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Nuclear Department

JUN 03 1992

NLR-N92076

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

RETEST SCHEDULE FOR DRYWELL TO SUPPRESSION CHAMBER
VACUUM BREAKERS PER TECHNICAL SPECIFICATION 4.6.2.1.g
HOPE CREEK GENERATING STATION
DOCKET NO. 50-354

On May 26, 1992, at 1000 hours, Hope Creek Generating Station (HCGS) entered Limiting Condition for Operation (LCO) 3.6.1.1 because the Surveillance Requirements of 4.6.2.1.g could not be met. The plant was subsequently brought to Hot Shutdown at 2215 hours on that date.

Inspection of the eight drywell to suppression chamber vacuum breakers revealed that three of the valves had indication of leakage at the valve seating areas. The cause of this leakage was a loosening of the seat bolting ring hardware. The loosening, caused by loss of torque on the nuts, is suspected to be the result of frequent cycling of the valves required by TS 4.6.4.1.b and valve cycling that occurs during nitrogen make-up to the drywell/torus. Details of the inspections and specific corrective actions will be provided in a Licensee Event Report due on June 25, 1992.

In accordance with the Surveillance Requirements of TS 4.6.2.1.g and discussion with Mr. S. Dembek - NRC Licensing Project Manager, Public Service Electric and Gas Company (PSE&G) provides the following proposed test schedule for Commission review and approval.

Prior to start-up from the forced outage, TS 4.6.2.1.g was successfully performed on May 30, 1992. HCGS is scheduled to commence its fourth refueling outage on September 12, 1992. Prior to start-up from that refueling outage, HCGS will again perform Surveillance Requirement 4.6.2.1.g, after which the eighteen month interval between tests will be re-established.

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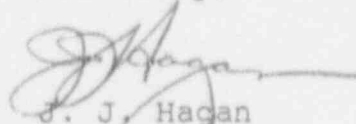
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PSE&G is confident that this schedule is justified based on good past performance of these valves. The TS surveillance has been successfully performed during the previous three eighteen-month surveillances (approximately 54 months).

Although the inspections performed revealed no abnormal degradation of the valves seats, they were replaced with new material as a precautionary measure. The other five drywell to torus vacuum breaker inspection results were satisfactory including a leak check and check on the tightness on the sea-bolting ring hardware.

If you have any questions on this submittal, please do not hesitate to contact us.

Sincerely,



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General Manager -
Hope Creek Operations

C Ms. A. Keller
Licensing Project Manager

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