UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

PUBLIC SERVICE COMPANY OF COLORADO

FORT ST. VRAIN NUCLEAR GENERATING STATION

DOCKET NO. 50-267

AMENDMENT TO POSSESSION ONLY LICENSE

Amendment No. 84
License No. DPR-34

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by Public Service Company of Colorado (the licensee), dated December 24, 1991, as supplemented April 14, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will be maintained in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment, and paragraphs 2.D.(2) and 2.D.(3) of Possession Only License No. DPR-34 are hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 84, are hereby incorporated in the license. The licensee shall maintain the facility in accordance with the Technical Specifications.

(3) Until all nuclear fuel has been removed from the Protected Area, the licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: Fort St. Vrain Security Plan, Fort St. Vrain Guard Training and Qualification Plan, and Fort St. Vrain Safeguards Contingency Plan. All plans are with revisions submitted through June 6, 1990, as supplemented by letter dated September 14, 1990. After all nuclear fuel has been removed from the Protected Area, these plans may be terminated.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Seymour H. Weiss

Seymour H. Weiss, Director
Non-Power Reactors, Decommissioning
and Environmental Project Directorate
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosure:
Appendix A Technical
Specifications Changes

Date of Issuance: June 5, 1992

ENCLOSURE TO LICENSE AMENDMENT NO. 84

POSSESSION ONLY LICENSE NO. DPR-34

DOCKET NO. 50-267

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	Insert
7.1-1	7.1-1
7.1-2	7.1-2
7.1-3	7.1-3
7.1-4	7.1-4
7.1-6	7.1-6
7.1-7	7.1-7
7.1-8	7.1-8
7.1-8a	7.1-8a
7.1-8b	7.1-8b
7.1-8c	7.1-8c
7.1-9	7.1-9
7.1-10	7.1-10
7.1-11	7.1-11
7.1-11a	7.1-11a
7.1-12	7.1-12
7.1-13	7.1-13
7.1-14	7.1-14
7.1-18	7.1-18
7.1-19	7.1-19
7.2-1	7.2-1
7.4-1	7.4-1
7	7.5-3
7.	7.5-4
7..	7.5-7
	7.8-1
	7.8-2

7.1 ORGANIZATION, REVIEW, AND AUDIT-ADMINISTRATIVE CONTROLS

Applicability

Applies to the lines of authority and responsibility for the operational safety of the facility, and the organization for periodic review and audit of facility operation.

Objectives

To define the principal lines of authority and responsibility for providing continuing review, evaluation, and improvement of the plant operational safety.

SPECIFICATION AC 7.1.1 - ORGANIZATION, ADMINISTRATIVE CONTROLS

1. RESPONSIBILITY

   a. The Defueling and Decommissioning Program Director and Station Manager shall be responsible for overall unit operation and shall delegate in writing the succession to this responsibility during his absence.

   b. The Shift Supervisor (or during his absence from the control room and Shift Supervisor's office, a designated individual) shall be responsible for the control room command function. A management directive to this effect, signed by the Vice President responsible for nuclear activities shall be reissued to all station personnel on an annual basis.

2. ORGANIZATION

a. Onsite and Offsite Organizations

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

(1) Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels, including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the FSAR.

(2) The Defueling and Decommissioning Program Director and Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.

(3) The Vice President responsible for nuclear activities shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

(4) The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.

b. Unit Staff

(1) Until all nuclear fuel has been removed from the Protected Area (as shown on Figure 6.3-2), each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 7.1-1.

(2) A licensed operator must be in the control room at all times when fuel is in the reactor.

(3) During shutdown and defueling conditions, until all nuclear fuel has been removed from the Protected Area, an individual with a valid RO (or SRO) license shall be present in the control room.

(4) During the performance of reactor startup, reactor shutdown, and recovery from reactor trip, two licensed operators must be in the control room.

(5) All core alterations shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

(6) An operator or technician qualified in radiation protection procedures shall be present at the facility at all times that there is nuclear fuel within the Protected Area, and during any physical activities involving radiologically contaminated systems or equipment.

(7) Fire Brigade staffing requirements have been incorporated into the Fire Protection Program Plan. A Fire Brigade is not required after all nuclear fuel has been removed from the Protected Area.

(d) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on shift.

(9) If unusual circumstances arise requiring deviation from the above guidelines, such deviation shall be authorized by the Defueling and Decommissioning Program Director and Station Manager, his designee, or higher levels of management, and with documentation of the basis for granting the deviation. Controls shall be established such that excessive individual overtime hours have not been assigned. The paramount consideration in overtime assignment shall be that significant reductions in the effectiveness of operating personnel would be highly unlikely. Routine deviation from the above guidelines is not authorized.

Authorized deviations to the working hour guidelines shall be documented and available for review by the Nuclear Regulatory Commission.

(10) The Shift Supervisors, and the Operations Manager shall hold a Senior Reactor Operator's license. The Reactor Operators shall hold a Reactor Operator's license. These are not requirements after all nuclear fuel has been removed from the Protected Area.

(11) Except for the Shift Supervisor, the Shift Crew Composition may be one less than the minimum requirements of Table 7.1-1 for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on-duty shift crew members, provided immediate action is taken to restore the Shift Crew Composition to within the minimum requirements of Table 7.1-1. This provision does not permit any shift crew position to be unmanned upon shift change due to an oncoming shift crewman being late or absent. This provision also does not exclude the requirement for an SRO licensed individual to be in the control room at all times other than the shutdown conditions specified in Table 7.1-1.

(12) During any absence of the Shift Supervisor from the control room or Shift Supervisor's office while the unit is in Hot, Cold, or Refueling Shutdown, an individual with a valid SRO license or RO license shall be designated to assume the control room command function. During any absence of the Shift Supervisor from the control room or Shift Supervisor's office during all other conditions, an individual with a valid SRO license shall be designated to assume the control room command function. These are not requirements after all nuclear fuel has been removed from the Protected Area.

TABLE 7.1-1

MINIMUM SHIFT CREW COMPOSITION (c)

Position	Number of Individuals Required to Fill Position	
	During Hot, Cold, or Refueling Shutdown (a)	All Other Conditions
SS (SRO)	1	1
SRO	Not Required	1
RO	1	2 (b)
EO	1	1
AT	Not Required	1

SS - Shift Supervisor with a Senior Reactor Operator's License
SRO - Individual with a Senior Reactor Operator's License
RO - Individual with a Reactor Operator's License
EO - Equipment Operator
AT - Auxiliary Tender

NOTES

a. Per Technical Specification definitions, Section 2.0.

b. One of the two Reactor Operators may be an Equipment Operator with a valid RO license provided that the staffing requirement for Equipment Operators is being met by another individual qualified as an Equipment Operator.

c. Licensed operators and a minimum shift crew composition are not required after all nuclear fuel has been removed from the Protected Area.

3. TECHNICAL ADVISORS

These requirements have been deleted in their entirety.

4. UNIT STAFF QUALIFICATIONS

a. The staffing of the plant shall be in accordance with American National Standards Institute (ANSI) N18.1-1971, "Selection and Training of Personnel for Nuclear Power Plants".

b. Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for the comparable position, except for the Radiation Protection Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September, 1975.

5. TRAINING

a. A retraining and replacement training program for the facility staff shall be maintained under the direction of the Facility Support Manager, and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR 55.

b. Until all nuclear fuel has been removed from the Protected Area, a training program for the Fire Brigade shall be maintained under the direction of the Facility Support Manager, and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for Fire Brigade training/drill sessions which shall be held at least once per calendar quarter.

Specification AC 7.1.2 - Plant Operations Review Committee (PORC), Administrative Controls

The organization, responsibilities, and authority of the PORC shall be as follows:

1. Membership

The Plant Operations Review Committee shall be composed of the following:

Chairman: Defueling and Decommissioning Program Director and Station Manager
Deputy Program Director
Facility Support Manager
Radiation Protection Manager
Operations Manager
Defueling/Maintanance Manager
Systems Engineering Manager
Decommissioning Engineering Manager

2. Alternates

An alternate chairman and alternate members, if required, shall be appointed in writing by the PORC Chairman to serve in the absence of a chairman or a member; however, no more than two alternate members shall participate as voting members in PORC activities at any one time.

3. Meeting Frequency

The PORC shall meet at least once per calendar month and as convened by the Chairman or his designated alternate.

4. Quorum

A quorum shall consist of the Chairman or alternate Chairman, and four members including alternates.

5. Responsibilities

The PORC shall be responsible for:

a. Review of all procedures required by Technical Specification 7.4(a), (b), (c), (d) and changes thereto, and any other proposed procedure or changes to approved procedures as determined by the Defueling and Decommissioning Program Director and Station Manager to affect nuclear safety.

b. Review of all proposed tests and experiments that affect nuclear safety.

c. Review of all proposed changes to the Technical Specifications.

d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety

e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering the evaluation and recommendations to prevent recurrence to the Defueling and Decommissioning Program Director and Station Manager and to the Chairman of the Nuclear Facility Safety Committee.

f. Review of all Reportable Events.

g. Review of facility operations to detect potential nuclear safety hazards.

h. Performance of special reviews, investigations, and reports thereon as requested by the Chairman of the Nuclear Facility Safety Committee.

i. Review of the Plant Security Plan and implementing procedures.

j. Review of the plant Defueling Emergency Response Plan and implementing procedures.

k. Review of the Fire Protection Program Plan and implementing procedures.

6. Authority

The PORC shall:

a. Function to advise the Defueling and Decommissioning Program Director and Station Manager on all matters that affect nuclear safety.

b. Recommend to the Defueling and Decommissioning Program Director and Station Manager in writing, approval or disapproval of items considered under 5.a through 5.d, above.

c. Render determinations in writing with regard to whether or not each item considered under 5.a through 5.e above constitutes an unreviewed safety question.

d. Provide immediate written notification to the Defueling and Decommissioning Program Director and Station Manager, and the Chairman of NFSC of disagreement between the PORC and the Defueling and Decommissioning Program Director and Station Manager; however, the Defueling and Decommissioning Program Director and Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.a above.

7. Records

The PORC shall maintain written minutes of each meeting and copies shall be provided to the Defueling and Decommissioning Program Director and Station Manager, and Chairman of the Fort St. Vrain Nuclear Facility Safety Committee.

Specification AC 7.1.3 - Nuclear Facility Safety Committee (NFSC), Administrative Controls

The organization, responsibilities, and authority of the NFSC shall be as follows:

1. Function

The Nuclear Facility Safety Committee shall collectively have the competence required to review problems in the following areas:

a. Nuclear Power Plant Operations

b. Nuclear Engineering

c. Chemistry and Radiochemistry

d. Metallurgy

e. Instrumentation and Control

f. Radiological Safety

g. Mechanical and Electrical Engineering

h. Quality Assurance Practices

i. Other appropriate fields associated with the unique characteristics of the nuclear power plant.

2. Membership

The NFSC shall be composed of the following:

Chairman (As appointed - See Step 4)

| Defueling and Decommissioning Program Director and Station Manager

| Deputy Program Director

| Nuclear Licensing Manager

| Systems Engineering Manager

| Decommissioning Engineering Manager

| Quality Assurance Manager

| Radiation Protection Manager

Safety and Security Director

| Consultants, as required, shall be appointed in writing by the Vice President responsible for nuclear activities.

3. Alternates

Alternate members, if required, shall be appointed in writing by

| the Vice President responsible for nuclear activities; however, no more than two alternate members shall participate as voting members in NFSC activities at any one time.

4. Chairman

The Chairman and Alternate Chairman of the NFSC shall be appointed in writing by the Vice President responsible for nuclear activities and shall serve as members of the NFSC.

5. Consultants

Consultants shall be utilized as determined by the Chairman, NFSC, to provide expert advice to the NFSC.

6. Meeting Frequency

The NFSC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

7. Quorum

A quorum of the NFSC shall consist of the Chairman or his designated alternate and a majority of the NFSC members including alternates. No more than a minority of the quorum shall have line responsibilities for operation of the facility.

8. Responsibilities

a. The Nuclear Facility Safety Committee shall review:

(1) The safety evaluations for safety significant changes to procedures, equipment, or systems

personnel or an outside fire protection firm;

(b) a biennial audit of the fire protection program and implementing procedures;

(c) a triennial fire protection and loss prevention inspection and audit utilizing an outside qualified fire consultant.

(9) The Offsite Dose Calculation Manual and Process Control Program and implementing procedures at least once per 24 months.

(10) The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months.

(11) The performance of activities required by the Quality Assurance Program to meet the provisions of Regulatory Guide 1.21 Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975, at least once per 12 months.

9. Authority

The NFSC shall report to and advise the Vice President responsible for nuclear activities on those areas of responsibility specified in 8.a, 8.b and 8.c above.

10. Records

Records of NFSC activities shall be prepared, approved, and distributed as indicated below:

a. Minutes of each NFSC meeting shall be prepared and forwarded to the Vice President responsible for nuclear activities within 30 days following each meeting.

b. After preliminary approval by the Vice President responsible for nuclear activities, the minutes shall be distributed to all NFSC members, and approved at the next NFSC meeting.

c. Reports of reviews encompassed by Section 8.a, above shall be forwarded to the Vice President responsible for nuclear activities, within 30 days following completion of the review.

d. Audit reports encompassed by Section 8.c, above shall be forwarded to the Vice President responsible for nuclear activities, and to the management positions responsible for the areas audited within 30 days after completion of the audit.

7.2 SAFETY LIMITS, ADMINISTRATIVE CONTROLS

Applicability

Applies to the administrative procedures to be followed in the event that a safety limit is exceeded.

Objectives

To define the administrative procedures which will be followed in the event that a safety limit is exceeded.

Specification AC 7.2 - Action to be Taken if a Safety Limit is Exceeded, Administrative Controls

If a safety limit is exceeded, as defined in Specification SL 3.1 and 3.2, the following action shall be taken:

a. The reactor will be shut down immediately and reactor operations shall not be resumed until approval is received from the NRC.

b. The safety limit violation shall be reported to the Commission, the Defueling and Decommissioning Program Director and Station Manager, and to the Chairman, NFSC immediately.

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission, the Chairman, NFSC and the Defueling and Decommissioning Program Director and Station Manager within ten days of the violation.

7.4 PROCEDURES - ADMINISTRATIVE CONTROLS

Applicability

Applies to administrative procedures which will govern plant operations.

Objective

To ensure that written procedures will be maintained to define requirements for plant operation.

Specification AC 7.4 - Procedures, Administrative Controls

a. Written procedures shall be established, implemented and maintained covering the activities referenced below:

1. The applicable procedures recommended in Appendix A of Safety Guide 33, November, 1972.

2. Defueling activities.

3. Surveillance and test activities of safety-related equipment.

4. Security Plan implementation.

5. Emergency Plan implementation.

6. Process Control Program (PCP) implementation.

and surveillance, in-service inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totaling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the whole body dose received from external sources shall be assigned to specific major work functions.

c. Monthly Operating Report

The requirement for this report has been deleted.

d. Annual Radiological Environmental Monitoring Report

A report on the Radiological Environmental Monitoring Program for the previous calendar year shall be submitted to the Nuclear Regulatory Commission in accordance with 10 CFR 50.4 as a separate document by May 1 of each year.

prepared and submitted to the Nuclear Regulatory Commission in the Annual Radiological Environmental Monitoring Report.

e. Semi-annual Radioactive Effluent Release Report

Routine Radioactive Effluent Release Reports covering the activities of the unit during the previous six months shall be submitted within 60 days after January 1 and July 1 of each year.

The Radioactive Effluent Release Reports shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit as outlined in Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants," Revision 1, June, 1974, with data summarized on a quarterly basis following the format of Appendix B thereof.

An annual summary of hourly meteorological data collected over the previous year shall be maintained for five years by the licensee.

7.8 HIGH RADIATION AREA

7.8.1 Pursuant to 10 CFR 20, in lieu of the "control device" or "alarm signal", each high radiation area, as defined in 10 CFR Part 20, shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Health Physics personnel) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mR/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

a. A radiation monitoring device which continuously indicates the radiation dose rate in the area, or

b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them, or

c. A health physics qualified individual (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the facility Health Physics staff in the RWP.

7.8.2 In addition to the requirements of 7.8.1, areas accessible to personnel with radiation levels greater than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be provided with locked enclosures to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Shift Supervisor on duty and/or health physics supervision. Enclosures shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area. In lieu of the stay time specification of the RWP, direct or remote (such as use of closed circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area.

For individual areas accessible to personnel with radiation levels of greater than 1000 mR/h that are located within large areas, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be barricaded, conspicuously posted and a flashing light shall be activated as a warning device whenever the dose rate in the area exceeds or will shortly exceed 1000 mR/h.