

The Light company

Houston Lighting & Power South Texas Project Electric Generating Station P. O. Box 289 Wadsworth, Texas 77483

June 5, 1992
ST-HL-AE-4113
File No.: G26
10CFR50.73

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

South Texas Project
Unit 2
Docket No. STN 50-499
Licensee Event Report 92-005
Containment Ventilation Isolation Actuation Due to a
Failure in the RM-023A Module

Pursuant to 10CFR50.73, Houston Lighting & Power Company (HL&P) submits the attached Licensee Event Report 92-005 regarding a Containment Ventilation Isolation Actuation due to a failure in the RM-023A module. This event did not result in an adverse impact on the health and safety of the public.

If you should have any questions on this matter, please contact Mr. C. A. Ayala at (512) 972-8628 or me at (512) 972-7205.

William J. Jump
William J. Jump
Manager,
Nuclear Licensing

MKJ/lf

Attachment: LER 92-005 (South Texas, Unit 2)

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A Subsidiary of Houston Industries Incorporated

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Houston Lighting & Power Company
South Texas Project Electric Generating Station

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Revised 10/11/91

L4/NRC/

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P.530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555 AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1) **South Texas, Unit 2** DOCKET NUMBER (2) **050004991** PAGE (3) **03**

TITLE (4) **Containment Ventilation Isolation Actuation Due to a Failure in the RM-023A Module.**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
05	08	92	92	005	00	06	05	92			050000
											050000

OPERATING MODE (9) **1** THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

POWER LEVEL (10) 100	20.402(b)	20.405(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
	20.405(a)(1)(i)	50.36(c)(1)		50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)		50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 386A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(vii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME **Charles Ayala - Supervising Licensing Engineer** TELEPHONE NUMBER **512972-8628**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPRDS
X	I L M O N	G 0 6 3		YES					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15) **091092**

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On May 8, 1992, Unit 2 was in Mode 1 at 100% power. At approximately 324 hours a Containment Ventilation Isolation (CVI) actuation occurred. Operations personnel verified that all equipment actuated as designed. The radiation monitoring system did not indicate any high radiation conditions. The Containment Ventilation Isolation actuation appears to be the result of an equipment failure in a radiation monitoring RM-023A module. Troubleshooting of the suspect RM-023A module and maintenance history evaluations are being performed.

LICENSEE EVENT REPORT (LER)
 TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) South Texas, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 4 9 9 9 2 - 0 0 5 - 0 0 0 2 OF 0 3	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT:

On May 8, 1992, Unit 2 was in Mode 1 at 100% power. At approximately 1324 hours a Containment Ventilation Isolation (CVI) actuation occurred. Operations personnel verified that all equipment actuated as designed. The NRC was notified pursuant to 10CFR50.72 at 1445 hours.

Technical Specification related radiation monitors have control and display functions contained within individual RM-023 modules located in the control room ZCP-023 console. Each RM-023 module is a microprocessor based unit that processes data from an individual radiation monitor. The RM-023 module generates the actuation signal for radiation monitors associated with Engineered Safety Features.

Technicians were at the radiation monitor panel (ZCP-023) performing an operability test on one of the two Spent Fuel Pool Exhaust Monitors (RT-8035) for the Fuel Handling Building. The technicians were in the process of completing data sheets for RT-8035 when the Shift Supervisor informed them that RT-8012's display was blank, the error light was flashing and a CVI actuation had occurred. RT-8012 is one of the two radiation monitors for the Containment Purge System. Although the RM-023 modules for RT-8035 and RT-8012 are located in the same cabinet, they are physically and electronically independent. The error light is an indication of a loss of communications to the RM-023A module. Diagnostics testing on the RT-8012 RM-023A module was unsuccessful in that 99% of the display returned and remained on. The RM-023A module associated with the RT-8012 monitor was replaced. The redundant radiation monitor RT-8013 was verified to have normal radiation levels.

The suspect RM-023A module is being tested in the plant shop by Maintenance technicians.

CAUSE OF EVENT:

The Engineered Safety Features Containment Ventilation Isolation actuation appears to be the result of an equipment failure in the RM-023A module. A definite cause of the equipment failure has not been established.

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LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) South Texas, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 4 9 9	LER NUMBER (8)			PAGE (3)		
		YEAR 9 2	SEQUENTIAL NUMBER — 0 0 5	REVISION NUMBER — 0 0 0	3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

ANALYSIS OF EVENT:

Unplanned actuation of an Engineered Safety Feature is reportable pursuant to 10CFR50.73(a)(2)(iv). All ESF equipment actuated as designed. No evidence of high radiation was found. While any unnecessary challenge to an Engineered Safety Feature is undesirable, actuation of Containment Ventilation Isolation represents a minimal hazard since it could not cause, worsen, or prevent mitigation of any accident.

CORRECTIVE ACTIONS:

The following corrective actions are being taken as a result of this event:

1. The RM-023 module associated with the RT-8012 monitor was replaced.
2. Troubleshooting of the RT-8012 RM-023A module to determine a cause for the failure will be completed by July 23, 1992.
3. Maintenance history of the RM-023 modules will be reviewed and evaluated for common failures by August 6, 1992.

A supplemental report documenting the results of these evaluations will be provided by September 10, 1992.

ADDITIONAL INFORMATION:

The radiation monitor which caused this event was manufactured by General Atomics.

Several other events have been documented in regards to Engineered Safety Features actuation as a result of spurious actuations of the radiation monitoring system. There have been no ESF actuations in either unit, with the exception of the Reactor Protection System as a result of reactor trips, since July 7, 1991.

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