-408 R-006 Missing Cotter Pin	MR-A-037886 R-0698 for Reinspection
SQ-002	Hanger 2-CSH-440 R-0022 Loose Nut	MR-037886 R-0697 for Reinspection
SQ-003	Hanger 2-RHRH-451 R-0031 Four Loose Locknuts	MR-A-037886 R-0745 for Reinspection
SQ-004	Hanger 2-RHRH-452 R-0032 Loose Nut, Broken Spherical Bearing	MR-A-112055 R-1042 for Reinspection
SQ-005	Hanger 2-MSH-390 R-0033 Loose Nut	MR-A-037886 R-0700 for Reinspection
SQ-006	Hanger 2-FDH-245, Missing Cotter Pin, R-0053	MR-A-037886 R-0701 f Reinspection
SQ-007	Weld 2-CVC-1045, Fourteen Rounded Indications R-0077	MR-A-112052 R-0527 for Reinspection
SQ-008	Weld 2-CVC-1253 R-0078 One-Inch Linear Indication	MR-A-112053 R-0528 for Reinspection
SQ-009	Hanger 2-MSH-308 R-0092 Five Loose Nuts	MR-A-037886 R-0699 for Reinspection

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Owner: Tennessee Valley Authority  
Division of Nuclear Power  
1750 Chestnut Street Tower II  
Chattanooga, Tennessee 37401

Plant Unit: 2

Owner Certificate of  
Authorization: Not Required

Plant: Sequoyah Nuclear Plant  
P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
1 June 1982

National Board Number for Unit:  
No Number Assigned

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SUMMARY OF NOTIFICATIONS

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Notification Report No.	Discrepancy	Disposition Complete
SQ-0010	Hanger 2-MSH-429 R-0093 Bolts not Carrying Weight of Plate	MR-A-037886 R-1203 for Reinspection
SQ-0011	Hanger 2-MSH-310 R-0094 No Fluid in Sight Glass	SI-162-2 R-1229 for Reinspection
SQ-0012	Hanger 2-MSH-311 R-0095 Loose Locknut	MR-A-037886 R-1050 for Reinspection
SQ-0013	Hanger 2-MSH-315 R-0096 Loose Locknut	MR-A-037886 R-1049 for Reinspection
SQ-0014	Hanger 2-MSH-315 R-0097 Loose Locknut	MR-A-037886 R-1048 for Reinspection
SQ-0015	Hanger 2-MSH-431 R-0098 Loose Locknut Cotter Pin Missing 5/16" Space on Base Plate	MR-A-037886 R-1202 for Reinspection
SQ-0016	Hanger 2-CVCH-126 R-0179 Arc Strike	MR-A-112051 R-0529 for Reinspection
SQ-0017	Hanger 2-SIH-31 R-0184 Loose Nut	MR-A-037886 R-1047 for Reinspection

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Owner: Tennessee Valley Authority  
Division of Nuclear Power  
1750 Chestnut Street Tower II  
Chattanooga, Tennessee 37401

Plant Unit: 2

Owner Certificate of  
Authorization: Not Required

Plant: Sequoyah Nuclear Plant  
P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
1 June 1982

National Board Number for Unit:  
No Number Assigned

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SUMMARY OF NOTIFICATIONS

---

Notification Report No.	Discrepancy	Disposition Complete
SQ-0018	Hanger 2-RHRH-423 R-0195 Loose Locknut	MR-A-037886 R-0773 for Reinspection
SQ-0019	Bolting (Steam Generator Nos. 2 and 3), R-0210, Ten Unacceptable Bolts	MR-A-111817 R-0242 for Replacement Bolting
SQ-0020	Bolting (Steam Generator Nos. 2 and 3), R-0211, 1-MT Indication	MR-A-111817 R-0241 for Replacement Bolt
SQ-0021	Hanger 2-SIH-165 R-0239 Arc Strikes	MR-A-112056 R-1176 for Reinspection
SQ-0022	Hanger 2-SIH-55 R-0240 Rust on Welds and Loose Nuts	MR-A-037886 R-1051 for Reinspection
SQ-0023	Hanger 2-RHRPH-2A-A R-0282 Bolts Missing	MR-A-037886 R-1214 for Reinspection
SQ-0024	Hanger 2-RHRH-404 R-0218 Inconsistent Washers	MR-A-037886 R-1209 for Reinspection
SQ-0025	Hanger 2-SIH-219 R-0308 Weld Separated	MR-A-112057 R-1177, See Reinspection Report

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Owner: Tennessee Valley Authority  
Division of Nuclear Power  
1750 Chestnut Street Tower II  
Chattanooga, Tennessee 37401

Plant Unit: 2

Owner Certificate of  
Authorization: Not Required

Plant: Sequoyah Nuclear Plant  
P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
1 June 1982

National Board Number for Unit:  
No Number Assigned

SUMMARY OF NOTIFICATIONS

Notification Report No.	Discrepancy	Disposition Complete
SQ-0026	Hanger 2-SIH-59 R-0309 Loose Wall Anchor	MR-A-037886 R-1232, See Reinspection Report
SQ-0027	Seal Assembly No. 1, R-0316, R.C.P. No. 3	No Reinspection Performed N.O.I Dispositioned to Use USQD - 83-25 "as is", See R-0316
SQ-0028	Seal Assembly No. 2, R-0317, R.C.P. No. 3	MR-A-086443 R-0329 for Reinspection
SQ-0029	Seal Assembly No. 3, R-0318, R.C.P. No. 3	MR-A-086443 R-0328 for Reinspection
SQ-0030	Steam Generator No. 4, Hot Leg Manway Cover, R-0358 Boron Leakage	MR-A-111820 R-1057, See New NOI-SQ-0075
SQ-0031	Steam Generator No. 1, Hot Leg Manway Cover, R-0359 Boron Leakage	MR-A-111819 R-1053, See New NOI SQ-0074
SQ-0032	Seal Assembly No. 2, R-0407, R.C.P. No. 1	MR-A-086441 R-0410 for Reinspection
SQ-0033	Seal Assembly No. 3, R-0408, R.C.P. No. 1	MR-A-086441 R-0409 for Reinspection

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Owner: Tennessee Valley Authority  
Division of Nuclear Power  
1750 Chestnut Street Tower II  
Chattanooga, Tennessee 37401

Plant Unit: 2

Owner Certificate of  
Authorization: Not Required

Plant: Sequoyah Nuclear Plant  
P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
1 June 1982

National Board Number for Unit:  
No Number Assigned

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SUMMARY OF NOTIFICATIONS

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Notification Report No.	Discrepancy	Disposition Complete
SQ-0034	Hanger 2-MSH-353 R-0434 Loose Bolt	MR-A-037886 R-1144 for Reinspection
SQ-0035	Hanger 2-FDH-322 R-0448 Loose Bolt	Mh A-037886 R-0868 for Reinspection
SQ-0036	Valve Bolting, 63-622 R-0453 Corroded Washers	No Reinspection, Use "As Is", See USQD-83-26
SQ-0037	Hanger 2-FDH-240 R-0467 Loose Nut	MR-A-037886 R-0867 for Reinspection
SQ-0038	Hanger 2-MSH-342 R-0476 Loose Nut	MR-A-037886 R-1130 for Reinspection
SQ-0039	Hanger 2-MSH-343 R-0477 No Fluid in Sight Glass	MR-A-037039 R-1210 for Reinspection
SQ-0040	Hanger 2-MSH-421 R-0481 Locking Ring Loose	MR-A-037886 R-1133 for Reinspection
SQ-0041	Hanger 2-MSH-423 R-0482 Locking Ring Loose	MR-A-037886 R-1128 for Reinspection
SQ-0042	Hanger 2-MSH-423 R-0483 Evidence of Leakage	MR-A-037886 R-1134 for Reinspection

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Daisy, Tennessee

Commercial Service Date:  
1 June 1982

National Board Number for Unit:  
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Notification Report No.	Discrepancy	Disposition Complete
SQ-0043	Hanger 2-SIH-453 R-0520 No Load Indicated	MR-A-037886 R-1143 for Reinspection
SQ-0044	Flange Bolting RCS-148 BC R-0551, Boron Leakage	MR-A-037886 R-1077 for Reinspection
SQ-0045	Flange Bolting, CVC 1057/1057A R-0552, Boron Leakage	Reinspected on R-1109, See New NOI SQ 0080
SQ-0046	Bolting 62-660 R-0553 Inconsistency Washers	No Reinspection Performed, See USQD 83-27
SQ-0047	Hanger 2-RHRH-447 R-0575 Loose Nut	MR-A-037886 R-1145 for Reinspection
SQ-0048	Hanger 2-RHRH-469 R-0585 Loose Nut	MR-A-037886 R-1175 for Reinspection
SQ-0049	RHR Valve 2-74-1 R-0683 Corrosion on Bolting	MR-A-107364 R-0685 for Reinspection
SQ-0050	Hanger RHXH-1 R-0684 Loose Nuts	MR-A-037886 R-1213 for Reinspection
SQ-0051	RC Valve 68-541 R-0702 Corrosion on Bolting	Re-NOI-SQ-0081 R-1136 Pitting on Bolts

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P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
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National Board Number for Unit:  
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SUMMARY OF NOTIFICATIONS

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Notification Report No.	Discrepancy	Disposition Complete
SQ-0052	RC Valve 68-508 R-0703 Corrosion on Bolting	Re-NOI, SQ-0081 R-1136 Pitting on Bolts
SQ-0053	Hanger 2-SIH-11 R-0791 Stroke Setting	MR-A-037886 RE-NOI, See SQ-0087
SQ-0054	Hanger 2-RHXH-2 R-0822 Loose Nut, Interference	MR-A-037886 R-1212 for Reinspection
SQ-0055	Hanger 2-UH1H-131 R-0858 Loose Nut	No MR Issued, R-1118 for Reinspection
SQ-0056	Hanger 2-UH1H-10 R-0888 Loose Nut	MR-A-037886 R-1114 for Reinspection
SQ-0057	Hanger 2-UH1H-13 R-0891 Loose Nut	MR-A-037886 R-1178 for Reinspection
SQ-0058	Hanger 2-UH1H-35 R-0895 Loose Jam Nut	MR-A-037886 R-115 for Reinspection
SQ-0059	Hanger 2-UH1H-38 R-0897 Loose Nut	MR-A-037886 R-1116 for Reinspection
SQ-0060	Hanger 2-UH1H-72 R-0902 Loose Jam Nut	MR-A-037886 R-1117 for Reinspection

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Plant Unit: 2

Owner Certificate of  
Authorization: Not Required

Plant: Sequoyah Nuclear Plant  
P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
1 June 1982

National Board Number for Unit:  
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SUMMARY OF NOTIFICATIONS

Notification Report No.	Discrepancy	Disposition Complete
SQ-0061	Hanger 2-UH1H-140 R-0903 Missing Cotter Pin	MR-A-037886 R-1119 for Reinspection
SQ-0062	Hanger 2-UH1H-143 R-0906 Loose Jam Nut	MR-A-037886 R-1120 for Reinspection
SQ-0063	Hanger 2-RCH-133 R-0946 Stroke Setting Low	See USQD-83-28, No Reinspection Performed
SQ-0064	Hanger 2-RCH-147 R-0960 Loose Nut on Clamp	MR-A-037866 R-1121 for Reinspection
SQ-0065	Hanger 2-RCH-165 R-0963 Loose Nut on Clamp	MR-A-037886 R-1122 for Reinspection
SQ-0066	Hanger 2-RCH-192 R-0976 Missing Cotter Pin	MR-A-037886 R-1123 for Reinspection
SQ-0067	Hanger 2-RCH-240 R-0989 Loose Nut	MR-A-37886 R-1124 for Reinspection
SQ-0068	Hanger 2-RCH-250 R-0993 Loose Nut on Pipe Clamp	MR-A-037886 R-1125 for Reinspection
SQ-0069	Hanger 2-RCH-251 R-0994 No Load on Hanger and Loose Nuts	MR-A-037886 R-1226 for Reinspection

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Chattanooga, Tennessee 37401

Plant Unit: 2

Owner Certificate of  
Authorization: Not Required

Plant: Sequoyah Nuclear Plant  
P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
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National Board Number for Unit:  
No Number Assigned

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SUMMARY OF NOTIFICATIONS

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Notification Report No.	Discrepancy	Disposition Complete
SQ-0070	Hanger 2-RCH-894 R-1034 No Load on Spring Hanger	No MR issued, R-1146 for Reinspection
SQ-0071	Hanger 2-RCH-920 R-1036 Loose Jam Nut	MR-A-037886 R-1126 for Reinspection
SQ-0072	Lower Reactor Vessel Internals, R-1043, Paint Chips	No Reinspection Performed, See USQD 83-24
SQ-0073	Reactor Vessel Upper Internals, R-1044, Paint Flakes	No Reinspection Performed, See USQD-83-24
SQ-0074	Steam Generator No. 1, Manway Bolting and Bolts Rejected, R-1053	MR-A-111819 R-1054, See USQD-83-25
SQ-0075	Steam Generator No. 4, Manway Bolting, Six Bolts Rejected, R-1057	MR-111820 R-1059, See USQD-83-25
SQ-0076	Steam Generator No. 4, Manway Bolting, R-1058, Linear Indication	MR-A-111820 R-1060 for Reinspection
SQ-0077	RPV No. RVI-80-10-A Bonnet Bolting, R-1062, Axial Indication	MR-A-04-7223 R-1063 for Reinspection

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Plant Unit: 2

Owner Certificate of  
Authorization: Not Required

Plant: Sequoyah Nuclear Plant  
P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
1 June 1982

National Board Number for Unit:  
No Number Assigned

SUMMARY OF NOTIFICATIONS

Notification Report No.	Discrepancy	Disposition Complete
SQ-0076	Hanger 2-RHRH-7 R-1107 Hydraulic Leak	MR-A-037886 R-1232 for Reinspection
SQ-0079	Valve FCV-74-2 R-1108 Rust and Boron on Bolting	MR-A-037886 R-1215 for Reinspection
SQ-0080	Bolting, CVC-1057/1057A R-1109, Corrosion and Pitting	No Reinspection Performed, See USQD 83-26
SQ-0081	R.C. Valve 68-541 R-1136 Rust and Pitting on Bolting	No Reinspection Performed, See USQD-83-26
SQ-0082	Valve 68-508 R-1137 Pitting on Bolting	MR-A-112108 R-1208 for Reinspection
SQ-0083	Hanger 2-UH1H-152 R-1195 Stroke Setting	No Reinspection Performed, See USQD 83-28
SQ-0084	Hanger 2-UH1H-153 R-1196 Stroke Setting	No Reinspection Performed, See USQD 83-28
SQ-0085	Hanger 2-UH1H-18 R-1197 Stroke Setting Out of Tolerance	No Reinspection Performed, See USQD-83-28

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Owner: Tennessee Valley Authority  
Division of Nuclear Power  
1750 Chestnut Street Tower II  
Chattanooga, Tennessee 37401

Plant Unit: 2

Owner Certificate of  
Authorization: Not Required

Plant: Sequoyah Nuclear Plant  
P. O. Box 2000  
Daisy, Tennessee

Commercial Service Date:  
1 June 1982

National Board Number for Unit:  
No Number Assigned

SUMMARY OF NOTIFICATIONS

Notification Report No.	Discrepancy	Disposition Complete
SQ-0086	Hanger 2-UH1H-46 R-1198 Stroke Setting Out of Tolerance	No Reinspection Performed, See USQD-83-28
SQ-0087	Hanger 2-SIH-11 R-1207 Stroke Setting Out of Tolerance	No Reinspection Performed, See USQD 83-28

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APPENDIX H  
SUPPORT LOAD CRITERIA

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Type  
 VS-Variable Spring  
 RS-Rigid Support  
 MS-Mechanical Snubber  
Model  
 RS-Rigid Strut  
 RH-Rigid Hanger

Plant: SQNP Unit: 2

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IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S				HOT	COLD		
	2-AFDH-200	3	RS	RH				
	2-AFDH-244	3	RH	BE 415 RS Size 1				
	2-AFDH-245	3	VS	BE 401 Type A Size 8	668#	687#	1/16" Up	5/16"-1"
	2-AFDH-246	3	VS	BD 401 Type G Size 5	311#	319#	3/32" Up	11/32"-1"
	2-AFDH-247	3	RS	BE 415 RS Size 1				
	2-AFDH-248	3	RS	RH				
	2-AFDH-249	3	RS	RH				
	2-AFDH-250	3	RS	RH				
	2-AFDH-251	3	RS	RH				
	2-AFDH-252	3	VS	BE 401 Type G Size 7	531#	595#	3/32" Up	11/32"-1"
	2-AFDH-295	3	RS	RH				

IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S				HOT	COLD		
	2-AFDH-305	3	VS	BE 401 Type B Size 11	1559#	1594#	1/16" Up	5/16"-1"
	2-AFDH-306	3	RS	RH				
	2-AFDH-307	3	RS	RH				
	2-AFDH-308	3	RS	RH				
	2-AFDH-309	3	VS	BE 401 Type B Size 2	144#	148#	1/32" Up	9/32"-1"
	2-AFDH-310	3	RS	RH				
	2-AFDH-311	3	RS	RH				
	2-AFDH-312	3	VS	BE 402 Type F Size 9	1058#	1058#	Slight*	1/4"-2 1/4"***
	2-AFDH-313	3	RS	BE 415 RS Size 1				
	2-AFDH-331	3	RS	RH				
	2-AFDH-342	3	VS	BE 401 Type B Size 10	1222#	1255#	1/16" Up	5/16"-1"
	2-AFDH-343	3	RS	RH				
	2-AFDH-344	3	RS	BE 415 RS Size 2				

\*Thermal movement as stated by EN DES on BE 4180-2AFD sheet H3-31E as constructed.

\*\*\*No thermal movement accounted for.

IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-AFDH-345	3	RS	RH					
	2-AFDH-346	3	RS	BE 415 RS Size 1					
	2-AFDH-347	3	RS	RH					
	2-AFDH-348	3	RS	RH					
	2-AFDH-349	3	VS	BE 401 Type F Size 9		1055#	1055#	Slight*	1/4"-1"***
	2-AFDH-350	3	RS	BE 415 RS Size 1					
	1-AFDH-227	3	RS	RH					
	1-AFDH-229	3	MS	BE 417 PSA-3	5"	1'-5 5/8"	1'-5 7/8"	1/4"	1/2"-4 3/4"
	1-AFDH-242	3	RS	RH					
	1-AFDH-250	3	VS	GR SPRG Fig. 82 Type F Size 5		350#	350#	0	1/4"-1"
	1-AFDH-251	3	VS	GR SPRG Fig. 82 Type F Size 3		129#	129#	0	1/4"-1"

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\*Thermal movement stated by EN DES.  
\*\*No thermal movement accounted for.

IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S				HOT	COLD		
	1-AFDH-252	3	RS	BE 415 Size 2 RS				
	1-AFDH-253	3	RS	GR SPRG Fig. 82 Type F Size 5	293#	293#	0	1/4"-1"
	1-AFDH-254	3	RS	BE 415 Size 2 RS				

IDENTIFIER		SAFETY CLASS	SUPPORT			DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT	COLD		
	ICVCMH-439	3	RS	RH					
	ICVCMH-440	3	RS	RH					
	ICVCMH-441	3	RS	RH					
	ICVCMH-442	3	RS	RH					
	ICVCMH-455	3	RS	RH					
	ICVCMH-457	3	RS	RH					



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Type  
 RS-Rigid Support  
 VS-Variable Spring  
 MS-Mechanical Snubber  
 IA-Integrally Welded Attachment  
 Model  
 RS-Rigid Strut  
 RH-Rigid Hanger

Plant: SQNP Unit: 2

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-CSH-416	3	RS	RH					
	2-CSH-416-A	3	MS	BE 417 PSA 3	5"	2'-7 7/8"	2'-7 3/4"	1/8"	1/4"-4 5/8"
	2-CSH-417	3	RS	RS BE 415 Size 1					
	2-CSH-418	3	RS	RS BE 415 Size 4					
	2-CSH-419	3	RS	RH					
	2-CSH-420	3	RS	Id					
	2-CSH-421	3	RS	RH					
	2-CSH-422	3	RS	RH					
	2-CSH-423	3	RS	RH					
	2-CSH-424	3	FS	RH					
	2-CSH-425	3	RS	RH					
	2-CSH-426	3	RS	RH					

IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-CSH-427	3	RS	RH					
	2-CSH-428	3	RS	RH					
	2-CSH-429	3	RS	RH					
	2-CSH-430	3	RS	RH					
	2-CSH-431	3	RS	RH					
	2-CSH-432	3	R	RH					
	2-CSH-433	3	RS	RH					
	2-CSH-434	3	RS	RH					
	2-CSH-435	3	RS	RH					
	2-CSH-436	3	RS	RH					
	2-CSH-437	3	RS	RH					
	2-CSH-438	3	RS	RH					
	2-CSH-439	3	RS	Size 1 RS BE 415					
	2-CSH-451	3	RS	RH					
	2-CSH-452	3	RS	BE 415 RS Size 1					

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Type  
 RS-Rigid Support  
 VS-Variable Spring  
 MS-Mechanical Snubber  
 IA-Integrally Welded Attachment  
 HS-Hydraulic Snubber  
 Model  
 RS-Rigid Strut  
 RH-Rigid Hanger

Plant: SQNP Unit: 2

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IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	SUPPORT MODEL	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S					HOT	COLD		
	2-CCH-4-158	3	RS	RH					
	1-CCH-400	3	VS	BE Sprg. Fig. 401 Type B Size 15		5604#	5745#	1/16" Up	1/4"-15/16"
	1-CCH-401	3	RS	RH					
	1-CCH-402	3	HS	BE 411 Size 1 1/2	5"	1'-8.19"	1'-8 1/2"	31"	1.185"-5 3/8"
	1-CCH-403	3	MS	BE 417 PSA 10	6"	4'-9 7/8"	4'-9 3/4"	1/8"	1/4"-5 5/8"
	1-CCH-404	3	VS	Grinnell Sprg. Fig 82 "F" Size 12		1994#	1882#	1/8" Down	1/4"-7/8"
	1-CCH-405	3	VS	BE Sprg. Fig 401 Type B Size 14		3943#	3837#	1/16" Down	1/4"-15/16"

Support identifiers 1-CCH-400 through 1-CCH-769 are supports to be examined with the Unit 2 ISI sample.

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-CCH-413	3	VS	Gr. Sprg. Fig 82 Type B Size 13		3000#	2925#	1/16" Down	1/4"-5/16"
	1-CCH-414	3	VS	Gr. Sprg. Fig 82 Type B Size 13		2571#	2421#	1/8" Down	1/4"-7/8"
	1-CCH-415	3	HS	BE 411 Size 2½	5"	3 7/16"	3 1/2"	1/16"	1 5/16"-5 3/4"
	1-CCH-416	3	VS	Gr. Sprg. Fig 82 Type B Size 16		8327#	8139#	1/16" Down	1/4"-15/16"
	1-CCH-420	3	RS	RH					
	1-CCH-421	3	HS	BE 411 Size 3¼"	5"	3 5/16"	3 9/16"	1/4"	1 5/8"-5 7/8"
	1-CCH-422	3	RS	RH					
	1-CCH-423	3	RS	RH					
	1-CCH-424	3	RS	RH					
	1-CCH-425	3	RS	RH					
	1-CCH-426	3	RS	RH					
	1-CCH-428	3	RS	RH					
	1-CCH-454	3	RS	RH					
	1-CCH-455	3	RS	RH					
	1-CCH-458	3	RS	RH					

IDENTIFIER		SAFETY CLASS	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT		
	1-CCH-459	3	RS	RH				
	*1-CCH-476	3	RS					
	1-CCH-482	3	RS	RH				
	1-CCH-483	3	RS	RS				
	1-CCH-489	3	RS	RH				
	1-CCH-490	3	RS	RH				
	1-CCH-491	3	RS	RH				
	1-CCH-492	3	RS	RH				
	1-CCH-492A	3	RS	RH				
	1-CCH-493	3	RS	RH				
	1-CCH-494	3	VS	BE Sprg. Fig 401 Type B Size 15	5124#	4983#	1/16" Down	1/4"-15/16"
	1-CCH-495	3	RS	RH				
	1-CCH-496	3	RS	RH				
	1-CCH-545	3	RS	RH				
	1-CCH-545A	3	RS	RH				
	1-CCH-546	3	RS	RH				
	1-CCH-547	3	RS	RH				
	1-CCH-548	3	RS	RH				

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-CCH-549	3	RS	RH					
	1-CCH-550	3	RS	RH					
	1-CCH-551	3	RS	RH					
	1-CCH-552	3	RS	RH					
	1-CCH-553	3	RS	RH					
	1-CCH-554	3	RS	RH					
	1-CCH-555	3	RS	RH					
	1-CCH-556	3	RS	RH					
	1-CCH-557	3	RS	RH					
	1-CCH-557A	3	RS	BE 415 RS Size 1					
	1-CCH-558	3	RS	RH					
	1-CCH-574	3	RS	RH					
	1-CCH-576	3	RS	RH					
	1-CCH-577	3	RS	RH					
	1-CCH-578	3	RS	Fig 415 RS Size 4					
	1-CCH-579	3	RS	BE 415 RS Size 7					

IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-CCH-580	3	RS	RH					
	1-CCH-581	3	RS	RH					
	1-CCH-582	3	RS	RH					
	1-CCH-583	3	VS	Gr. Sprg. Fig 82 Type F Size 12	2017#	2130#	1/8" Up	3/8"-1"	
	1-CCH-584	3	RS	RH					
	1-CCH-585	3	RS	RH					
	1-CCH-586	3	RS	BE 415 RS Size 1					
	1-CCH-587	3	RS	RH					
	1-CCH-588	3	RS	RH					
	1-CCH-589	3	RS	RH					
	1-CCH-590	3	RS	RH					
	1-CCH-591	3	RS	RH					
	1-CCH-592	3	RS	RH					
	1-CCH-593	3	RS	RH					
	1-CCH-594	3	RS	RH					
	1-CCH-595	3	RS	BE 415 RS Size 3					

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IDENTIFIER		SAFETY CLASS	SUPPORT			DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT	COLD		
	1-CCH-596	3	RS	RH					
	1-CCH-597	3	RS	RH					
	1-CCH-598	3	RS	RH					
	1-CCH-599	3	RS	RH					
	1-CCH-600	3	RS	Gr. Sprg Fig 82 Type F Size 12		2144#	2234#	.10 Down .250"-.9"	
	1-CCH-601	3	RS	BE 415 RS Size 3					
	1-CCH-602	3	RS	RH					
	1-CCH-603	3	RS	RH					
	1-CCH-604	3	RS	BE 415 RS Size 1					
	1-CCH-605	3	RS	RH					
	1-CCH-606	3	RS	RH					
	1-CCH-607	3	RS	RH					
	1-CCH-608	3	RS	RH					
	1-CCH-609	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT		
	1-CCH-609A	3	VS	Gr. Sprg. Fig 82 Type F Size 7		587#	601#	1/16" Up 5/16"-1"
	1-CCH-648	3	RS	RH				
	1-CCH-649	3	RS	RH				
	1-CCH-650	3	RS	RH				
	1-CCH-651	3	RS	RH				
	1-CCH-652	3	RS	RH				
	1-CCH-653	3	RS	RH				
	1-CCH-653A	3	RS	RH				
	1-CCH-654	3	RS	BE 415 RS Size 2				
	1-CCH-655	3	RS	RH				
	1-CCH-656	3	RS	RH				
	1-CCH-657	3	RS	RH				
	1-CCH-658	3	RS	BE 415 RS Size 1				
	1-CCH-663	3	RS	RH				
	1-CCH-664	3	RS	RH				

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IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S				HOT	COLD		
	1-CCH-665	3	RS	RH				
	1-CCH-677	3	RS	BE 415 RS Size 3				
	1-CCH-678	3	VS	Gr. Sprg. Fig 82 Type B Size 14	4153#	4253#	1/16" Up	5/16"-1"
	1-CCH-679	3	RS	RH				
	1-CCH-700	3	RS	RH				
	1-CCH-701	3	RS	RH				
	1-CCH-702	3	RS	RH				
	1-CCH-703	3	RS	RH				
	1-CCH-707	3	RS	RH				
	1-CCH-711	3	RS	RH				
	1-CCH-712	3	RS	BE 415 RS Size 1				
	1-CCH-713	3	RS	BE 415 RS Size 2				
	1-CCH-714	3	RS	RH				
	1-CCH-715	3	RS	RH				

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IDENTIFIER		SAFETY CLASS	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT		
	1-CCH-716	3	RS	RH				
	1-CCH-717	3	RS	RH				
	1-CCH-718	3	RS	RH				
	1-CCH-719	3	VS	Gr. Sprg. Fig 82 Type F Size 12		2007#	2106#	.11 Up .36"-1"
	1-CCH-720	3	MS	BE 416 PAS 10	6"	1'-11 7/16"	1'-11 3/8"	1/16" 1/4"-5 11/16"
	1-CCH-721	3	RS	RH				
	1-CCH-722	3	RS	BE 415 RS Size 1				
	1-CCH-723	3	RS	RH				
	1-CCH-724	3	RS	RH				
	1-CCH-725	3	VS	Gr. Sprg. Fig 82 Type F Size 12		2076#	2189#	1/8" Up 3/8"-1"
	1-CCH-726	3	RS	RH				
	1-CCH-727	3	RS	BE 415 RS Size 1				
	1-CCH-728	3	MS	BE 416 PSA 3	5"			1/16" Compression 5/16"-4.75"

IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-CCH-729	3	RS	RH					
	1-CCH-730	3	RS	RH					
	1-CCH-731	3	RS	BE 415 RS Size 3					
	1-CCH-732	3	RS	RH					
	1-CCH-733	3	MS	BE 416 PSA 1	4"	1'-3 5/8"	1'-3 5/8"	.06	1/4"-3 3/4"
	1-CCH-734	3	VS	Gr. Sprg. Fig 82 Type B Size 8		781#	799#	1/16" Up	5/16"-1"
	1-CCH-735	3	RS	RH					
	1-CCH-736	3	RS	BE 415 RS Size 2					
	1-CCH-737	3	RS	BE 415 RS Size 2					
	1-CCH-738	3	RS	BE 415 RS Size 1					
	1-CCH-739	3	RS	BE 415 RS Size 1					
	1-CCH-740	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-CCH-741	3	RS	RH					
	1-CCH-742	3	RS	RH					
	1-CCH-743	3	RS	RH					
	1-CCH-744	3	RS	RH					
	1-CCH-745	3	RS	RH					
	1-CCH-746	3	RS	RH					
	1-CCH-747	3	RS	RH					
	1-CCH-748	3	RS	RH					
	1-CCH-749	3	RS	RH					
	1-CCH-750	3	RS	RH					
	1-CCH-751	3	RS	RH					
	1-CCH-752	3	RS	RH					
	1-CCH-753	3	MS	BE 416 PSA 10	6"	2'-0 7/16"	2'-0 5/8"	3/16"	7/16"-5 3/4"
	1-CCH-754	3	RS	RH					
	1-CCH-755	3	RS	RH					
	1-CCH-756	3	RS	RH					
	1-CCH-757	3	RS	RH					
	1-CCH-758	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE	
TVA'S	ISI'S				HOT	COLD			
	1-CCH-760	3	MS	BE 416 Size PSA 3	5"	1'-4 7/8"	1'-5 3/16"	5/16"	9/16"-4 3/4"
	1-CCH-761	3	RS	BE 415 RS Size 1					
	1-CCH-762	3	MS	BE 417 PSA 10	6"	1'-7 13/16"	1'-8 8/3"	9/16"	13/16"-5 3/4"
	1-CCH-763	3	VS	BE 402 Type A Size 9		924#	1016#	1/4" Up	1/2"-2 1/4"
	1-CCH-764	3	MS	BE 417 PSA 1	4"	3'-0 1/8"	3'-0 1/16"	1/16"	1/4"-3 11/16"
	1-CCH-765	3	VS	Gr. Sprg. Fig 82 Type C Size 8		701#	767#	1/4" Up	1/2"-1"
	1-CCH-766	3	VS	Gr. Sprg. Fig 82 Type B Size 5		314#	361#	3/8" Up	5/8"-1"
	1-CCH-767	3	VS	Gr. Sprg. Fig 82 Type B Size 9		955#	1047#	1/4" Up	1/2"-1"
	1-CCH-768	3	MS	BE 417 PSA 3	5"	7'-1 3/4"	7'-2"	1/4"	1/2"-4 3/4"
	1-CCH-769	3	MS	PSA 1	4"	1'-7/8"	1'-1"	1/8"	3/8"-3 3/4"

IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-CCH-60	3	VS	Gr. Sprg. Fig 268 Type F Size 5		271#	283#	1/4" Up	1/2"-2 1/4"
	2-CCH-61	3	RS	RH					
	2-CCH-62	3	MS	PSA 1	4"	13 1/2"	13"	1/2"	1/4"-3 1/4"
	2-CCH-63	3	RS	RH					
	2-CCH-64	3	RS	RH					
	2-CCH-65	3	RS	RH					
	2-CCH-66	3	RS	RH					
	2-CCH-67	3	RS	RH					
	2-CCH-68	3	RS	RH					
	2-CCH-69	3	RS	RH					
	2-CCH-70	3	RS	RH					
	2-CCH-71	3	RS	RH					
	2-CCH-72	3	RS	RH					
	2-CCH-73	3	RS	RH					
	2-CCH-74	3	RS	RH					
	2-CCH-75	3	RS	RH					
	2-CCH-76	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-CCH-77	3	RS	RH					
	2-CCH-78	3	RS	RH					
	2-CCH-800	3	RS	RH					
	2-CCH-801	3	RS	RH					
	2-CCH-802	3	RS	RH					
	2-CCH-803	3	RS	RH					
	2-CCH-805	3	RS	RH					
	2-CCH-806	3	RS	RH					
	2-CCH-807	3	RS	RH					
	2-CCH-808	3	RS	RH					
	2-CCH-809	3	RS	RH					
	2-CCH-810	3	RS	RH					
	2-CCH-811	3	RS	RH					
	2-CCH-812	3	RS	RH					
	2-CCH-813	3	RS	RH					
	2-CCH-814	3	RS	RH					
	2-CCH-815	3	RS	RH					
	2-CCH-816	3	RS	RH					
	2-CCH-817	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	SUPPORT			DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT	COLD		
	2-CCH-844		MS	BE 416 PSA 3	5"	20 1/2"	20 7/8"	3/8"	5/8"-4 3/4"
	2-CCH-845		MS	BE 416 PSA 3	5"	1'-9 5/16"	1'-8 7/8"	7/16"	1/4"-4 5/16"

IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	SUPPORT MODEL	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S					HOT	COLD		
	2-ERCWH-5-1	3	RS	RH					
	2-ERCWH-5-2	3	RS	RH					
	2-ERCWH-5-3	3	RS	RH					
	2-ERCWH-5-4	3	RS	RH					
	2-ERCWH-5-18	3	RS	RH					
	2-ERCWH-5-20	3	RS	RH					
	2-ERCWH-5-20A	3	RS	RH					
	2-ERCWH-5-20B	3	RS	RH					
	2-ERCWH-5-20C	3	RS	RH					
	2-ERCWH-5-25A	3	RS	RH					
	2-ERCWH-5-25B	3	RS	RH					
	2-ERCWH-5-26	3	RS	RH					
	2-ERCWH-5-27	3	RS	RH					
	2-ERCWH-5-28	3	RS	RH					
	2-ERCWH-5-29	3	RS	RH					
	2-ERCWH-5-30	3	RS	RH					
	2-ERCWH-5-31	3	RS	RH					
	2-ERCWH-5-32	3	RS	RH					
	2-ERCWH-5-33	3	RS	RH					

IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-ERCWH-5-34	3	RS	RH					
	2-ERCWH-5-39	3	RS	RH					
	2-ERCWH-5-40	3	RS	RH					
	2-ERCWH-2-64	3	RS	RH					
	2-ERCWH-2-65A	3	RS	RH					
	2-ERCWH-2-65B	3	RS	RH					
	2-ERCWH-2-66	3	RS	RH					
	2-ERCWH-2-67	3	RS	RH					
	2-ERCWH-3-39	3	RS	RH					
	2-ERCWH-3-40A	3	RS	RH					
	2-ERCWH-3-40B	3	RS	RH					
	2-ERCWH-3-41	3	RS	RH					
	2-ERCWH-3-42	3	RS	RH					
	2-ERCWH-3-43	3	RS	RH					
	2-ERCWH-3-44	3	RS	RH					
	2-ERCWH-3-46A	3	RS	RH					
	2-ERCWH-3-46B	3	RS	RH					
	2-ERCWH-3-46C	3	RS	RH					
	2-ERCWH-3-57	3	RS	RH					
	2-ERCWH-3-58	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	SUPPORT			DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT	COLD		
	2-ERCWH-3-59A	3	RS	RH					
	2-ERCWH-3-59B	3	RS	RH					
	2-ERCWH-3-59C	3	RS	RH					
	2-ERCWH-3-59D	3	RS	RH					
	2-ERCWH-3-68	3	RS	RH					
	2-ERCWH-3-70	3	RS	RH					
	2-ERCWH-3-76	3	RS	RH					
	2-ERCWH-3-77	3	RS	RH					
	2-ERCWH-4-50	3	RS	RH					
	2-ERCWH-4-51	3	RS	RH					
	2-ERCWH-4-52	3	RS	RH					
	2-ERCWH-5-2A	3	RS	RH					
	2-ERCWH-5-2B	3	RS	RH					
	2-ERCWH-5-3A	3	RS	RH					
	2-ERCWH-5-3B	3	RS	RH					
	2-ERCWH-25-312	3	Hor. MS	PSA 3	5"	NA	17"	.17C .04T	.42"-4.71"
			Ver. MS	PSA 3	5"	NA	17"	.02C	.27"-4 3/4"
	2-ERCWH-25-340	3	Hor. MS	PSA 3	5"	NA	17"	.02C .16T	.27"-4.59"
			Ver. MS	PSA 3	5"	NA	17"	.02C	.27"-4 3/4"

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IDENTIFIER		SAFETY CLASS	SUPPORT			DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT	COLD		
	2-ERCWH-25-368	3	Hor. MS	PSA 1	4"	12 29/32	13"	.02C	.27"-3.59"
			Ver. MS	PSA 3	5"	NA	17"	.16T	.27"-4.75"
								.02C	.27"-4.75"
	2-ERCWH-25-396	3	Hor. MS	PSA 3	5"	NA	17 5/8"	1/16C	5/16"-4 3/4"
			Ver. MS	PSA 3	5"	NA	16 3/4"	1/16C	5/16"-4 3/4"
	2-ERCWH-25-441	3	MS	PSA 3	5"	NA	17"	5/8C	7/8"-4 3/4"
	2-ERCWH-25-442	3	MS	PSA 3	5"	NA	17"	5/16T	1/4"-4 7/16"
	2-ERCWH-25-445	3	MS	FSA 3	5"	NA	17"	.23T	.25"-4.52"
	2-ERCWH-25-446	3	MS	PSA 3	5"	NA	17"	5/16T	1/4"-4 7/16"
	2-ERCWH-2	2	RS	RH					
	2-ERCWH-3	2	RS	RH					
	2-ERCWH-4	2	RS	RH					
	2-ERCWH-5	2	RS	RH					
	2-ERCWH-6	2	RS	RH					
	2-ERCWH-7	2	RS	RH					
	2-ERCWH-8	2	RS	RH					
	2-ERCWH-9	2	RS	RH					
	2-ERCWH-10	2	RS	RH					
	2-ERCWH-11	2	RS	RH					
	2-ERCWH-12	2	RS	RH					
	2-ERCWH-13	2	RS	RH					

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IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S				HOT	COLD		
	2-ERCWH-14	2	RS	RH				
	2-ERCWH-15	2	VS	Gr. Fig 82 Type B Size 6	408#	429#	1/8 Up	3/8"-1"
	2-ERCWH-18	3	RS	RS BE 415 Size 1				
	2-ERCWH-19	3	RS	RH				
	2-ERCWH-20	3	VS	Gr. Fig 82 Type A Size 7	623#	665#	3/16 Up	7/16"-1"
	2-ERCWH-21	3	RS	RH				
	2-ERCWH-22	3	RS	RH				
	2-ERCWH-23	3	RS	RH				
	1-ERCWH-52	3	VS	Gr. Fig 82 Type B Size 13	2288# 2264#	2276#	.01" Up & Dn	.26"-.99"
	1-ERCWH-53	3	RS	RH				
	1-ERCWH-54	3	RS	RH				
	1-ERCWH-55	3	RS	RH				
	1-ERCWH-56	3	RS	RS BE 415 Size 3				

IDENTIFIER		SAFETY CLASS	SUPPORT			DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT	COLD		
	1-ERCWH-57	3	RS	RH					
	1-ERCWH-58	3	MS	PSA 10	6"	1'-0 1/2"	2'-0 3/8"	1/8T	1/4"-5 5/8"
				PSA 10	6"	2'-0 1/2"	2'-0 3/8"	1/8T	1/4"-5 5/8"
	1-ERCWH-59	3	RS	RH					
	1-ERCWH-60	3	RS	RS BE 415 Size 1					
	1-ERCWH-61	3	RS	RH					
	1-ERCWH-62	3	RS	RH					
	1-ERCWH-63	3	RS	RH					
	1-ERCWH-64	3	RS	RH					
	1-ERCWH-65	XS	RH						
	1-ERCWH-66	3	RS	RH					
	1-ERCWH-67	3	RS	RS BE 415 Size 2					
	1-ERCWH-68	3	RS	RH					
	1-ERCWH-69	3	RS	RS BE 415 Size 4					
	1-ERCWH-70	3	RS	RH					
	1-ERCWH-71	3	RS	RH					
	1-ERCWH-72	3	RS	RH					

IDENTIFIER		SAFETY CLASS	SUPPORT			DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT	COLD		
	1-ERCWH-73	3	RS	RH					
	1-ERCWH-74	3	VS	Gr. Fig 82 Type B Size 16		8247#	8153#	1/32" Dn 1/4"-31/32"	
	1-ERCWH-75	3	RS	RH					
	1-ERCWH-76	3	RS	RH					
	1-ERCWH-77	3	RS	RH					
	1-ERCWH-79	3	RS	RH					
	1-ERCWH-80	3	RS	RH					
	1-ERCWH-81	3	RS	RH					
	1-ERCWH-82	3	RS	RH					
	1-ERCWH-83	3	RS	"					
	1-ERCWH-84	3	RS	RH					
	1-ERCWH-121	3	RS	RH					
	1-ERCWH-122	3	RS	RH					
	1-ERCWH-123	3	RS	RH					
	1-ERCWH-124	3	RS	RH					
	1-ERCWH-125	3	RS	RH					
	1-ERCWH-126	3	RS	RH					
	1-ERCWH-127	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-ERCWH-128	3	RS	RS BE 415 Size 6					
	1-ERCWH-129	3	RS	RH					
	1-ERCWH-130	3	RS	RS BE 415 Size 3					
	1-ERCWH-131	3	RS	RH					
	1-ERCWH-132	3	RS	RH					
	1-ERCWH-133	3	RS	RH					
	1-ERCWH-134	3	RS	RH					
	1-ERCWH-135	3	RS	RH					
	1-ERCWH-136	3	VS	Gr. Fig 82 Type B Size 13	3224# 3308#	3272#	.04 Up .03 Dn	.29"-.97"	
	1-ERCWH-137	3	RS	RH					
	1-ERCWH-137A	3	RS	RH					
	1-ERCWH-138	RS	RH						
	1-ERCWH-139	3	RS	RH					
	1-ERCWH-140	3	RS	RH					
	1-ERCWH-142	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-ERCWH-143	3	RS	RS BE 415 Size 4					
	1-ERCWH-144	3	RS	RH					
	1-ERCWH-145	3	RS	RH					
	1-ERCWH-146	3	RS	RH					
	1-ERCWH-147	3	RS	RH					
	1-ERCWH-160	3	RS	RH					
	1-ERCWH-161	3	RS	RH					
	1-ERCWH-162	3	RS	RH					
	1-ERCWH-163	3	RS	RH					
	1-ERCWH-164	3	RS	RH					
	1-ERCWH-165	3	RS	RH					
	1-ERCWH-166	3	RS	RH					
	1-ERCWH-167	3	RS	RH					
	1-ERCWH-168	3	RS	RH					
	1-ERCWH-190	3	RS	RH					
	1-ERCWH-191	3	RS	RH					
	1-ERCWH-193	3	RS	RH					
	1-ERCWH-194	3	RS	RH					
	1-ERCWH-195	3	RS	RH					

IDENTIFIER		SAFETY CLASS	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT		
	1-ERCWH-196	3	RS	RH				
	1-ERCWH-197	3	RS	RH				
	1-ERCWH-198	3	RS	RH				
	1-ERCWH-204	3	RS	RH				
	1-ERCWH-205	3	RS	RH				
	1-ERCWH-206	3	RS	RH				
	1-ERCWH-207	3	RS	RH				
	1-ERCWH-208	3	RS	RH				
	1-ERCWH-209	3	RS	RS BE 415 Size 1				
	1-ERCWH-210	3	RS	RS BE 415 Size 1				
	1-ERCWH-211	3	RS	RH				
	1-ERCWH-212	3	RS	RH				
	1-ERCWH-213	3	RS	RH				
	1-ERCWH-214	3	RS	Rh				
	1-ERCWH-215	3	RS	RH				
	1-ERCWH-216	3	RS	RH				
	1-ERCWH-217	3	RS	RH				

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-ERCWH-218	3	RS	RH					
	1-ERCWH-219	3	RS	RH					
	1-ERCWH-220	3	RS	RH					
	1-ERCWH-221	3	RS	RH					
	1-ERCWH-222	3	RS	RH					
	1-ERCWH-223	3	RS	RS BE 415 Size 6					
	1-ERCWH-224	3	RS	RH					
	1-ERCWH-224A	3	RS	RH					
	1-ERCWH-225	3	RS	RS BE 415 Size 3					
	1-ERCWH-226	3	RS	RH					
	1-ERCWH-227	3	RS	RS BE 415 Size 7					
	1-ERCWH-227A	3	RS	RS BE 415 Size 7					
	1-ERCWH-228	3	RS	RH					
	1-ERCWH-229	3	RS	RH					
	1-ERCWH-233A	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S				HOT	COLD		
	1-ERCWH-233B	3	RS	RS BE 415 Size 3				
	1-ERCWH-234	3	RS	RH				
	1-ERCWH-235	3	RS	RH				
	1-ERCWH-235A	3	RS	RH				
	1-ERCWH-236	3	RS	RH				
	1-ERCWH-237	3	RS	RS BE 415 Size 4				
	1-ERCWH-238	3	RS	RH				
	1-ERCWH-239	3	RS	RS BE 415 Size 2				
	1-ERCWH-240	3	RS	RH				
	1-ERCWH-241	3	RS	RS BE 415 Size 2				
	1-ERCWH-242	3	RS	RH				
	1-ERCWH-243	3	RS	RH				
	1-ERCWH-262	3	RS	RH				
	1-ERCWH-263	3	RS	kH				

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-ERCWH-264	3	RS	RS BE 415 Size 1					
	1-ERCWH-265	3	RS	RH					
	1-ERCWH-266	3	RS	RH					
	1-ERCWH-267	3	RS	RH					
	1-ERCWH-268	3	RS	RH					
	1-ERCWH-269	3	RS	RH					
	1-ERCWH-270	3	RS	RH					
	1-ERCWH-271	3	RS	RH					
	1-ERCWH-272	3	RS	RH					
	1-ERCWH-273	3	RS	RH					
	1-ERCWH-274	3	RS	RH					
	1-ERCWH-275	3	RS	RH					
	1-ERCWH-276	3	RS	RH					
	1-ERCWH-277	3	RS	RH					
	1-ERCWH-300	3	RS	RH					
	1-ERCWH-301	3	RS	RH					
	1-ERCWH-302	3	RS	RH					
	1-ERCWH-342	3	RS	RH					

IDENTIFIER		SAFETY CLASS	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT		
	1-ERCWH-343	3	RS	RH				
	1-ERCWH-344	3	RS	RH				
	1-ERCWH-350	3	RS	RH				
	1-ERCWH-351	3	RS	RH				
	1-ERCWH-352	3	RS	RH				
	1-ERCWH-357	3	RS	RH				
	1-ERCWH-358	3	RS	RS BE 415 Size 1				
	1-ERCWH-359	3	RS	RS BE 415 Size 1				
	1-ERCWH-360	3	RS	RH				
	1-ERCWH-361	3	RS	RS BE 415 Size 1				
	1-ERCWH-362	3	RS	RH				
	1-ERCWH-363	3	RS	RH				
	1-ERCWH-364	3	RS	RH				
	1-ERCWH-365	3	RS	RH				
	1-ERCWH-366	3	RS	RS BE 415 Size 1				

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-ERCWH-367	3	RS	RH					
	1-ERCWH-368	3	RS	RH					
	1-ERCWH-370	3	RS	RH					
	1-ERCWH-371	3	RS	RH					
	1-ERCWH-372	3	RS	RS BE 415 Size 1					
	1-ERCWH-373	3	RS	RH					
	1-ERCWH-374	3	RS	RH					
	1-ERCWH-375	3	RS	RH					
	1-ERCWH-376	3	RS	RH					
	1-ERCWH-377	3	RS	RS BE 415 Size 1					
	1-ERCWH-378	3	RS	RH					
	1-ERCWH-379	3	RS	RS BE 415 Size 1					
	1-ERCWH-380	3	RS	RH					
	1-ERCWH-381	3	RS	RS BE 415 Size 1					
	1-ERCWH-382	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	1-ERCWH-500	3	RS	RH					
	1-ERCWH-505	3	RS	RH					
	1-ERCWH-511	3	RS	RH					
	1-ERCWH-512	3	RS	RH					
	1-ERCWH-513	3	RS	RH					
	1-ERCWH-514	3	RS	RH					
	1-ERCWH-515	3	RS	RH					
	1-ERCWH-520	3	RS	RH					

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Type  
 RS-Rigid Support  
 MS-Mechanical Snubber  
Model  
 RS-Rigid Strut  
 RH-Rigid Hanger

Plant: SQNP Unit: 2

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-SIH-72	3	MS	BE 416 PSA 10	6"	24 1/2"	24 3/8"	1/8"	1/4"-5 5/8"
	2-SIH-73	3	MS	BE 416 PSA 3	5"	1'-9 9/16"	1'-9"	9/16"	1/4"-4 3/16"
	2-SIH-74	3	RS	RH					
	2-SIH-112	3	RS	RH					
	2-SIH-400	3	RS	RH					
	2-SIH-401	3	RS	RH					
	2-SIH-402	3	RS	RH					
	2-SIH-403	3	RS	RH					
	2-SIH-404	3	RS	RH					
	2-SIH-405	3	RS	RH					
	2-SIH-406	3	RS	RH					
	2-SIH-407	3	RS	RH					
	2-SIH-408	3	RS	RH					
	2-SIH-409	3	RS	RH					

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-SIH-410	3	RS	BE 415 RS Size 5					
	2-SIH-411	3	RS	RH					
	2-SIH-412	3	RS	BE 415 RS Size 2					
	2-SIH-413	3	RS	RH					
	2-SIH-414	3	RS	RH					
	2-SIH-415	3	VS	Gr. Sprg Fig 82 Type B Size 12	2236#	2264#	1/8" Up	3/8"-1"	
	2-SIH-416	3	RS	RH					
	2-SIH-417	3	RS	RH					
	2-SIH-418	3	RS	RH					
	2-SIH-419	3	RS	RH					
	2-SIH-420	3	RS	RH					
	2-SIH-438	3	RS	RH					

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Type  
 VS-Variable Spring  
 RS-Rigid Support  
Model  
 RS-Rigid Strut  
 RH-Rigid Hanger

Plant: SQNP Unit: 2

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IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	DESIGN SETTING	THERMAL MOVEMENT	ACCEPTABLE RANGE		
TVA'S	ISI'S						MODEL	STROKE
	2-RHRH-431	3	VS	BE 401 Type F Size 11	1273#	1495#	5/16" Up	9/16"-1"
	2-RHRH-432	3	RS	RH				
	2-RHRH-433	3	RS	BE 415 RS Size 1				
	2-RHRH-434	3	RS	RH				
	2-RHRH-435	3	RS	RH				
	2-RHRH-436	3	RS	RH				
	2-RHRH-437	3	VS	BE 401 Type B Size 10	1175#	1234#	1/8" Up	3/8"-1"

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Type  
HS-Hydraulic Snubber  
IA-Integrally Welded Attachment  
MS-Mechanical Snubber  
RS-Rigid Support  
VS-Variable Spring  
Model  
RS-Rigid Strut  
RH-Rigid Hanger

Plant: SQNP Unit: 2

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IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-MSH-300	2	HS	BE 411 Size 8" X=1 1/4"	5"	4 1/4"	3 3/8"	7/8"T	1 1/2"-5 1/8"
	2-MSH-301	2	HS	BE 411 Size 8"	5"	3 7/16"	4 1/16"	5/8"C	2 1/8"-6"
	2-MSH-302	2	VS	Gr. Sprg. Fig. C-98 Type B Size 18		10,065#	12,725#	2" Up	2 1/4"-4 3/4"
	2-MSH-303	2	HS	BE 411 Size 4	5"	2 7/8"	4 3/8"	1 1/2"C	2 7/8"-5 7/8"
	2-MSH-304	2	VS	Gr. Sprg Fig 268 Type B Size 18		11,420#	13,016#	.6 Up	.85"-2 1/4"
	2-MSH-305	2	RS	RH					
	2-MSH-306	2	HS	BE 410 Size 4	5"	3 1/2"	2 3/4"	3/4"	1 3/8"-5 1/8"
	2-MSH-307	2	VS	Gr. Sprg. Fig C-268 Type A Size 22		37,210#	30,542#	.8" Down	.25"-1.45"

IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-MSH-308	2	VS	Gr. Sprg. Fig C-268 Type F Size 21		33,840#	20,762#	.82 Down	.25"-1.43"
	2-MSH-309	2	MS	BE 416 Size PSA 10	6"	2'-0 1/16"	2'-0 13/16"	.75"C	1.00"-5.75"
	*2-MSH-310	2	HS	BE 410 Size 3 1/4"	5"	2.875"	3 1/4"	.375"C	1.75"-5.875"
	*2-MSH-310	2	HS	BE 410 Size 3 1/4"	5"	3.125"	3 1/2"	.375"C	1.75"-5.875"
	2-MSH-311	2	RS	BE 415 RS Size #4					
	2-MSH-312	2	IA						
	2-MSH-312	2	RS	RH					
	2-MSH-313	2	RS	RH					
	2-MSH-314	2	HS	BE 410 Size 3 1/4" X=1 1/8"	5"	3 7/8"	3 3/8"	1/2"	1.375"-5.375"
	2-MSH-315	2	HS	BE 411 Size 3 1/4" X=1 1/8"	5"	4 1/16"	2 9/16"	1.5"T	1.375"-4.375"
	2-MSH-316	2	RS	RH					
	2-MSH-317	2	RS	RH					

\*Snubbers to operate simultaneously.

IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE	
TVA'S	ISI'S				HOT	COLD			
	2-MSH-340	2	HS	BE 411 Size 8" X=1 1/4"	5"	4 1/4"	3 5/16"	.938T	1.50"-5.062"
	2-MSH-341	2	HS	BE 411 Size 8" X=1 1/4"	5"	3 3/8"	4 1/8"	3/4"C	2.25"-6"
	2-MSH-342	2	VS	Gr. Sprg. F-C-98 Type B Size 18		10,885#	13,412#	1 15/16" Up	2.19"-4.75"
	2-MSH-343	2	HS	BE 411 Size 4 X=1 1/8"	5"	2 7/8"	4 3/8"	1 1/2"C	2.875"-5.875"
	2-MSH-344	2	VS	Gr. Sprg. Fig. C-268 Type B Size 18		10,380#	11,604#	7/16" Up	.75"-2.25"
	2-MSH-345	2	RS	RH					
	2-MSH-346	2	VS	Gr. Sprg. Fig C-268 Type C Size 19		16,925#	14,093#	13/16" Down	.25"-1.44"
	2-MSH-347	2	MS	BE 416 PSA 35	6"	2'-6 13/16"	2'-6"	13/16"T	.25"-3.937"
	2-MSH-348	2	HS	BE 410 Size 4 X=1.125	5"	4.5"	3.69"	.81"T	1.375"-5.065"
	2-MSH-349	2	IA						

IDENTIFIER		SAFETY CLASS	SUPPORT			DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT	COLD		
	2-MSH-349	2	RS	RH					
	2-MSH-350	2	MS	BE 416 PSA 10	6"	1'-6 7/16"	1'-7 1/8"	11/16"C	15/16"-5.75"
	2-MSH-351	2	VS	Gr. Sprg. Fig C-268 Type C Size 19		18,600#	16,167#	11/16" Down	.25"-1.562"
	2-MSH-352	2	HS	BE 410 Size 4"	5"	3 11/16"	3 1/2"	3/16"T	1.375"-5.69"
	2-MSH-353	2	MS	BE 417 PSA 10	6"	4'-1 5/16"	4'-0 5/16"	1"T	.25"-4.75"
	2-MSH-354	2	RS	RH					
	2-MSH-355	2	RS	RH					
	2-MSH-380	2	HS	BE 411 Size 8" X=1.25"	5"	4 3/16"	3 5/16"	7/8"T	1.5"-5.125"
	2-MSH-381	2	HS	BE 411 Size 8" X=1.25"	5"	3 7/16"	4 1/16"	5/8"C	2.125"-6"
	2-MSH-382	2	VS	Gr. Sprg. Fig C-98 Type B Size 18		11,380#	14,040#	2" Up	2.25"-4.75"
	2-MSH-383	2	HS	BE 411 Size 4 X=1.25"	5"	3 7/8"	4 3/8"	1.5"C	2.875"-5.875"



IDENTIFIER		SAFETY CLASS	SUPPORT			DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S		TYPE	MODEL	STROKE	HOT	COLD		
	2-MSH-384	2	VS	Gr. Sprg. Fig C-268 Type D Size 15		4,565#	5,105#	1/2" Up	.75"-2.25"
	2-MSH-385	2	RS	RH					
	2-MSH-386	2	VS	Gr. Sprg. C-268 Type C Size 20		26,050#	22,760#	.7" Down	.25"-1.55"
	2-MSH-387	2	MS	BE 416 PSA 10	6"	23 1/16"	22 1/4"	13/16" T	.25"-4.94"
	2-MSH-388	2	VS	Gr. Sprg. Fig C-268 Type D Size 15		6,370#	5,506#	13/16" Down	.24"-1.44"
	2-MSH-389	2	IA						
	2-MSH-389	2	RS	RH					
	2-MSH-390	2	VS	Gr. Sprg. Fig C-268 Type B Size 19		18,705#	16,935#	1/2" Down	.25"-1.75"
	2-MSH-391	2	MS						
	2-MSH-392	2	HS	BE 410 Size 3 1/4" X=1.125	5"	3.75"	3.56"	.188T	1.38"-5.69"
	2-MSH-393	2	MS	BE 417 PSA 10	6"	3'-10 15/16"	4'-0 5/16"	1.4" c	1.65"-5.75"

IDENTIFIER		SAFETY CLASS	TYPE	SUPPORT		DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S			MODEL	STROKE	HOT	COLD		
	2-MSH-394	2	RS	RH					
	2-MSH-395	2	RS	RH					
	2-MSH-420	2	HS	BE 411 Size 8" X=1.25"	5"	4.1875"	3.1875"	1" T	1.5"-5"
	2-MSH-421	2	HS	BE 411 Size 8" X=1.25"	5"	2.56"	3.25"	.69" C	2.19"-6"
	2-MSH-422	2	VS	Gr. Sprg. Fig C-98 Type B Size 18		10,735#	13,528#	2 1/8" Up	2.375"-4.75"
	2-MSH-423	2	HS	BE 411 Size 4" X=1.125"	5"	2.875"	4.375"	1.5" C	2.875"-5.875"
	2-MSH-424	2	RS	BE 415 RS Size 11					
	2-MSH-425	2	VS	Gr. Sprg. Fig C-268 Type D Size 15		4,480#	5,128#	.625" Up	.875"-2.25"
	2-MSH-426	2	RS	RH					
	2-MSH-427	2	VS	Gr. Sprg. Fig C-268 Type F Size 20		23,930#	20,640#	.7 Down	.25"-1.55"
	2-MSH-428	2	RS	RH					

IDENTIFIER		SAFETY CLASS	SUPPORT TYPE	MODEL	STROKE	DESIGN SETTING		THERMAL MOVEMENT	ACCEPTABLE RANGE
TVA'S	ISI'S					HOT	COLD		
	2-MSH-429	2	VS	Gr. Sprg. Fig C-268 Type D Size 19		17,150#	15,734#	.375" Down	.25"-1.875"
	2-MSH-430	2	MS	BE 417 PSA 10	6"	20"	20 3/8"	3/8"C	.625"-5.75"
	2-MSH-431	2	RS	BE 415 RS Size 6					
	2-MSH-432	2	HS	BE 410 Size 4" X=1.125"	5"	3.57"	3.69"	.12"C	1.5"-5.875"
	2-MSH-433	2	MS	BE 417 PSA 10	6"	1'-8"	1'-9 1/8"	1.125"C	1.375"-5.75"
	2-MSH-434	2	RS	RH					
	2-MSH-435	2	RS	RH					
	2-MSH-438	2	RH	RH					
	2-MSH-439	2	RS	BE 415 RS Size 1					
	2-MSH-440	2	MS	BE 416 PSA 1/2	2 1/2	10 1/2	11 13/16	1 5/16C	1 9/16"-2 1/4"