ENCLOSURE 1

NRC DOCKET 50-366
OPERATING LICENSE NPF-5
EDWIN I. HATCH NUCLEAR PLANT UNIT 2
REQUEST TO AMEND TECHNICAL SPECIFICATIONS

The proposed change to the Unit 2 Technical Specifications (Appendix A to Operating License NPF-5) would be incorporated as follows:

Remove Page	Insert Page
3/4 3-11	3/4 3-11
3/4 3-15	3/4 3-15
3/4 3-16	3/4 3-16
3/4 3-21	3/4 3-21

TABLE 3.3.2-1

ISOLATION ACTUATION INSTRUMENTATION

	TRIP FUNCTION	VALVE GROUPS OPERATED BY SIGNAL (a)	MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b) (c)	APPLICABLE OPERATIONAL CONDITION	ACTION
1.	PRIMARY CONTAINMENT ISOLATION				
a.	Reactor Vessel Water Level				
	1. Low (Level 3) (2B21-N680 A,B,C,D)	2, 6, 10, 11, 12,	2	1, 2, 3	20
	2. Low-Low (Level 2) (2B21-N682 A,B,C,D)	5, #,*	2	1, 2, 3	20
	3. Low-Low-Low (Level 1) (2B21-N681 A,B,C,D)	1	2	1, 2, 3	20
b.	Drywell Pressure - High (2C71-N650 A, B, C, D)	2, 6, 7, 10 12, #,*	2	1, 2, 3	20
c.	Main Steam Line				
	1. Radiation - High (2D11-K603 A,B,C,D)	1, 12, #, (d)	2	1, 2, 3	21
	2. Pressure - Low (2B21-N015 A,B,C,D)	1	2	1	22
	3. Flow - High (2B21-N686 A,B,C,D) (2B21-N687 A,B,C,D) (2B21-N688 A,B,C,D) (2B21-N689 A,B,C,D)	1, #	2/line	1, 2, 3	21
d.	Main Steam Line Tunnel High Temperature - High (2B21-N623 A,B,C,D) (2B21-N624 A,B,C,D) (2B21-N625 A,B,C,D) (2B21-N626 A,B,C,D)	1	2/line(e)	1, 2, 3	21
e.	Condenser Vacuum - Low (2B21-N056 A, B, C, D)	1	2	1,2(f),3(f)	23
f.	Turbine Building Area Temperature - High (2U61-R001, 2U61-R002, 2U61-R003, 2U61-R004)	1	2(e)	1, 2, 3	21
g.	Drywell Radiation-High (2D11-K621 A,B)	(j)	1	1, 2, 3	29

TABLE 3.3.2-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION
ACTION

ACTION 20 - Be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the next 30 hours.

ACTION 21 - Be in at least STARTUP with the main steam line isolation valves closed within 2 hours or be in at least HOT SHUTDOWN within 6 hours and in COLD SHUTDOWN within the next 30 hours.

ACTION 22 - Be in at least STARTUP within 2 hours.

ACTION 23 - Be in at least STARTUP with the Group 1 isolation valves closed within 2 hours or in at least HOT SHUTDOWN within 6 hours.

ACTION 24 - Establish SECONDARY CONTAINMENT INTEGRITY with the standby gas treatment system operating within one hour.

ACTION 25 - Isolate the reactor water cleanup system.

ACTION 26 - Close the affected system isolation valves and declare the affected system inoperable.

ACTION 27 - Verify power availability to the bus at least once per 12 hours or close the affected system isolation valves and declare the affected system inoperable.

ACTION 28 - Close the shutdown cooling supply and reactor vessel head spray isolation valves unless reactor steam dome pressure $\leq$ 145 psig.

ACTION 29 - Either close the affected isolation valves within 24 hours or be in hot shutdown within the next 6 hours and in cold shutdown within the next 30 hours.

NOTES

\# Actuates operation of the main control room environmental control system in the pressurization mode of operation.

* Actuates the standby gas treatment system.

** When handling irradiated fuel in the secondary containment.

a. See Specification 3.6.3, Table 3.6.3-1 for valves in each valve group.

b. A channel may be placed in an inoperable status for up to 2 hours for required surveillance without placing the trip system in the tripped condition provided at least one other OPERABLE channel in the same trip system is monitoring that parameter.

c. With a design providing only one channel per trip system, an inoperable channel need not be placed in the tripped condition where this would cause the Trip Function to occur. In these cases, the inoperable channel shall be restored to OPERABLE status within 2 hours or the ACTION required by Table 3.3.2-1 for that Trip Function shall be taken.

d. Trips the mechanical vacuum pumps.

e. A channel is OPERABLE if 2 of 4 instruments in that channel are OPERABLE.

f. May be bypassed with all turbine stop valves closed.

g. Closes only RWCU outlet isolation valve 2G31-F004.

h. Alarm only.

i. Adjustable up to 60 minutes.

j. Isolates containment purge and vent Valves.

TABLE 3.3.2-2

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

TRIP FUNCTION	TRIP SETPOINT	ALLOWABLE VALUE
1. PRIMARY CONTAINMENT ISOLATION		
a. Reactor Vessel Water Level		
1. Low (Level 3)	≥ 8.5 inches*	≥ 8.5 inches*
2. Low Low (Level 2)	≥ -55 inches*	≥ -55 inches*
3. Low Low Low (Level 1)	≥ -121.5 inches*	≥ -121.5 inches*
b. Drywell Pressure - High	≤ 1.85 psig	≤ 1.85 psig
c. Main Steam Line		
1. Radiation - High	≤ 3 x full power background	≤ 3 x full power background
2. Pressure - Low	≥ 825 psig	≥ 825 psig
3. Flow - High	≤ 138% of rated flow	≤ 138% of rated flow
d. Main Steam Line Tunnel Temperature - High	≤ 194°F	≤ 194°F
e. Condenser Vacuum - Low	≥ 7" Hg vacuum	≥ 7" Hg vacuum
f. Turbine Building Area Temp. - High	≤ 200°F	≤ 200°F
g. Drywell Radiation-High	≤ 138 R/hr	≤ 138 R/hr
2. SECONDARY CONTAINMENT ISOLATION		
a. Reactor Building Exhaust Radiation - High	≤ 60 mr/hr	≤ 60 mr/hr
b. Drywell Pressure - High	≤ 1.85 psig	≤ 1.85 psig
c. Reactor Vessel Water Level - Low Low (Level 2)	≥ -55 inches*	≥ -55 inches*
d. Refueling Floor Exhaust Radiation - High	≤ 20 mr/hr	≤ 20 mr/hr

*See Bases Figure B 3/4 3-1.

TABLE 4.3.2-1

ISOLATION ACTUATION INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TRIP FUNCTION	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS IN WHICH SURVEILLANCE REQUIRED
1. PRIMARY CONTAINMENT ISOLATION				
a. Reactor Vessel Water Level				
1. Low (Level 3)	S	M	R	1, 2, 3
2. Low Low (Level 2)	S	M	R	1, 2, 3
3. Low Low Low (Level 1)	S	M	R	1, 2, 3
b. Drywell Pressure - High	S	M	R	1, 2, 3
c. Main Steam Line				
1. Radiation - High	D	W(a)	R	1, 2, 3
2. Pressure - Low	NA	M	Q	1
3. Flow - High	S	M	R	1, 2, 3
d. Main Steam Line Tunnel Temperature - High	S	M	R	1, 2, 3
e. Condenser Vacuum - Low	NA	M	Q	1, 2#, 3#
f. Turbine Building Area Temp. - High	NA	M	R	1, 2, 3
g. Drywell Radiation-High	D	M	R	1, 2, 3
2. SECONDARY CONTAINMENT ISOLATION				
a. Reactor Building Exhaust Radiation - High	D	M(a)	R	1, 2, 3, 5 and *
b. Drywell Pressure - High	S	M	R	1, 2, 3
c. Reactor Vessel Water Level - Low Low (Level 2)	S	M	R	1, 2, 3
d. Refueling Floor Exhaust Radiation - High	D	M(a)	Q	1, 2, 3, 5 and *

*When handling irradiated fuel in the secondary containment.
#May be bypassed with all turbine stop valves closed.
(a) Instrument alignment using a standard current source.

ENCLOSURE 2

NRC DOCKET 50-366
OPERATING LICENSE NPF-5
EDWIN I. HATCH NUCLEAR PLANT UNIT 2
REQUEST TO AMEND TECHNICAL SPECIFICATIONS

Pursuant to 10 CFR 50.92, Georgia Power Company has evaluated the attached proposed amendment and has determined that its adoption would not involve a significant hazard. The basis for this determination is as follows:

PROPOSED CHANGE

Add limiting conditions for operation, trip setpoints, and surveillance requirements for the monitors which provide the high radiation isolation signals to the containment purge and vent valves.

BASIS

This change constitutes an additional restriction not presently included in the Technical Specifications. This change does not affect the probability or consequences of an accident or malfunction analyzed in the FSAR. This change does not create the possibility of an accident or malfunction of a different type than any analyzed in the FSAR. The margin of safety as defined in the basis for any technical specification is not affected. The effect of this change is therefore within the acceptance criteria and the change is consistent with Item (ii) of the "Examples of Amendments that are Considered Not Likely to Involve Significant Hazards Considerations" listed on page 14,870 of the April 6, 1983, issue of the Federal Register.