



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

June 3, 1992

Docket 50-341

Mr. William S. Orser  
Senior Vice President - Nuclear  
Operations  
Detroit Edison Company  
6400 North Dixie Highway  
Newport, Michigan 48166

Dear Mr. Orser:

SUBJECT: RESPONSE TO ISSUES RAISED BY THE ADVISORY COMMITTEE ON REACTOR  
SAFEGUARDS (ACRS) REGARDING FERMI-2 POWER UPRATE SUBMITTAL  
(TAC NO. M82102)

During the presentations by GE and DECO representatives and the NRC staff to the ACRS Subcommittee on Thermal Hydraulic Phenomena, the ACRS asked a number of questions regarding both the generic BWR power uprate program and the Fermi-2 plant-specific uprate amendment request. By separate letter, the staff has requested that GE provide additional information to the ACRS regarding the generic BWR power uprate program. Additionally, we understand that your response to a staff Request for Additional Information (RAI) addresses a number of these questions.

Following is a list of the outstanding technical questions pertaining to the Fermi-2 power uprate amendment request, which were raised during the ACRS Subcommittee meeting. Please provide responses to these questions, or indicate the source of this information, if provided under separate cover.

- (1) Discuss the impact of increased reactor operating pressure on HPCI and RCIC reliability.
- (2) Explain the impact of power uprate on the minimum water level experienced during a loss-of-feedwater (LOFW) transient. (GE is providing a generic response to this question.)
- (3) Describe the effect of the flattening of the core radial power distribution on core void fraction, core thermal margins, and stability. (GE is providing a generic response to this question.)
- (4) Discuss the impact of power uprate on operator action times during transient and accident conditions. For example, how does the decrease in minimum water level associated with the LOFW transient affect the time available for operator actions to mitigate further failures?
- (5) Describe the effect of power uprate (flatter radial power distribution, increased decay heat production) on the acceptability of the core spray system to adequately cool the core in the event of a loss-of-coolant accident (LOCA).

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Mr. William S. Orser

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This request for information affects fewer than ten respondents; therefore, OMB clearance is not required under P. L. 96-511.

Please respond to these questions within 30 days of receipt of this letter. Your response will be forwarded by the staff to the ACRS for their review. If you have any questions regarding this letter, please contact me at (301) 504-1341

Sincerely,

Original signed by

Timothy G. Colburn, Sr. Project Manager  
Project Directorate III-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

cc: See next page

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Mr. William Orser  
Detroit Edison Company

Fermi-2

cc:

John Flynn, Esquire  
Senior Attorney  
Detroit Edison Company  
2000 Second Avenue  
Detroit, Michigan 48226

Nuclear Facilities and Environmental  
Monitoring Section Office  
Division of Radiological Health  
Department of Public Health  
3423 N. Logan Street  
P. O. Box 30195  
Lansing, Michigan 48909

Mr. Stan Stasek  
U.S. Nuclear Regulatory Commission  
Resident Inspector Office  
6450 W. Dixie Highway  
Newport, Michigan 48166

Monroe County Office of Civil  
Preparedness  
963 South Raisinville  
Monroe, Michigan 48161

Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Mr. A. Cecil Settles  
Director - Nuclear Licensing  
Detroit Edison Company  
Fermi 2  
6400 North Dixie Highway  
Newport, Michigan 48166