NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Susquehanna Steam Electric Station - Unit 2	0 5 0 0 0 3 8 8	1 OF 0 2

TITLE (4)
RWCU Isolation Caused By Differential Pressure Instrument Drift.

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
												0 5 0 0 0
0 8	0 2	8 4	8 4	–	0 1 5	–	0 0	0 8	3 1	8 4		0 5 0 0 0

OPERATING MODE (9)	2	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)					
POWER LEVEL (10)	0 1 0 6	20.402(b)	20.405(c)	X	50.73(a)(2)(iv)		73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
		20.405(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
		20.405(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Benjamin L. Wilks	7 1 1 7 5 4 2 - 1 3 9 1 1 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	C E	P D S	B 0 8 0	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 1840 hours on 8-2-84, the Reactor Water Cleanup (RWCU) system isolated on high flow. The existance of no other abnormal alarms or indication at the time of the isolation indicated a pipe break had not occurred as was confirmed by an immediate walk down of the RWCU piping. Following the isolation, sampling of the reactor coolant conductivity was initiated in compliance with Technical Specification Section 4.4.4(c). Further investigations revealed that one of two switches independently used for RWCU high flow trip and system isolation, PDIS-G33-2N044A, was out of calibration due to instrument drift. PDIS-G33-2N044A was recalibrated, and returned to service. The RWCU system was returned to service at 0400 hours on 8-3-84. The performance of Differential Pressure instrument PDIS-G33-2N044A will be evaluated following future testing.

Isolation of the RWCU system is an Engineered Safety Feature (ESF) actuation due to the closure of the system's Containment Isolation Valve. The occurrence of this event had no adverse effects on the health and safety of the public.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Susquehanna Steam Electric Station Unit 2	0 5 0 0 0 3 8 8	8 4	— 0 1 5	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1840 hours on 8-2-84, the Reactor Water Cleanup (RWCU) system isolated on high flow. (Flows above normal in the RWCU system could only be expected in the event of a leak or pipe break; the consequences of which are minimized by isolation of the RWCU influent piping inside the primary containment.) The existance of no other abnormal alarms or indication at the time of the isolation indicated a pipe break had not occurred, as was confirmed by an immediate walk-down of the RWCU system piping. Following the isolation, sampling of the reactor coolant conductivity was initiated in accordance with the surveillance requirements of Technical Specification Section 4.4.4.(c). Further investigations revealed switch PDIS-G33-2N044A (Barton Model 289) was responsible for the RWCU system high flow trip and closure of Inboard Isolation Valve F001. The switch, one of two independently used for sensing high flow conditions (and system isolation) in the RWCU system influent piping in containment, was reading 5.0 inches of water column with the system isolated and 7.8 inches or a flow of 234 gpm, with the system operating. The trip setpoint for this instrument is 415 gpm which corresponds to 11.26 inches of water. Since PDIS-G33-2N044A was left with a calibration setpoint of 11.26 ±.24 inches following completion of maintenance activities on the RWCU High Flow Channel 'A' in May, 1984, the isolation of the RWCU on high system flow was due to setpoint drift. Furthermore, a review of previous occurrences showed this event to be an isolated occurrence. PDIS-G33-2N044A was recalibrated, and returned to service. The RWCU system was returned to service at 0400 hours on 8-3-84. The performance of Differential Pressure instrument PDIS-G33-N044A will be evaluated following future testing.

Isolation of the RWCU system is an ESF actuation due to the closure of the system's containment isolation valve. This occurrence caused no adverse effects on the health and safety of the public.

PP&L Pennsylvania Power & Light Company
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August 31, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
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SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 84-015-00
ER 100450 FILE 841-23
PLA-2295

Docket No. 50-388
License No. NPF-21

Attached is Licensee Event Report 84-015-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that an unplanned Engineered Safety Feature (ESF) actuation occurred due to the setpoint drift in a differential pressure instrument. This resulted in the isolation of the Reactor Water Cleanup (RWCU) system on indicated high system flow.

H.W. Keiser
Superintendent of Plant - Susquehanna

BLW/pjg

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