UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

BOSTON EDISON COMPANY

DOCKET NO. 50-293

PILGRIM NUCLEAR POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 78
License No. DPR-35

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Boston Edison Company (the licensee) dated December 28, 1983, as supplemented February 21, and July 12, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-35 is hereby amended to read as follows:

B Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 78 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 4, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 78

FACILITY OPERATING LICENSE NO. DPR-35

DOCKET NO. 50-293

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	Insert
205B-2	205B-2
205E-4	205E-4
205E-5	205E-5
205E-6	205E-6
206m	206m

TABLE 3.11-1
OPERATING LIMIT MCPR VALUES

A. MCPR Operating Limit from Beginning of Cycle (BOC) to BOC + 6000 MWD/T.

	8 x 8	P8 x 8R/BP8x8R
For all values of τ	1.36	1.40

B. MCPR Operating Limit from BOC + 6000 MWD/T to End of Cycle.

τ	8 x 8	P8x8R/BP8x8R
$\tau \leq 0$	1.38	1.40
$0.0 < \tau \leq 0.1$	1.39	1.41
$0.1 < \tau \leq 0.2$	1.39	1.41
$0.2 < \tau \leq 0.3$	1.40	1.42
$0.3 < \tau \leq 0.4$	1.40	1.42
$0.4 < \tau \leq 0.5$	1.41	1.43
$0.5 < \tau \leq 0.6$	1.41	1.43
$0.6 < \tau \leq 0.7$	1.42	1.44
$0.7 < \tau \leq 0.8$	1.42	1.44
$0.8 < \tau \leq 0.9$	1.43	1.45
$0.9 < \tau \leq 1.0$	1.43	1.45

FIGURE 3.11-4
MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE VERSUS PLANAR AVERAGE EXPOSURE

FUEL TYPES P8DRB265L and BPDRB265L

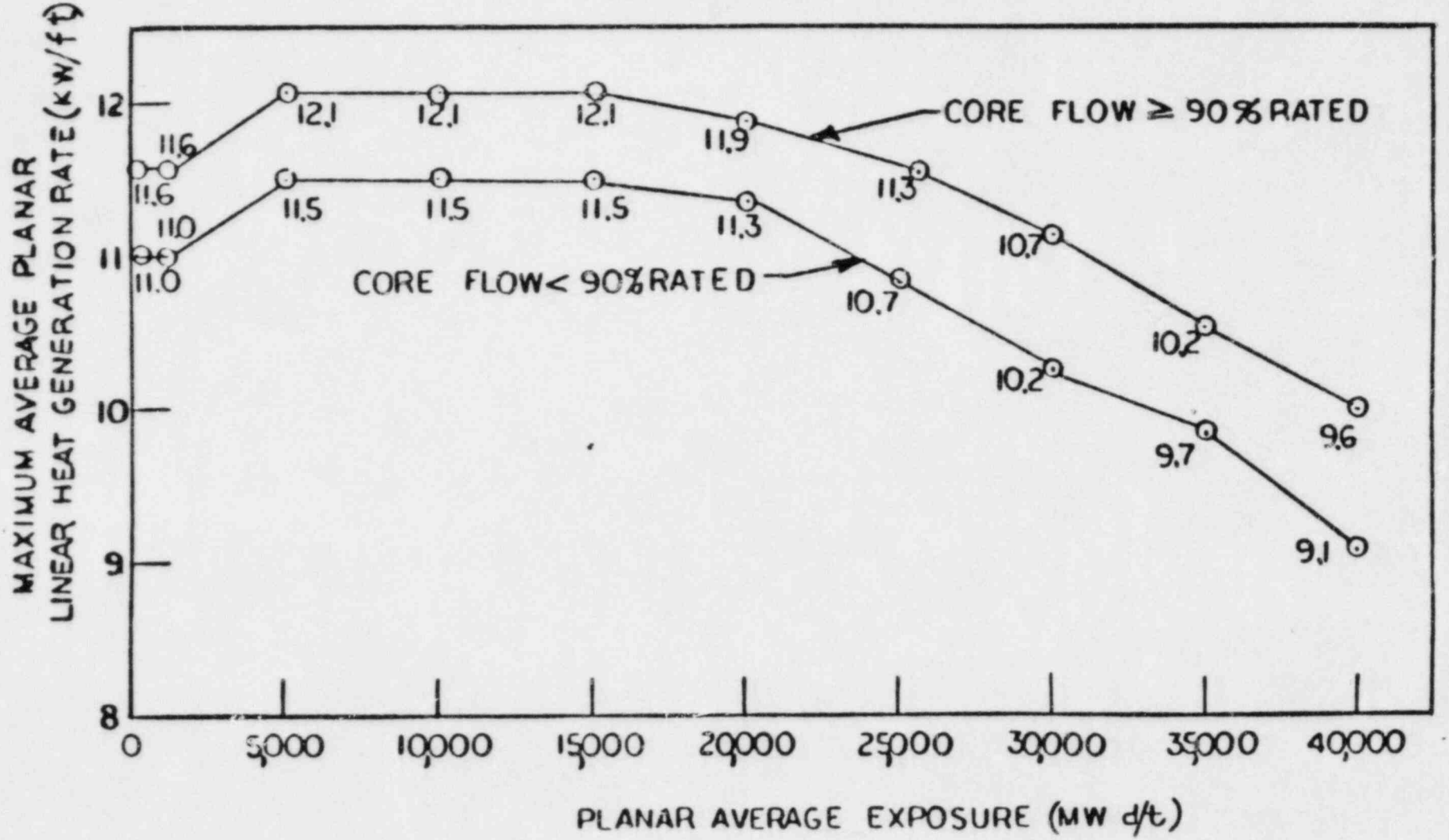


FIGURE 3.11-5
MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE VERSUS PLANAR AVERAGE EXPOSURE

FUEL TYPES P8DRB282 and BP8DRB282

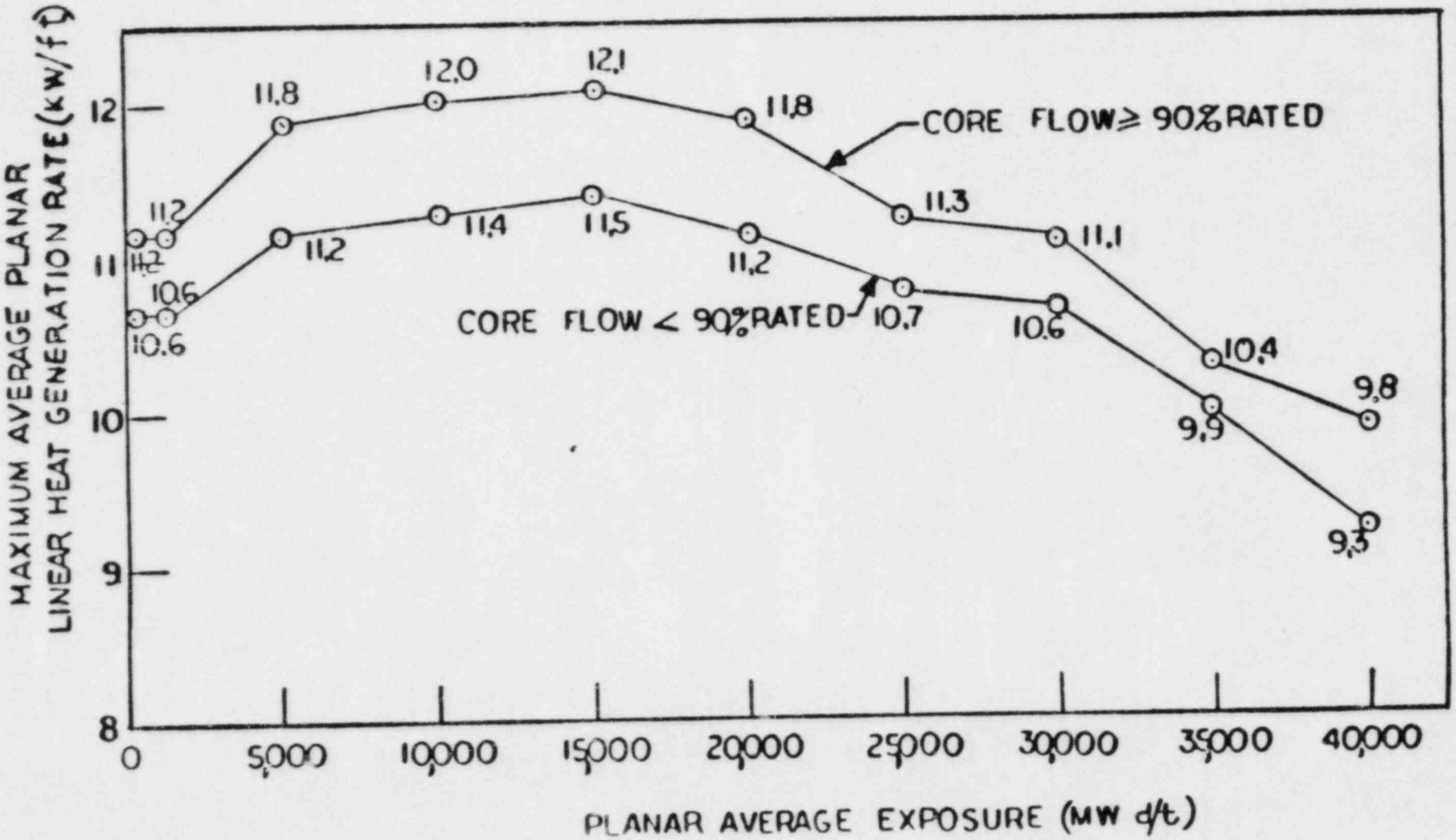
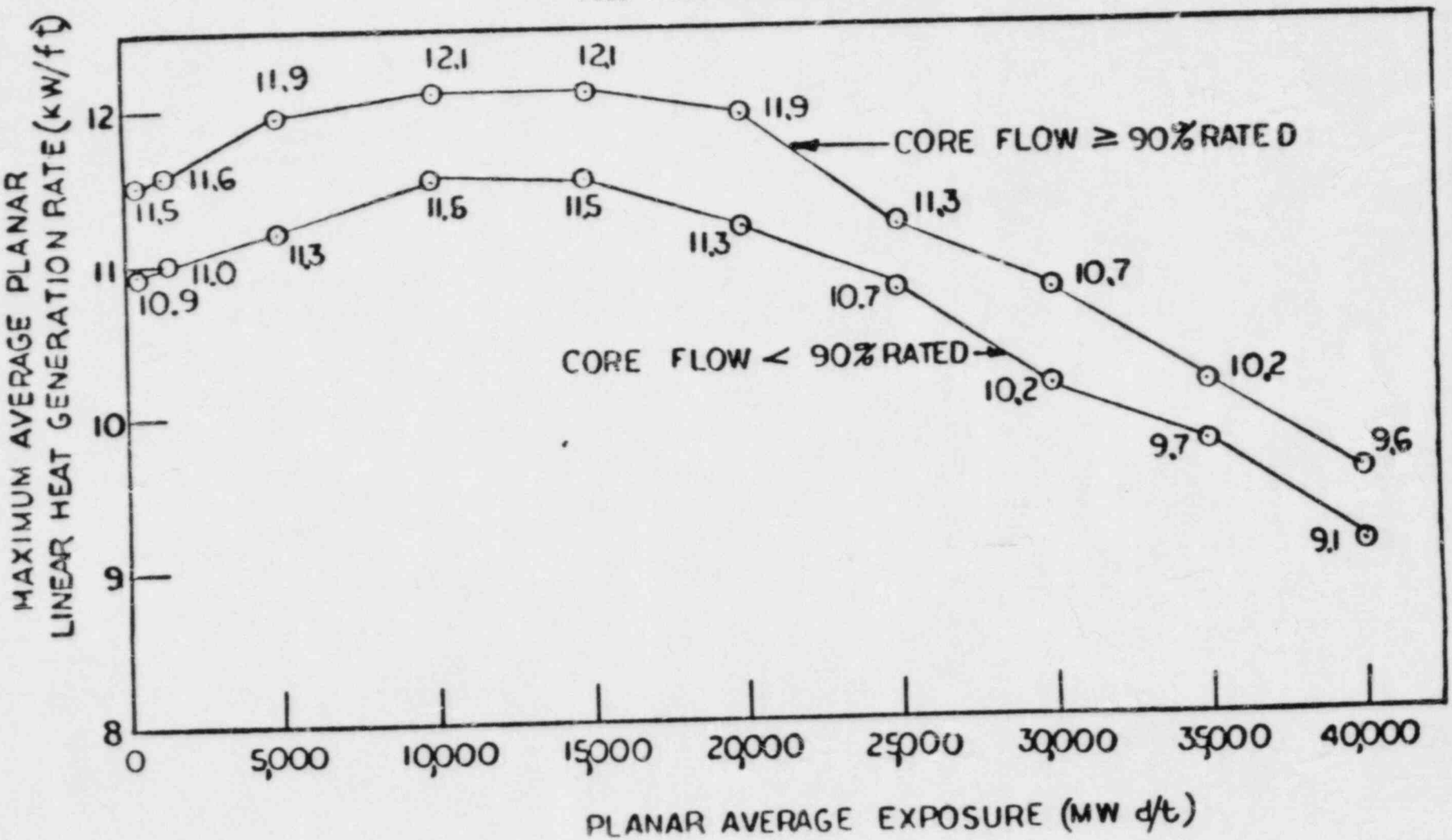


FIGURE 3.11-6
MAXIMUM AVERAGE PLANAR LINEAR HEAT GENERATION RATE VERSUS PLANAR AVERAGE EXPOSURE

FUEL TYPES P8DRB265H and BP8DRB265H



5.0 MAJOR DESIGN FEATURES

5.1 SITE FEATURES

Pilgrim Nuclear Power Station is located on the Western Shore of Cape Cod Bay in the Town of Plymouth, Plymouth County, Massachusetts. The site is located at approximately 41°51' north latitude and 70°35' west longitude on the Manomet Quadrangle, Massachusetts, Plymouth County 7.5 Minute Series (topographic) map issued by U.S. Geological Survey. UTM coordinates are 19-46446N-3692E.

The reactor (center line) is located approximately 1800 feet from the nearest property boundary.

5.2 REACTOR

A. The core shall consist of not more than 580 fuel assemblies of 8x8 (63 fuel rods), P8x8R (62 fuel rods), and BP8x8R (62 fuel rods).

B. The reactor core shall contain 145 cruciform-shaped control rods. The control material shall be boron carbide powder (B_4C) compacted to approximately 70% of theoretical density.

5.3 REACTOR VESSEL

The reactor vessel shall be as described in Table 4.2.2 of the FSAR. The applicable design codes shall be as described in Table 4.2.1 of the FSAR.

5.4 CONTAINMENT

A. The principal design parameters for the primary containment shall be as given in Table 5.2.1 of the FSAR. The applicable design codes shall be as described in Section 12.2.2.8 of the FSAR.

B. The secondary containment shall be as described in Section 5.3.2 of the FSAR.

C. Penetrations to the primary containment and piping passing through such penetrations shall be designed in accordance with standards set forth in Section 5.2.3.4 of the FSAR.