



Commonwealth Edison

Quad Cities Nuclear Power Station
22710 208 Avenue North
Cordova, Illinois 61242
Telephone 309/654-2241

GCT-92-027

June 4, 1992

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of May 1992.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

Gerald C. Tietz
Gerald C. Tietz
Technical Superintendent

GCT/MB/dak

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

MAY 1992

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241, extensions 2995 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

For the month of May 1992, Quad Cities Unit One performed nine load reductions per Chicago Load Dispatch (CLD). They occurred on the following days to the limiting load provided:

<u>Day</u>	<u>Load</u>	<u>Additional Reason</u>
5-1-92	725 MWe	
5-2-92	750 MWe	
5-2-92	225 MWe	Drywell Equipment Drain Sump Replacement.
5-6-92	665 MWe	
5-9-92	535 MWe	
5-11-92	650 MWe	
5-13-92	550 MWe	
5-15-92	700 MWe	
5-21-92	600 MWe	

On May 30, Unit One was shutdown to repair tube leaks on the main condenser.

B. Unit Two

Quad Cities Unit Two went critical on May 5 to end Refuel Outage Q2R11. The generator was loaded to the grid on May 11 and full load was reached on May 17. The period before reaching full load had a number of power changes below 50% for testing purposes (i.e. RCIC, TIP, etc...). On May 23, power was reduced to 465 MWe to determine the Flow Control Line (FCL). CLD ordered load reductions to 700 MWe for the mornings of May 24 and 25.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS,
AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q00783	0670	Replace secondary disconnect MF and MG in Cub 1 Bus 13-1.	As Found: MF and MG slide contacts broken. As Left: Installed slide contacts into Cubicle 10 into Cubicle 1.
Q98600	6700	Repair racking screw on breaker #60.	As Found: Racking screw slightly bent. As Left: Replaced racking screw.
Q99947	9400	Replace Compressor "B" Train A/C with like for like.	As Left: Replaced compressor.

UNIT 2 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q00535	6705	Output breaker from 1/2 DG to BUS 23-1 damaged.	As Found: Secondary contacts broken. As Left: Installed new sliding contact block.
Q00560	2301	Repair leak on HPCI oil filter.	As Found: O-Ring hard and cracked. As Left: Replaced with new O-Ring.
Q00686	1601	Repair air leaks and check valves on air control system on torus relief valve.	As Found: No visual damage. As Left: Tightened fitting.
Q00690	1601	Repair air leak on valve 2-1601-56 Torus Purge.	As Found: Air lines leaking. As Left: Repaired air lines.
Q00932	756	Investigate and repair Rod Block Monitor #7.	As Found: RBM 7 bypassed. As Left: Cleaned edge connector contacts. RBM 7 operational.
Q86175	302	Repair SCRAM Discharge Volume Level Switch.	As Left: Replaced relay on aux relay board.
Q95695	1402	Investigate and repair Solenoid 2B CS Testable Check Valve.	As Left: Replaced solenoid with a new one. Installed new magnetic switch. Corrected wiring.
Q98674	8292	Replace the circuit breaker for the U2 DG cooling pump fan - Cubicle E5.	As Found: Cubicle at MCC 29-2 intact. As Left: Installed new breaker into cubicle.
Q98675	8292	Replace the circuit breaker for the U2 DG cooling pump fan - Cubicle F1.	As Found: Cubicle at MCC 29-2 intact. As Left: Installed new breaker into cubicle.
Q99329	756/2417	Repair power supply to High Voltage LPRM 24-17 Detector B.	As Left: Replaced power supply.

UNIT 2 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q99380	1402	Investigate and repair B Core Spray Testable Check Valve.	As Found: Packing was loose. As Left: Realigned shafts. Replaced missing tubing connector.
Q99679	261	Repair spring can on flow flash pot.	As Found: Rods were bent. As Left: Replaced rod.
Q99767	2252	Repair U2 DG room panel 2252-37 wiring.	As Found: Removed pinched section of wire and installed new lug. Installed new ty-raps.
Q99830	203	Repair main steam line inside isolation air operated valve.	As Found: Limit switches set incorrectly. As Left: Reset limit switches. Lubricated stem and spring guides.
Q99842	201	Repair support for spring can for high side line 261-9.	As Found: Rods bent or missing. As Left: Replaced bolts and nuts.
Q99842	201	Repair support for spring can for high side line 261-9.	As Found: Rods bent or missing. As Left: Replaced bolts and nuts.
Q99842	201	Repair support for spring can for low side line 261-9 A.	As Found: Rods bent or missing. As Left: Replaced bolts and nuts.
Q99842	201	Repair support for spring can for low side line 261-9 B.	As Found: Rods bent or missing. As Left: Replaced bolts and nuts.
Q99853	756/3249	Investigate and repair LPRM 32-49 Detector B cards.	As Found: Lo light unlit. As Left: Calibrated downscale alarm light.
Q99868	1001	Replace contactor for limiterque RHR LPCI mode inject valve inboard.	As Found: Contact was removed. As Left: Installed new contactor assembly.

UNIT 2 MAINTENANCE SUMMARY

<u>WORK REQUEST</u>	<u>SYSTEM</u>	<u>EID DESCRIPTION</u>	<u>WORK PERFORMED</u>
Q99905	2301	Repair air leak on solenoid HPCI testable check valve.	As Left: Replaced solenoid. No leaks found after snooping.
Q99940	220	Replace broken support on U2 D.W. spring can pipe.	As Found: Rods broken. As Left: Installed new rod.
Q99943	756	Investigate power to B and C level LPRMs from APRM #1.	As Found: Fuse blown for the 5V power supply for the A and C LPRM's. APRM bypassed. As Left: Pulled trip card and installed new fuse. Work request written for repair of bad card.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

UNIT 1

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
92-011	05-30-92	A Recirc Loop Temp Recorder not working. Missed T.S. Surveillance.
92-013	04-23-92	Second level undervoltage relays setpoint nonconservative. This one from last month was cancelled. Included in 2-92-013.

UNIT 2

92-015	05-12-92	RCIC Flow Oscillations - RCIC INOP.
92-016	05-19-92	CAM line pipe hanger outside design basis.
92-017	05-24-92	RCIC INOP due to a hole in the 2B C.S./RCIC submarine door.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

0
2
9

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-254
UNIT One
DATE June 4, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

0000 050192
1. REPORTING PERIOD: 2400 053192 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>721.20</u>	<u>3351.50</u>	<u>139862.60</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. HOURS GENERATOR ON LINE	<u>721.10</u>	<u>3333.10</u>	<u>135564.20</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. GROSS THERMAL ENERGY GENERATED (MWH).....	<u>1678708.8</u>	<u>8015704.0</u>	<u>292065934.0</u>
10. GROSS ELECTRICAL ENERGY GENERATED (MWH).....	<u>539753.0</u>	<u>2611829.0</u>	<u>94686165.0</u>
11. NET ELECTRICAL ENERGY GENERATED (MWH).....	<u>515825.0</u>	<u>2525975.0</u>	<u>89235343.0</u>
12. REACTOR SERVICE FACTOR.....	<u>96.94</u>	<u>91.90</u>	<u>79.23</u>
13. REACTOR AVAILABILITY FACTOR.....	<u>96.94</u>	<u>91.90</u>	<u>81.17</u>
14. UNIT SERVICE FACTOR	<u>96.92</u>	<u>91.39</u>	<u>76.80</u>
15. UNIT AVAILABILITY FACTOR	<u>96.92</u>	<u>91.39</u>	<u>77.31</u>
16. UNIT CAPACITY FACTOR (Using MDC)	<u>90.16</u>	<u>90.07</u>	<u>65.74</u>
17. UNIT CAPACITY FACTOR (Using Design MWe)	<u>87.87</u>	<u>87.78</u>	<u>64.07</u>
18. UNIT FORCED OUTAGE RATE	<u>6.08</u>	<u>9.22</u>	<u>5.81</u>

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-265
UNIT Two
DATE June 4, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 554-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 050192 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>555.30</u>	<u>555.55</u>	<u>134038.25</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>2985.3</u>
7. HOURS GENERATOR ON LINE	<u>460.30</u>	<u>460.55</u>	<u>130480.45</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. GROSS THERMAL ENERGY GENERATED (MWH).....	<u>1007769.6</u>	<u>1007944.80</u>	<u>281091831.80</u>
10. GROSS ELECTRICAL ENERGY GENERATED (MWH).....	<u>320051.00</u>	<u>320101.00</u>	<u>90250291.00</u>
11. NET ELECTRICAL ENERGY GENERATED (MWH).....	<u>303536.00</u>	<u>281910.00</u>	<u>85416085.00</u>
12. REACTOR SERVICE FACTOR.....	<u>74.64</u>	<u>15.23</u>	<u>76.61</u>
13. REACTOR AVAILABILITY FACTOR.....	<u>74.64</u>	<u>15.23</u>	<u>78.32</u>
14. UNIT SERVICE FACTOR	<u>61.87</u>	<u>12.63</u>	<u>74.58</u>
15. UNIT AVAILABILITY FACTOR	<u>61.87</u>	<u>12.63</u>	<u>74.98</u>
16. UNIT CAPACITY FACTOR (Using MDC)	<u>53.05</u>	<u>10.05</u>	<u>63.49</u>
17. UNIT CAPACITY FACTOR (Using Design MWe)	<u>51.71</u>	<u>9.80</u>	<u>61.88</u>
18. UNIT FORCED OUTAGE RATE	<u>0.0</u>	<u>0.0</u>	<u>8.06</u>

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE June 4, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH May 1992

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>764</u>
2.	<u>762</u>
3.	<u>350</u>
4.	<u>693</u>
5.	<u>776</u>
6.	<u>777</u>
7.	<u>737</u>
8.	<u>775</u>
9.	<u>709</u>
10.	<u>772</u>
11.	<u>732</u>
12.	<u>766</u>
13.	<u>715</u>
14.	<u>771</u>
15.	<u>764</u>
16.	<u>767</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>766</u>
18.	<u>767</u>
19.	<u>770</u>
20.	<u>766</u>
21.	<u>763</u>
22.	<u>706</u>
23.	<u>764</u>
24.	<u>768</u>
25.	<u>771</u>
26.	<u>773</u>
27.	<u>772</u>
28.	<u>770</u>
29.	<u>702</u>
30.	<u>0</u>
31.	<u>-8</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE June 4, 1992
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH May 1992

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u> -8 </u>
2.	<u> -8 </u>
3.	<u> -8 </u>
4.	<u> -8 </u>
5.	<u> -8 </u>
6.	<u> -8 </u>
7.	<u> -8 </u>
8.	<u> -8 </u>
9.	<u> -8 </u>
10.	<u> -8 </u>
11.	<u> 46 </u>
12.	<u> 143 </u>
13.	<u> 237 </u>
14.	<u> 265 </u>
15.	<u> 355 </u>
16.	<u> 408 </u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u> 527 </u>
18.	<u> 774 </u>
19.	<u> 785 </u>
20.	<u> 783 </u>
21.	<u> 782 </u>
22.	<u> 782 </u>
23.	<u> 701 </u>
24.	<u> 750 </u>
25.	<u> 760 </u>
26.	<u> 789 </u>
27.	<u> 787 </u>
28.	<u> 790 </u>
29.	<u> 785 </u>
30.	<u> 783 </u>
31.	<u> 789 </u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Unit Two

DATE June 4, 1992

COMPLETED BY Matthew Benson

TELEPHONE 309-654-2241

REPORT MONTH May 1992

NO.	DATE	TYPE P OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-01	5-01-92	S	188.7	C	4	-	-	-	Continuation of Unit 2 Scheduled Refuel Outage.
92-02	5-23-92	S	4.4	B	5	-	-	-	Flow Control Line Determination
92-03	5-24-92	S	10.1	F	5	-	-	-	Reduced Load per Chicago L.D.
92-04	5-25-92	S	5.7	F	5	-	-	-	Reduced Load per Chicago L.D.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME Unit 1

DATE June 4, 1992

REPORT MONTH May 1992

COMPLETED BY Matthew Benson

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
92-13	5-01-92	S	4.2	F	5	-	-	-	Reduced Load per Chicago L.D.
92-14	5-02-92	S	1.2	F	5	-	-	-	Reduced Load per Chicago L.D.
92-15	5-02-92	S	30.7	F	5	-	-	-	Reduced Load per Chicago L.D.
92-16	5-06-92	S	6.3	F	5	-	-	-	Reduced Load per Chicago L.D.
92-17	5-09-92	S	5.4	F	5	-	-	-	Reduced Load per Chicago L.D.
92-18	5-11-92	S	5.0	F	5	-	-	-	Reduced Load per Chicago L.D.
92-19	5-13-92	S	5.6	F	5	-	-	-	Reduced Load per Chicago L.D.
92-20	5-15-92	S	1.5	F	5	-	-	-	Reduced Load per Chicago L.D.
92-21	5-21-92	S	7.8	F	5	-	-	-	Reduced Load per Chicago L.D.
92-22	5-30-92	S	71.8	B	2	-	-	-	Condenser Work

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit: Two

Date: 5/11/92

Valves Actuated
2-203-3A, 3B, 3C, 3D, 3E

No. & Type of Actuation
Normal Surveillance and Post
Maintenance following Q2R11.

Plant Conditions: Startup from Q2R11.

Description of Events:

The 2-203-3A Target Rock Safety/Relief valve was operated twice due to the Acoustic monitor associated by this valve not giving open indication on the first try. Acoustic Monitor worked on second operation.

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS
 PERFORMED ON UNIT 1 & 2 CONTROL
 ROD DRIVES, FROM 1-1-92 TO 12-31-92

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5	20	50	90		
		0.375	0.900	2.00	3.5	7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
2-19-92	2	0.28	0.67	1.43	2.48	J-2 2.55	Drive Replacement (J-2), Scram Valve N-7
2-20-92	1	0.32	0.69	1.45	2.45	N-5 2.45	Scram Valve Work N-5
5-12-92	177	0.31	0.69	1.47	2.58	L-13 3.43	Start-up Scram Timing Unit Two

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-532
Revision 2
October 1989

1. Unit: Q1 Reload: 11 Cycle: 12
2. Scheduled date for next refueling shutdown: 9-5-92
3. Scheduled date for restart following refueling: 12-5-92
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:
NOT AS YET DETERMINED.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
NOT AS YET DETERMINED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
NONE AT PRESENT TIME.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 1405
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3657
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 2
October 1989

1. Unit: Q2 Reload: 11 Cycle: 12
2. Scheduled date for next refueling shutdown: 03/06/93
3. Scheduled date for restart following refueling: 06/05/93
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:
NOT AS YET DETERMINED.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
NOT AS YET DETERMINED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
NONE AT PRESENT TIME.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 2439
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3897
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLRT	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRs
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCIOMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RPS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRs
SDV	- Scram Discharge Volume
SRM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center