

L I C E N S E E E V E N T R E P O R T (L E R)

FACILITY NAME (1) Arkansas Nuclear One, Unit Two DOCKET NUMBER (2) | PAGE (3)
 |0151010101 3| 6| 8|110F1011

TITLE (4) Reactor Trip Caused by High Steam Generator Level

EVENT DATE (5) | LER NUMBER (6) | REPORT DATE (7) | OTHER FACILITIES INVOLVED (8)
 Month | Day | Year | Sequential | Revision | Month | Day | Year | Facility Names | Docket Number(s)
 01 | 7 | 21 | 8 | 3 | 4 | 8 | 4 | -- | 0 | 2 | 1 | -- | 0 | 0 | 0 | 8 | 21 | 9 | 8 | 4 | | | 0151010101 | |
 | | | | | | | | | | | | | | | | | | | | | | | | | | 0151010101 | |

OPERATING MODE (9) | 2 | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §:
 (Check one or more of the following) (11)

POWER LEVEL (10) | 0 | 0 | 15 | | 20.402(b) | | 20.405(c) | | X | 50.73(a)(2)(iv) | | 73.71(b)
 | | | | | 20.405(a)(1)(i) | | 50.36(c)(1) | | | 50.73(a)(2)(v) | | 73.71(c)
 | | | | | 20.405(a)(1)(ii) | | 50.36(c)(2) | | | 50.73(a)(2)(vii) | | Other (Specify in
 | | | | | 20.405(a)(1)(iii) | | 50.73(a)(2)(i) | | | 50.73(a)(2)(viii)(A) | | Abstract below and
 | | | | | 20.405(a)(1)(iv) | | 50.73(a)(2)(ii) | | | 50.73(a)(2)(viii)(B) | | in Text, NRC Form
 | | | | | 20.405(a)(1)(v) | | 50.73(a)(2)(iii) | | | 50.73(a)(2)(x) | | 366A)

Name | Patrick Rogers, Plant Licensing Engineer | Telephone Number
 | | | Area |
 | | | Code |
 | | | 510 | 191614 | -1311010

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

Cause	System	Component	Manufacturer	Reportable to NPRDS	Cause	System	Component	Manufacturer	Reportable to NPRDS

SUPPLEMENT REPORT EXPECTED (14) | EXPECTED SUBMISSION DATE (15) | Month | Day | Year
 Yes (If yes, complete Expected Submission Date) | No

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 7/28/84, at 0013 hours while in Mode 2 at 5% full power (FP), an automatic reactor trip occurred due to a high level on the 'B' steam generator (2E24B). Feedwater control was in manual and preparations were being made to synchronize the turbine to the grid. Main feedwater flow had been increased in response to decreasing steam generator level. This resulted in a rapid increase in steam generator levels such that, before the level increase could be turned, 2E24B reached the high level trip setpoint. Automatic feedwater control at low power levels is not feasible with the Feedwater Control System, and manual control is difficult at low power due to the coarse controls provided. No problems were experienced as a result of the trip, and no post-trip anomalies were noted. Similar to LERs (50-368) 84-004, 84-008, 84-011, 84-014 and 84-020.



ARKANSAS POWER & LIGHT COMPANY
POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000
August 29, 1984

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Licensee Event Report
No. 84-021-00

Gentlemen:

In accordance with 10CFR50.73(a)(2)(iv), attached is the subject report concerning an automatic reactor trip due to a high level "B" steam generator (2E24B).

Very truly yours,

John R. Marshall
Manager, Licensing

JRM:RJS:ac

Attachment

cc: Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Mr. Norman M. Haller, Director
Office of Management & Program Analysis
U. S. Nuclear Regulatory Commission
Washington, DC 20555

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