

ATTACHMENT 1

NRC DOCKET 50-321
OPERATING LICENSE DPR-57
EDWIN I. HATCH NUCLEAR PLANT UNIT 1
REVISION TO PROPOSED APRM/RBM
TECHNICAL SPECIFICATIONS

The proposed changes to Technical Specifications (Appendix A to Operating License DPR-57) would be incorporated as follows:

<u>Remove Page</u>	<u>Insert Page</u>
3.2-16a*	3.2-16a*
3.3-5*	3.3-5*
3.6-10	3.6-10
	Figure 3.6-5

*These pages should be used to replace the pages of the same respective number which were enclosed with the GPC Technical Specifications change proposal dated February 6, 1984 (NED-84-030).

September 6, 1984

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Table 3.2-7 (Continued)

Ref. No. (a)	Instrument	Trip Condition Nomenclature	Required Operable Channels per Trip System	Trip Setting	Remarks
4	RBM	Upscale	i(e)(f)		
		Low Trip Setpoint (LTSP)		$\leq 117/125$ of full scale	There are three upscale trip levels. Only one is applied over a specified operating core thermal power range. All RBM trips are automatically bypassed below the low power setpoint. The upscale LTSP is applied between the low power and the intermediate power setpoints. The upscale ITSP is applied between the intermediate power setpoint and the high power setpoint. The upscale HTSP is applied above the high power setpoint.
		Intermediate Trip Setpoint (ITSP)		$\leq 111.2/125$ of full scale	
		High Trip Setpoint (HTSP)		$\leq 107.4/125$ of full scale	
		Power Range Setpoints	Not applicable		
		Low Power Setpoint (LPSP)		$\leq 30\%$ rated core thermal power	Power range setpoints control the enforcement of the appropriate upscale trips over the proper core thermal power ranges. The power signal to the RBM is provided by the APRM.
		Intermediate Power Setpoint (IPSP)		$\leq 65\%$ rated core thermal power	
		High Power Setpoint (HPSP)		$\leq 85\%$ rated core thermal power	
		Bypass Time Delay (td ₂)	Not applicable	≤ 2.0 seconds	RBM bypass time delay is set low enough to assure minimum rod movement while upscale trips are bypassed.
5	Scram Discharge Volume	High Water Level	1(g)	≤ 18 gallons	

3.2-16a

3.3.F. Operation with a Limiting Control Rod Pattern (for Rod Withdrawal Error, RWE)

A Limiting Rod Pattern for RWE exists when:

1. Thermal power is below 90% of rated and the MCPR is less than 1.70, or
2. Thermal power is 90% of rated or above and the MCPR is less than 1.40.

During operation with a Limiting Control Rod Pattern for RWE and when core thermal power is $\geq 30\%$, either:

1. Both RBM channels shall be operable, or
2. If only one RBM channel is operable, control rod withdrawal shall be blocked within 24 hours, or
3. If neither RBM channel is operable, control rod withdrawal shall be blocked.

G. Limiting the Worth of a Control Rod Below 20% Rated Thermal Power

1. Rod Worth Minimizer (RWM)

Whenever the reactor is in the Start & Hot Standby or Run Mode below 20% rated thermal power, the Rod Worth Minimizer shall be operable or a second licensed operator shall verify that the operator at the reactor console is following the control rod program.

4.3.F Operation with a Limiting Control Rod Pattern (for Rod Withdrawal Error, RWE)

During operation when a Limiting Control Rod Pattern for RWE exists and only one RBM channel is operable, an instrument functional test of the RBM shall be performed prior to withdrawal of the control rod(s). A Limiting Rod Pattern for RWE is defined by 3.3.F.

G. Limiting the Worth of a Control Rod Below 20% Rated Thermal Power

1. Rod Worth Minimizer (RWM)

Prior to the start of control rod withdrawal at startup, and as soon as automatic initiation of the RWM occurs during rod insertion while shutting down, the capability of the Rod Worth Minimizer to properly fulfill its function shall be verified by the following checks.

- a. The correctness of the control rod withdrawal sequence input to the RWM computer shall be verified.
- b. The RWM computer on line diagnostic test shall be successfully performed.
- c. Proper annunciation of the selection error of at least one out-of-sequence control rod in each fully inserted group shall be verified.
- d. The rod block function of the RWM shall be verified by withdrawing or inserting an out-of-sequence control rod no more than to the block point.

4.6.I. Jet Pumps (Continued)

2. The indicated value of core flow rate varies from the value derived from loop flow measurements by more than 10%.
3. The diffuser to lower plenum differential pressure reading on an individual jet pump vary from the mean of all jet pump differential pressures by more than 10%.

3.6.J. Recirculation Pump Speeds

1. Core thermal power shall not exceed 1% of rated thermal power without forced recirculation.
2. Operation with a single recirculation pump is permitted for 24 hours unless the recirculation pump is sooner made operable. With one recirculation pump not in operation, initiate action within 15 minutes or continue action to reduce reactor power to or below the limit specified in Figure 3.6-5 within 2 hours. If the pump cannot be made operable or the limit of Figure 3.6-5 cannot be met within the required time, the reactor shall be in cold shutdown within 24 hours.
3. Following one pump operation the discharge valve of the low speed pump may not be opened unless the speed of the faster pump is less than 50% of its rated speed.

K. Structural Integrity of Primary System Boundary

The structural integrity of the primary system boundary shall be maintained at the level required to assure safe operation throughout the life of the unit. The reactor shall be maintained in a Cold Shutdown Condition until each indication of a defect has been investigated and evaluated.

4.6.J. Recirculation Pump Speeds

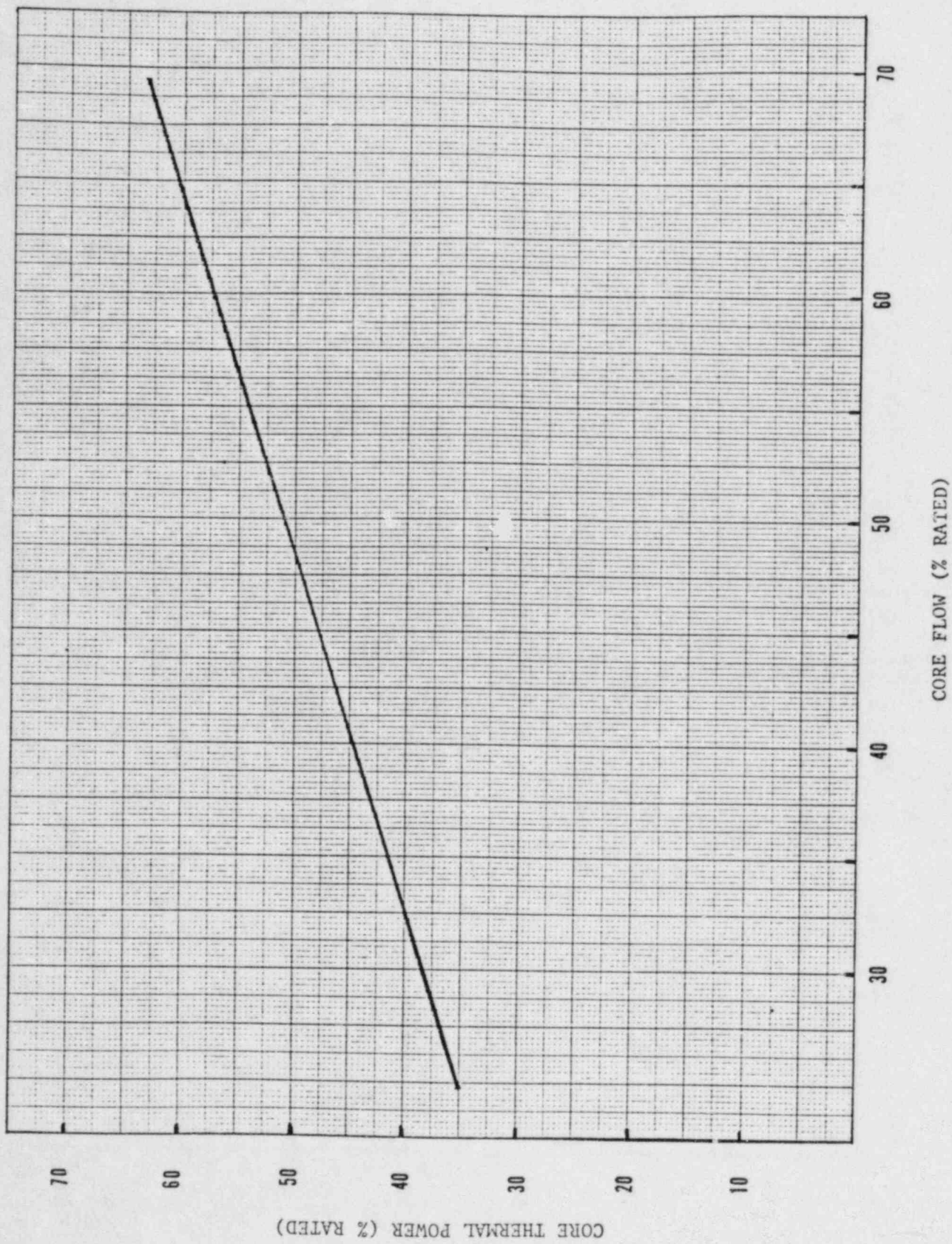
Recirculation pump speeds shall be recorded at least once per day.

K. Structural Integrity of Primary System Boundary

A preservice inspection of accessible components listed in Table 4.6-1 will be conducted before initial fuel loading to establish a preservice base for later inspections. The nondestructive inspections listed in Table 4.6-1 shall be performed as specified. The results obtained from compliance with this specification will be evaluated after 5 years and the conclusions of this evaluation will be reviewed with the NRC.

FIGURE 3.6-5

THERMAL POWER LIMITATIONS DURING OPERATION WITH LESS THAN TWO REACTOR COOLANT SYSTEM RECIRCULATION LOOPS IN OPERATION



ATTACHMENT 2

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The following is a listing of the specific Technical Specifications revisions requested by this submittal and the basis for consideration of these changes as not involving a significant hazard as described in 10 CFR 50.92:

- 1) Add "With one recirculation pump not in operation, initiate action within 15 minutes and continue action to reduce reactor power to or below the limit specified in Figure 3.6-5 within 2 hours." and "... or the limit of Figure 3.6-5 cannot be met within the required time..." to Specification 3.6.J.2, and add Figure 3.6-5 as a new page.

BASIS:

This change would restrict plant operation to the region at or below the limit shown in Technical Specifications Figure 3.6-5, within 2 hours after either recirculation loop is declared inoperable. That limit corresponds to a load line leading to 80% reactor power at rated core flow. This constitutes an additional restriction not presently in the Technical Specifications; therefore, this proposed change is consistent with Item (ii) of the "Examples of Amendments that are Considered Not Likely to Involve Significant Hazards Considerations" listed on page 14,870 of the April 6, 1983, issue of the Federal Register.

- 2) Add " \leq " to the RBM - LTSP Trip Setting on Table 3.2-7.

BASIS:

This change corrects an omission in the Technical Specifications revisions proposed by NED-84-030 (dated February 6, 1984) and otherwise has no effect on the significant hazards evaluation for that submittal. Therefore this proposed change is consistent with Item (i) of the "Examples of Amendments that are Considered Not Likely to Involve Significant Hazards Considerations" listed on page 14,870 of the April 6, 1983, issue of the Federal Register.

- 3) Remove "...within 1 hour" from Specification 3.3.F.

BASIS:

This change is within the bounds of the significant hazards evaluation enclosed in NED-84-030 (dated February 6, 1984), and has no affect on the conclusions of that evaluation.