

OPERATING DATA REPORT

DOCKET NO. 59-316
 DATE 8-3-84
 COMPLETED BY W.T. Gillett
 TELEPHONE 616-465-590

OPERATING STATUS

1. Unit Name: Donald C. Cook 2
 2. Reporting Period: July 1984
 3. Licensed Thermal Power (MWt): 3411
 4. Nameplate Rating (Gross MWe): 1133
 5. Design Electrical Rating (Net MWe): 1100
 6. Maximum Dependable Capacity (Gross MWe): 1100
 7. Maximum Dependable Capacity (Net MWe): 1060
 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): _____
 10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5111	57,695
12. Number Of Hours Reactor Was Critical	546.9	2183.7	39,968.9
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	484.5	2112.5	38,912.6
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,417,277	6,822,461	125,295,429
17. Gross Electrical Energy Generated (MWH)	446,380	2,239,560	40,466,170
18. Net Electrical Energy Generated (MWH)	430,209	2,161,815	39,015,186
19. Unit Service Factor	65.1	41.3	70.2
20. Unit Availability Factor	65.1	41.3	70.2
21. Unit Capacity Factor (Using MDC Net)	54.6	39.7	67.0
22. Unit Capacity Factor (Using DER Net)	52.6	38.5	65.8
23. Unit Forced Outage Rate	0	1.5	13.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
 December, 1984 - Surveillance Outage for Ice Basket Weighing and Ice Condenser Door Inspection.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

25. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

8409130427 840731
 PDR ADDOCK 05000316
 R PDR

IE24

(M/T)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-316

UNIT 2

DATE 8/2/84

COMPLETED BY W.T. Gillett

TELEPHONE 616-465-5901

MONTH July, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWE-Net)
1	-
2	-
3	-
4	-
5	-
6	-
7	-
8	-
9	-
10	-
11	37
12	282
13	415
14	412
15	546
16	656

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	750
18	946
19	1033
20	1028
21	1045
22	1048
23	1032
24	1037
25	1085
26	1086
27	1078
28	1083
29	1094
30	1084
31	1067

INSTRUCTIONS

On this format list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-316
 UNIT NAME D.C. Cook - Unit 2
 DATE 8-13-84
 COMPLETED BY B.A. Svensson
 TELEPHONE 616/465-5901
 PAGE 1 of 1

REPORT MONTH JULY, 1984

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
147 Cont'd.	840310	S	259.5	B&C	1	N.A.	ZZ	ZZZZZZ	The Unit was removed from service on 840310 for scheduled Cycle IV - V refueling/maintenance outage. The outage work has been completed and initial criticality for Cycle V occurred on 840707. Low power physics testing was completed on 840711 and the Unit was paralleled to the System the same day. The power ascension testing program was completed and the Unit reached 100% reactor power on 840719. The total length of the outage was 2966.4 hours.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely¹. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation

¹Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT #. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- A. If a component failed, use the component directly involved.
- B. If not a component failure, use the related component: e.g., wrong valve operated through error: list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Docket No.: 50-316
Unit Name: D. C. Cook Unit 2
Completed By: G. J. Peak
Telephone: (616) 465-5901
Date: 08/07/84
Page: 1 of 1

MONTHLY OPERATING ACTIVITIES - JULY 1984

Highlights:

The unit entered the reporting period in Mode 3 in the process of unit startup following a refueling outage. The reactor was taken critical and low power physics testing was completed. The unit was then loaded to 100% power where it was operating as the reporting period ended. No major power reductions occurred during the reporting period.

Total electrical generation for the month was 446,380 MWH.

Summary:

- 7-6-84 All control banks and shutdown banks were completely withdrawn at 2230 hours and the dilution to criticality started at 2251 hours.
- 7-7-84 Mode 2 was entered at 0138 hours and the reactor was critical at 0323 hours.
- 7-9-84 The unit went back into Mode 3 at 2220 hours due to problems with the rod position indication.
- 7-10-84 Mode 2 was entered again at 2120 hours.
- 7-11-84 The reactor went critical again at 0015 hours and Mode 1 was entered at 0621 hours. The main turbine was rolled at 0918 hours and the main generator was paralleled at 1930 hours.
- 7-14-84 The turbine driven auxiliary feedwater pump was inoperable from 1330 hours on 7-14-84 to 0010 hours on 7-15-84 due to a malfunctioning emergency leak off valve.
- 7-19-84 The unit reached full power at 1820 hours.
- 7-24-84 The east component cooling water loop was inoperable from 0300 hours on 7-24-84 to 1427 hours on 7-25-84 to repair a valve in the system.

The Control Room Cable Vault Halon System remains inoperable as of 1707 hours on 4/14/85. The backup CO₂ System remains operable.

DOCKET NO.	<u>50 - 316</u>
UNIT NAME	<u>D. C. Cook - Unit No. 2</u>
DATE	<u>8-13-84</u>
COMPLETED BY	<u>B. A. Svensson</u>
TELEPHONE	<u>(616) 465-5901</u>
PAGE	<u>1 of 1</u>

MAJOR SAFETY-RELATED MAINTENANCE

JULY, 1984

- M-1 Equalizing line around RH-128E was observed to be leaking. The leaking portions of pipe were replaced.
- M-2 The #3 Steam Generator blowdown isolation valve #DCR-330 was observed to be leaking through. The valve was disassembled, internal parts inspected and reassembled utilizing new gaskets.
- M-3 Removal and inspection of the #21 Reactor Coolant Pump seals was performed, replacing the #3 seal ring, all flex gaskets and o-rings.
- M-4 Removal and inspection of the #22 Reactor Coolant Pump seals was performed, replacing the #3 seal runner, #1 seal insert, ring and runner, all flex gaskets, and o-rings.
- M-5 Pressurizer Power Operated Relief Valve, NRV-153, was observed to be leaking through. The seat ring, gaskets, plug and stem assembly were replaced. Upon reassembly the valve was tested and returned to service.
- C&I-1 QTI-220, RCP #22 lower bearing temperature failed low. The RTD was found open and was replaced. Proper indication was restored.
- C&I-2 The RTD for loop 1 cold leg failed, causing Tavg to fail low. The bistables were tripped within 1 hour, the spare RTD was connected, the R/E converter recalibrated and the STP was run to verify operability of the loop. The temperature indication was restored.



INDIANA & MICHIGAN ELECTRIC COMPANY

Donald C. Cook Nuclear Plant
P.O. Box 458, Bridgman, Michigan 49106

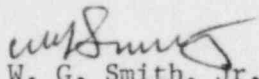
August 13, 1984

Director, Office Of Management Information
and Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

Pursuant to the requirements of Donald C. Cook Nuclear Plant Unit 2
Technical Specification 6.9.1.6, the attached Monthly Operating
Report for the Month of July, 1984 is submitted.

Sincerely,


W. G. Smith, Jr.
Plant Manager

WGS:ab

Attachments

cc: J. E. Dolan
M. P. Alexich
R. W. Jurgensen
NRC Region III
E. R. Swanson
R. O. Bruggee (NSAC)
R. C. Callen
S. J. Mierzwa
R. F. Kroeger
B. H. Bennett
J. D. Huebner
J. H. Hennigan
A. F. Kozlowski
R. F. Hering
J. F. Stietzel
PNSRC File
INPO Records Center
ANI Nuclear Engineering Department

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