

U. S. NUCLEAR REGULATORY COMMISSION

REGION III

Reports No. 50-454/84-42(DRP); 50-455/84-29(DRP)

Docket Nos. 50-454; 50-455

Licenses No. CPPR-130; CPPR-131

Licensee: Commonwealth Edison Company
Post Office Box 767
Chicago, IL 60690

Facility Name: Byron Station, Units 1 and 2

Inspection At: Byron Station, Byron, IL

Inspection Conducted: June 2 - July 31, 1984

Inspectors: J. M. Hinds, Jr.

P. G. Brochman

L. G. McGregor

Approved By: *R. LERCH FOR*
D. W. Hayes, Chief
Reactor Projects Section 1B

R. M. Lerch
Date

Inspection Summary

Inspection on June 2 - July 31, 1984 (Reports No. 50-454/84-42(DRP);
50-455/84-29(DRP))

Areas Inspected: Routine, unannounced safety inspection of licensee action on previously identified items; 10 CFR 50.55(e) Reports; IE Bulletins; Operational Staffing; Inservice Surveillance Procedure Review; Inservice Surveillance Procedure Performance; Administration and Maintenance Procedure Review; Plant Tours/ Housekeeping and other activities. The inspection consisted of 223 inspector-hours onsite by three NRC inspectors including 35 inspector-hours during off-shifts.

Results: No items of noncompliance or deviations were identified.

8409130507 840821
PDR ADOCK 05000454
G PDR

DETAILS

1. Persons Contacted

Commonwealth Edison Company

- V. I. Schlosser, Project Manager
- R. Tuetkin, Startup Coordinator
- *R. Querio, Station Superintendent
- M. Loehman, Project Construction Assistant Superintendent
- R. Klingler, Project Construction Quality Control Supervisor
- R. Ward, Assistant Superintendent, Administrative & Support Services
- L. Sues, Assistant Superintendent, Maintenance
- T. Joyce, Operating Shift Oversight Superintendent
- F. Hornbeak, Technical Staff Supervisor
- W. Dean, Assistant Technical Staff Supervisor, Licensing
- E. Grennan, Licensing Staff
- J. Poche, Licensing Staff
- S. Devine, Electrical Group Leader, Technical Staff
- B. Milner, Primary Group Leader, Technical Staff
- R. Flahive, Assistant Technical Staff Supervisor
- K. Hansing, Quality Assurance Site Superintendent
- *L. M. Johnson, QA Engineer
- *W. Burkamper, QA Supervisor (Operations)
- *P. Anthony, Tech Staff Engineer
- *D. E. St. Clair, Tech Staff Supervisor

The inspectors also contacted and interviewed other licensee and contractor personnel during the course of this inspection.

*Denotes those present during the exit interview on July 31, 1984.

2. Licensee Action on Previously Identified Items

- a. (Closed) Open Item 454/83-49-05; 455/83-35-05: Improper cable grip installation - Inspector review of licensee's evaluation of NCR F-852 and proposed corrective actions. The licensee determined that this item was reportable pursuant to 10 CFR 50.55(e) and on December 29, 1983, notified the Region III NRC office of the deficiency. The deficiency was entitled "Cable Grip Installation" and for tracking purposes was numbered 84-14. (454/84-14-EE; 455/84-14-EE)
- b. (Closed) Open Item 454/84-19-04; 455/84-14-04: Inspector review to determine the acceptability of programmed setpoint as well as instrument calibration and test procedures in eight of the Technical Specification values. The Westinghouse memorandum CAW-7271 dated April 24, 1983, reviewed the concerns related to the Cold Overpressure Mitigation System and stated that the methodology employed to determine the pressure setpoint corresponding to the highest temperature

setpoint specified for the "lowermost" Power Operated Relief Valve (PORV) is acceptable. The Precautions, Limitation and Setpoints document will be changed by Westinghouse to address the concerns. Based on a review of memorandum CAW-7271, Project Engineering Department recommended reducing the calibration tolerances of the affected instruments by 50% to prevent exceeding the maximum Technical Specification PORV setpoint curve (lower) to remain within the 10 CFR 50 Appendix G curve (upper). The station Instrument Maintenance department reviewed the latest calibration of the affected instruments and determined that the observed direct current values were within the specified tolerances.

- c. (Closed) Noncompliance 454/82-0-01: Radiographs that had penetrameters on the welds which are in the area to be interpreted. The inspector reviewed the final response dated May 3, 1984, CECo code inquiry to Subcommittee V of the ASME for interpretation, the ASME response, and the Code Case 1914 for alternate penetrameter placement. Code Case 1914 indicates that the placement of penetrameters on the weld is acceptable providing the lead penetrameter identifying numbers are not placed on the weld metal. The inspector agrees with the code case and considers the noncompliance closed.

3. 10 CFR 50.55(e) Reports

- a. (Closed) (454/79-04-EE; 455/79-04-EE): "Rod Drop Analysis". The Commonwealth Edison (CECo) letter, Tramm to Denton, dated December 6, 1983, stated that Westinghouse (W) advised the NRC in November 1979 of a deficiency in the rod drop analysis which had been performed for certain plants. CECo advised the NRC in December 1979 that this issue involved the Byron plant. The licensee's response indicated that NRR had completed the review of the W topical report (WCAP-10297(P)) on the revised dropped rod methodology for plants using a flux rate trip and also the revised Byron Final Safety Analysis Report (FSAR) Amendment 44 pages which incorporate the results of a reanalysis of rod drop transients using the revised methodology.
- b. (Closed) (454/81-04-EE; 455/81-04-EE): "Single random failure in Volume Control Tank (VCT) level control system could...lead to loss of redundancy in high head injection." The CECo letter, Tramm to Keppler, dated November 10, 1981, described the VCT system as incorporating information from two analog level instruments, provided tabulation of the various control and alarm functions together with W established setpoints, included the information provided to the operator in the event of failure of either level instrument, and indicated the amount of time which is available for protective action. CECo's response demonstrated that there is sufficient time for operator action in the event of a system malfunction to prevent loss of charging pump suction. No plant changes are required and required instructions will be incorporated in plant procedures.

- c. (Closed) (454/82-01-EE) (Open) (455/82-01-EE): "Cooper Energy Services diesel generator sets oil lube strainer malfunction." On April 23, 1982, Region III was notified that a defect exists in the Byron/Braidwood diesel generator lube oil strainers which is reportable pursuant to 10 CFR 50.55(e). Cooper Energy Services advised the licensee that the emergency diesel generators contain a defective strainer basket located in the lube oil strainers. Final report dated May 11, 1982, stated new basket strainers were designed and tested by the supplier and would be installed prior to fuel load.

The inspector reviewed the receiving inspection reports, installation documentation, applicable drawings and quality control inspections for the installation of the new lube oil strainers for the following diesel generators (DG): Unit 1; 1DG01KA and 1DG01KB. Unit 2; 2DG01KA. Because these DGs had operated during testing with the potentially defective lube oil strainers, the inspector visually examined the strainers after removal. No deterioration was observed and the deficient strainers were appropriately tagged to prevent reuse. This 50.55(e) is closed for Unit 1. The strainers on Unit 2 DG 2DG01UB had not been replaced to date and therefore 50.55(e) 455/82-01 will remain open until the replacement.

- d. (Open) (454/82-09-EE; 455/82-09-EE): "ITT Grinnel Figure 306/307 mechanical snubber assemblies." On December 3, 1982, Region III was notified of potential bracket interference which could interfere with pipe movement in certain snubber applications. Assemblies produced prior to April 1980 may have these deficiencies. Size 3 assemblies may have pipe clamp interference problems and size 35 assemblies may have end brackets which could interfere with the snubber. Final report dated June 2, 1983, stated the referenced snubbers were in process of being inspected for the potential interference problems and that approximately 3% had been inspected as of that date. Subsequent to that report the licensee received information from the supplier which technically justifies the reported worst case tolerances and conditions. The licensee intends to withdraw this deficiency report. The inspector requested the new final report be issued and the ITT Grinnell documentation justifying the withdrawal be furnished for review, prior to closure of this item. The revised final report has not yet been issued and the requested information has yet to be furnished, therefore, this item will remain open pending further review.

4. Inspection and Enforcement Bulletins (IEBs)

(Open) IEB (454/81-03-BB; 455/81-03-BB): "Flow blockage of cooling water to safety system components by Corbicula Sp. (Asiatic Clam) and Mytilus Sp. (Mussel)." This bulletin is reopened to review the licensee's program to monitor the Rock River at the river greenhouse forebay on an annual basis. The licensee was requested to provide the data of the last sampling survey for Corbicula Sp., the results of the survey, and procedures on administrative controls to implement and govern the required program to monitor, treat and antifoul any detected infestation.

5. Operational Staffing

- a. The inspector determined, by review of applicable Byron Final Safety Analysis Report (FSAR) sections and personnel resumes, that the staff positions have been filled with personnel possessing the ANSI 18.1 minimum education, experience or qualifications for the following positions:

Station Superintendent
Assistant Superintendent Operating
Assistant Superintendent Maintenance
Assistant Superintendent Administration and Support Services
Radiation Chemistry Supervisor
Training Coordinator

No items of noncompliance or deviations were identified.

- b. The inspector reviewed the operating organizational structures described in FSAR Chapter 13 and 16 against the Byron Station Organization Chart of June 30, 1984, to verify that the organization, as implemented, is in accord with the FSAR and draft Technical Specifications. The inspector identified the following discrepancies:

Station Control Room Engineers have in-line supervisory authority over Nuclear Station Operators, Equipment Operators and Equipment Attendants vice Shift Foremen. Additionally, the Fuel Handlers are responsible to the Fuel Foreman with in-line responsibility to the Shift Engineers.

Radiation Chemistry Technicians (RCTs) have in-line responsibility to both Health Physics Foremen and Chemistry Foremen.

Technical staff organization is missing the mid-level managers (Assistant Technical Staff Supervisors) and the Nuclear Group Leader organization is incorrectly described including the Lead Nuclear Engineer line authority as shown on the Station Organization Diagram.

The licensee has committed to revising the FSAR and Technical Specification organization descriptions and will incorporate these changes in the next amendment to the FSAR. This item will be followed as open item (454/84-42-01; 455/84-29-01) pending inspector review of the subject FSAR amendment.

6. Inservice Surveillance Procedure Review

The inspector reviewed the following inservice surveillance procedures and component demonstration test procedure against the FSAR, proposed Technical Specifications and the Boiler and Pressure Vessel Code, Section XI, Winter Addendum, 1981:

BVS 0.5-3.CC.1, Revision 0, "ASME Surveillance Requirements for Component Cooling Pumps"

BVS 5.2.f.3-1, Revision 0, "ASME Surveillance Requirements for Residual Heat Removal Pumps"

FH 2.32.11, Revision 2, "Operation of the New Fuel Elevator"

No items of noncompliance or deviations were identified.

7. Inservice Surveillance Procedure Performance

The inspector witnessed the performance of portions of the inservice surveillance procedure listed below in order to verify that the surveillance was conducted in accordance with approved procedures, to independently verify the acceptability of surveillance results, and to evaluate the performance of licensee personnel conducting the surveillance:

BVS 0.5-.CC.1, Revision 0, "ASME Surveillance Requirements for Component Cooling Pumps"

No items of noncompliance or deviations were identified.

8. Administration and Maintenance Procedure Review

a. General

The inspector's review included administrative, electrical, mechanical, and instrument procedures for Byron Units 1 and 2. The inspector verified that: responsibilities were assigned in writing to assure that the procedures will be reviewed, updated and approved as required (including 50.59 considerations); that controls have been established in writing to dispose of outdated procedures, issue temporary changes to procedures, assure that changes to procedures are approved and that procedures are consistent with the general guidance of ANSI N18.7-1972 and Regulatory Guide 1.33, Section 13.5 of the FSAR and Commonwealth Edison's Quality Assurance Program.

b. The following Administrative Procedures were selected for review:

BAP-100-02 Revision 0	Department Procedures
BAP-100-07 Revision 2	Overtime Guidelines for Personnel that Perform Safety Related Functions
BAP-100-10 Revision 0	Byron Station Chemical Safety Program
BAP-199-01 Revision 1	Byron Station Snow Emergency Organization
BAP-200-A1-6 Revision 1	Nuclear Station Instructor
BAP 200-A1-7 Revision 0	Training Department Skill, Knowledge and Responsibility Levels
BAP-300-02 Revision 5	Shift Manning
BAP-300-03 Revision 4	Daily Order Book

BAP-300-04 Revision 3	Operating Logs and Records
BAP-300-17 Revision 2	Operating Shift Turnover and Relief
BAP-300-18 Revision 5	Removing and Returning Equipment Out-Of Service
BAP-300-23 Revision 3	Operating Shift Turnover and Relief (After Fuel Load)

The inspector's comments and questions resulting from this review were provided to the licensee. All areas of concern were resolved. No items of noncompliance or deviations were identified.

c. The following Maintenance and Surveillance Procedures were selected for review:

(1) Electrical Maintenance Procedures

BHP-4200-002 Revision 2	Preventative Maintenance Inspection of 6.9KV and 4KV Breakers <u>W</u> type DHP
BHP-4200-005 Revision 4	Preventative Maintenance Inspection of 480 Volt Breakers <u>W</u> type DS
BHP-4200-006 Revision 1	Monthly Electrical Inspection of Hoists and Cranes
BHP-4200-010 Revision 1	8 Hour Battery Operated Emergency Light Units Quarterly Inspection
BHP-4200-014 Revision 1	Security Diesel Battery Inspection (Quarterly)
BHP-4200-015 Revision 3	Preventative Maintenance Inspection of Reactor Scram Breakers
BHP-4200-021 Revision 1	Annual "E" Field Maintenance Inspec- tion
BHP-4200-025 Revision 0	Annual Multiplexing Unit Tamper Alarm Maintenance Inspection
BHP-4200-028 Revision 0	Thermal Overload Testing
BHP-4100-039 Revision 1	Setting Torque Switches on Limit- torque Valve Operators
BHP-4200-046 Revision 0	Control Switch Replacement
BHP-4299-003 Revision 1	Electrical Maintenance "A" Perfor- mance Program
BHP-T-1 Revision 0	125 Volt DCAK RS 250 Breaker Inspection Check List

(2) Electrical Surveillance Procedures

BHS 3.1.1-1 Revision 0	Turbine Stop Valve Limit Switch
BHS 3.3.7.1-2 Revision 2	Semiannual Surveillance of the U.V. Fire Detection Instrument Trip Actuation Device and Supervised Circuit Detection Zones 38, 71, 72
BHS 6.4.2.b.3-1 Revision 0	Thermal Hydrogen Recombiner Heater Electrical Function Test

BHS 7.10.1.3.C-1 Revision 0	Diesel Fire Pump Battery 18 Month Surveillance
BHS 7.10.3.2.b.1-2 Revision 1	Diesel Generator Room 18 and Day Tank Room 18 Low Pressure CO ₂ System Actuation
BHS 7.10.3.2.h.1-4 Revision 4	Lower Cable Spreading Room Area 121 - Low Pressure CO ₂ System Actuation
BHS 7.10.4.C-1 Revision 0	Upper Cable Spreading Room 1661 Halon System Actuation
BHS 8.2.1.2.C-1 Revision 2	125 Volt Battery Bank 18 Month Surveillance
BHS 8.4.1.a.3-1 Revision 0	Containment Penetration Conductor Overcurrent Protection Device 260 Volt AC - RCD Power (Fuses)
BHS 8.4.1.b-1 Revision 1	Containment Penetration Conductor Overcurrent Protection Device from 6.9 KV Switch Gear
BHS 8.4.2.a-1 Revision 1	Motor Operated Valve Thermal Over Load Protection Surveillance
BHS DC.02 Revision 2	125 Volt Uninterruptable Power Supply Battery - 18 Month Inspection
BHS DC.10 Revision 0	Periodic Testing of Circuit Breakers That Feed Non 1E Loads from Class 1E DC Bus

(3) Mechanical Maintenance Procedures

BMP 1000-001 Revision 0	Control of Station Procedures Identified for Field Use in Maintenance Department
BMP 1200-001 Revision 5	Monthly/Yearly Crane Inspection
BMP 1200-002 Revision 1	Periodic and Annual Inspection for Rigging Equipment and Special Lifting Devices
BMP 3000-001 Revision 2	Control of the Movement of Heavy Loads
BMP 3100-002 Revision 2	Single or Multi-Stage Pump Repair
BMP 3100-003 Revision 2	Internal Inspection and/or Minor Repair of Valves
BMP 3100-009 Revision 0	Bench Testing Safety Relief Valves
BMP 3101-001 Revision 0	Preparation of Weld Inspection Record/Weld Map
BMP 3101-001 Revision 0	Dismantling, Inspection, Part Replacement and Assembly of the Auxiliary Feedwater Engine Driven Cooling Water Pumps (Hayward Tyler Pumps)
BMP 3106.001 Revision 0	Disassembly, Inspection, Part Replacement and Assembly of the Containment Spray Pumps

BMP 3108-001 Revision 1	Diesel Generator Inspection
BMP 3108-005 Revision 2	Inspection and/or Replacement of Diesel Generator Power Piston
BMP 3114-002 Revision 1	Charging and Draining Main Steam Isolation Valves - Nitrogen and Hydraulics
BMP 3118-001 Revision 1	Reactor Vessel Closure Head Removal
BMP 3118-007 Revision 1	Reactor Vessel Closure Head Installation
BMP 3118-008 Revision 1	Removal, Replacement/Inspection and Reassembly of Reactor Coolant Pump Seals (with motor and motor support in place)
BMP 3200-005 Revision 2	Annual Portable Extinguisher: Hose Reel and Hose Rack Maintenance Inspection
BMP 3200-003 Revision 0	Testing Main Steam Safety Valves
BMP 3200-008 Revision 0	Annual Fire Hydrant Hose Hydrostatic Test
BMP 3300-004 Revision 1	Control and Handling of Filler Material During Welding Process

(4) Mechanical Maintenance Surveillance Procedures

BMS AF C1 Revision 0	Auxiliary Feedwater System Checkvalves Surveillance
BMS 3.4.2.F-1 Revision 0	Inspection of Main Turbine Valves
BMS 7.10.4.b-1 Revision 2	Semiannual Surveillance of the Upper Cable Spreading Room Halon Fire Protection System
BMS 7.10.5.b-1 Revision 2	Annual Fire Hose Station Maintenance Surveillance
BMS 7.10.5.c.2-1 Revision 0	3 year Fire Hose Hydrostatic Test

(5) Instrument Maintenance Procedures

BIP 2401-002 Revision 0	Bench Calibration of <u>W</u> 7300 RTD Amplifier Card NRA <u>G</u> 01 and G03
BIP 2401-005 Revision 0	Bench Calibration of <u>W</u> 7300 Signal Comparator Card NAL- <u>G</u> 01
BIP 2401-024 Revision 0	Bench Calibration of <u>W</u> 7300 RTD Amplifier Card MRA <u>G</u> 02 and G04
BIP 2401-029 Revision 0	Bench Calibration of <u>W</u> 7300 Voltage to Pulse Converter MVP <u>G</u> 01 Card
BIP 2401-034 Revision 0	Bench Calibration of <u>W</u> 7300 Temperature Channel Test Card MTC <u>G</u> 01 and G04
BIP 240-040 Revision 0	Bench Calibration of <u>W</u> 7300 Master Test (MMT) Card

BIP 2401-048 Revision 1	Weekly Accuracy Check of Honeywell Visicorder and Associated Ampli- fiers
BIP 2000-001 Revision 0	Recertification Program for W 7300 Control System Circuit Cards
BIP 2000-003 Revision 0	Frequency of Calibration of In-Plant Instruments - FSAR Requirements
BIP 2400-001 Revision 1	Certification of Heise Gauge
BIP 2400-007 Revision 0	Calibration of Differential Pressure Indicating Switches
BIP 2400-012 Revision 0	Calibration of Liquid Level Controller
BIP 2400-014 Revision 0	Calibration of Level Transmitter with Sealed Capillary
BIP 2400-027 Revision 0	Certification of Magnetic Differential Pressure Gauge
BIP 2400-036 Revision 1	Calibration of Differential Pressure Transmitter
BIP 2400-040 Revision 0	Calibration of Pneumatic Temperature Transmitter
BIP 2400-050 Revision 0	Calibration of Magnehelic Differential Pressure Gauge
BIP 2400-071 Revision 0	Calibration of Honeywell Electronic Control
BIP 2400-081 Revision 0	Alignment of a General Atomics Ionization Chamber Detector
BIP 2500-002 Revision 1	Calibration of Steam Generator Power Operated Relief Valve Pressure Control Loop
BIP 2500-010 Revision 0	Calibration of a Flow Indicating Alarm and Control Loop
BIP 2500-012 Revision 1	Calibration of Boric Acid Blend Flow Control Loop
BIP 2500-017 Revision 0	Calibration of Feedwater Pump Flow/Speed Control Loop
BIP 2500-021 Revision 0	Calibration of Reactor Refueling Canal Water Level Loop
BIP 2500-027 Revision 0	Calibration of Delta T Auctioneer Loop
BIP 2500-030 Revision 0	Calibration of Pressure Control Loop
BIP 2500-038 Revision 0	Calibration of Steam Generator Pressure and Steam Dump to Condenser
BIP 2500-041 Revision 0	Calibration of Pressure Indicating Control Loop Comparator with Analog
BIP 2500-046 Revision 0	Calibration of Charging Header Pressure Indicating Loop
BIP 2500-049 Revision 0	Calibration of a Pressure Indicating Single Alarm Loop with Computer Input
BIP 2500-053 Revision 0	Calibration of Containment Spray Additive Flow Control Loop

BIP 2500-062 Revision 0	Calibration of Essential Service Water Pump Discharge Header Pressure
BIP 2500-079 Revision 0	Calibration of Boric Acid Processing Temperature Loop
BIP 2500-089 Revision 0	Calibration of Reactor Head Vent Temperature Loop

(6) Instrument Maintenance Surveillance

BIS 3.1.1-1 Revision 0	Surveillance Function Test for T/Tavg Protection Channel 1
BIS 3.1.1-6 Revision 0	Surveillance Function Test for Loop 1F-0424 Reactor Coolant 1B Flow Protection Channel 1
BIS 3.1.1-23 Revision 1	Surveillance Function Test for Loop 1L-0558 Steam Generator 1C Level Protection Channel 2
BIS 3.1.1-25 Revision 0	Surveillance Functional Test for Loop 1L-05 29 Steam Generator 1B Level Protection Channel 1
BIS 3.1.1-31 Revision 0	Surveillance Function Test for Loop 1L-0527 Steam Generator 1B Level Protection Channel 4
BIS 3.1.1-37 Revision 1	Surveillance Functional Test for Loop 1P-0455 Pressurizer Pressure Protection Channel 1
BIS 3.1.1-45 Revision 0	Surveillance Functional Test for Loop Turbine Impulse Chamber Pressure 1P-0506 Protection Channel 2
BIS 3.1.1-201 Revision 0	Surveillance Calibration of Reactor Coolant Flow Loop
BIS 3.1.1-206 Revision 1	Calibration of Pressurizer Pressure Protection Channel 1 Loop 0455
BIS 3.1.1-212 Revision 1	Surveillance Calibration of the Pressurizer Water Level Protection Channel 3 Loop 1L 0461
BIS 3.1.1-223 Revision 0	Surveillance Calibration of Nuclear Instrumentation System Intermediate Range N36 and N35
BIS 3.2.1- Revision 0	Surveillance functional Test for the Steam Generator Loop 1B Pressure Compensation 1P-0524 Channel 1
BIS 3.3.1-6 Revision 0	Surveillance Function Test of the Fuel Building Fuel Handling Incident Radiation Monitor ORT-AR055
BIS 3.3.1-205 Revision 1	Surveillance Calibration of a Liquid Gamma Process Radiation Monitor Detector
BIS 3.3.5-202 Revision 0	Surveillance Calibration of Miniflow Residual Heat Removal Flow Rate

BIS 3.3.6-201 Revision 0	Surveillance Calibration of Boric Acid Storage Tank Level
BIS 3.3.6-204 Revision 0	Surveillance Calibration of Wide Range Pressure Loop, 1A Hot Leg and 1C Hot Leg
BIS 3.3.9-201 Revision 0	Surveillance Calibration of Liquid Radwaste Release Flow Control Loop
BIS 3.3.10-004 Revision 0	Surveillance Functional Test of the Component Cooling Heat Exchanger 1 Outlet Radiation Monitor 1PR09J
BIS 3.3.10-011 Revision 0	Surveillance Functional Test of Auxiliary Building Unit 2 Exhaust Tunnel Air Flow Loop
BIS 4.6.1-001 Revision 0	Surveillance Functional Test of Hydrogen Analyzer
BIS 5.1.2.a-003 Revision 0	Surveillance Functional Test for Accumulator Tank 1B Level Loop 1L-0952
BIS 5.1.2.b-200 Revision 1	Surveillance Calibration of the Accumulator Tank Pressure

As a result of this review, the inspector provided comments and questions to the licensee for consideration where necessary to meet regulatory requirements. Because many of the procedures are the initial procedures and need to go through a trial period after the plant becomes operational, the inspector anticipated subsequent reviews by the licensee as the procedures are put to use.

No items of noncompliance or deviations were identified.

9. Plant Tours/Housekeeping

The inspectors conducted plant tours on June 5, 7, 8, July 3, 5, 9, 23, 24, 26, 27, 30, 31, 1984. The areas of the plant observed during the tours included Unit 1 and 2 containments, control room, fuel handling and storage areas, auxiliary building areas and diesel generator B room. Areas were inspected for work in progress, state of cleanliness resulting from lagging work, overall housekeeping, state of fire protection equipment and methods being employed, and the care and preservation of safety-related components and equipment. The inspectors were accompanied by licensee personnel on portions of the tours for the purpose of identifying areas where additional housekeeping efforts should be concentrated to bring the overall cleanliness of Unit 1 spaces up to par with the current state of construction. Inspector concerns were related to the licensee.

No items of noncompliance or deviations were identified.

10. Open Items

Open items are matters which have been discussed with the licensee, which will be reviewed further by the inspector, and which involve some action on the part of the NRC or licensee or both. An open item disclosed during the inspection is discussed in Paragraph 5.b.

11. Exit Interview

The inspector met with licensee representatives denoted in Paragraph 1 at the conclusion of the inspection on July 31, 1984. The inspector summarized the purpose and the scope of the inspection and the findings.