

LICENSEE EVENT REPORT

Updated report. Previous report dated 10/4/83.

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | I | L | D | R | S | 3 | 2 | d | d | - | d | o | o | o | o | - | o | o | 3 | 4 | 1 | 1 | 1 | 1 | 4 | _____ | 5
7 8 9 14 15 25 26 30 57 CAT 58

CON'T
01 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 2 | 4 | 9 | 7 | 1 | 0 | 0 | 4 | 8 | 3 | 8 | 0 | 9 | 0 | 4 | 8 | 4 | 9
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
02 | During Unit 3 drywell inspection, water was observed leaking from the "A" recirc
03 | pump bowl drain. During normal unit operation prior to shutdown for refueling
04 | no significant increase was noted in drywell leakage. All leakage was contained
05 | within the drywell. Floor and equipment drain leakage was within Tech Spec
06 | limits. The event had no effect on public health and safety as all leakage
07 | was contained in the drywell. The last similar previous occurrence was reported
08 | on RO 82-19 on Docket #050-249.

09 | SYSTEM CODE | C | B | 11 | CAUSE CODE | A | 12 | CAUSE SUBCODE | E | 13 | COMPONENT CODE | P | I | P | E | X | X | 14 | COMP. SUBCODE | A | 15 | VALVE SUBCODE | Z | 16
9 10 11 12 13 18 19 20
17 | LER/RO REPORT NUMBER | 8 | 3 | EVENT YEAR | 8 | 3 | SEQUENTIAL REPORT NO. | 0 | 3 | 5 | OCCURRENCE CODE | 0 | 3 | REPORT TYPE | X | REVISION NO. | 1
21 22 23 24 25 26 27 28 29 30 31 32
ACTION TAKEN | A | 18 | FUTURE ACTION | Z | 19 | EFFECT ON PLANT | Z | 20 | SHUTDOWN METHOD | Z | 21 | HOURS | 0 | 0 | 0 | 0 | 22 | ATTACHMENT SUBMITTED | N | 23 | NPRD-4 FORM SUB. | N | 24 | PRIME COMP. SUPPLIER | N | 25 | COMPONENT MANUFACTURER | G | 0 | 8 | 0 | 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)
10 | A metallurgical investigation was conducted to determine the cause of leakage
11 | in the socket weld taken from the "A" recirc pump bowl drain line. Based on the
12 | examination, the most probable cause for the leak was the result of the socket
13 | weld joint bottoming out. There was no evidence of a fatigue induced failure,
14 | or of intergranular stress corrosion cracking. According to Special (OVER)

15 | FACILITY STATUS | H | 28 | % POWER | 0 | 0 | 0 | 29 | OTHER STATUS | N/A | 30 | METHOD OF DISCOVERY | B | 31 | DISCOVERY DESCRIPTION | Inspection | 32
7 8 9 10 11 12 13 44 45 46

16 | ACTIVITY CONTENT | Z | 33 | RELEASED OF RELEASE | Z | 34 | AMOUNT OF ACTIVITY | N/A | 35 | LOCATION OF RELEASE | N/A | 36
7 8 9 10 11 44 45

17 | PERSONNEL EXPOSURES NUMBER | 0 | 0 | 0 | 37 | TYPE | Z | 38 | DESCRIPTION | N/A | 39
7 8 9 10 11 12 13

18 | PERSONNEL INJURIES NUMBER | 0 | 0 | 0 | 40 | DESCRIPTION | N/A | 41
7 8 9 10 11 12

19 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 | DESCRIPTION | N/A | 43
7 8 9 10

20 | PUBLICITY ISSUED | N | 44 | DESCRIPTION | N/A | 45
7 8 9 10

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PDR ADOCK 05000249
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11

NAME OF PREPARER S. Merritt PHONE (815) 942-2920 x421

Cause Description and Corrective Actions (Cont'd)

Process Procedures Manual section on general welding requirement for pipe socket welds, it states that prior to welding a socket weld there should be a minimum gap of 1/16" between the pipe and the socket to allow for weld shrinkage which will pull the pipe down into the socket. In this particular case, there was no clearance allowed at the joint, thus the drain line cracked at the socket weld. During the Unit 3 outage, the affected piping was removed and replaced with new low carbon stainless steel piping. The isolation valves were also removed and replaced. The welding was performed according to proper procedures. The original welding was performed during the original construction phase of the plant. Since the piping has been replaced, no further action is required,



Commonwealth Edison
Dresden Nuclear Power Station
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Telephone 815/942-2920

September 4, 1984

DJS Ltr #84-869

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Updated Reportable Occurrence Report #83-35/01X-1, Docket #050-249 is being submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.1.(c), abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment. This update is being submitted to provide the exact cause of the "A" recirc pump bowl drain line leakage.

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

IE22
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SUPPLEMENT TO DVR

| | | | | |
|---------|-----|------|------|------|
| DVR NO. | STA | UNIT | YEAR | NO. |
| D - 12 | - | 3 | - 83 | - 70 |

| | | | |
|---|--|-------------------|---------------|
| PART 1 | TITLE OF EVENT | OCCURRED | |
| | | 10/4/83 | 1200 |
| | Leakage from "A" Recirc Pump Bowl Drain Line | <u>DATE</u> | <u>TIME</u> |
| | REASON FOR SUPPLEMENTAL REPORT | | |
| | To provide exact cause of leakage | | |
| | | | |
| | | | |
| PART 2 | | | |
| ACCEPTANCE BY STATION REVIEW | <u>J. Brunner</u> | <u>J.A. Cusla</u> | _____ |
| DATE | <u>9/4/84</u> | <u>9/5/84</u> | _____ |
| SUPPLEMENTAL REPORT APPROVED AND AUTHORIZED FOR DISTRIBUTION | <u>Douglas Scott</u> | | <u>9/6/84</u> |
| | STATION SUPERINTENDENT | | DATE |