

MONTHLY OPERATIONS REPORT

JULY 1984

Throughout the report period, the Oyster Creek Station remained shutdown for the current Maintenance and Refueling Outage.

Installation of the SRM/IRM dry tubes was completed. Presently, there is a ground problem with IRM 11 and a number of LPRMs. The detector connectors are being dried out and will be retested. Replacement of the bottom entry instrumentation (BEI) started at the end of the report period.

Refueling of the reactor core was completed and verified on July 28, 1984. Shutdown Margin testing was satisfactorily completed on July 31, 1984. Preparation for cavity drain was in progress at the end of the report period. (NOTE: Cavity was drained and the separator was set in place on August 2, 1984). Video taping of the reactor feedwater spargers was completed prior to draining of the cavity. Reassembly of the reactor is the next major activity on the Reactor Building 119' level.

The outage for Diesel Generator No. 1 and Standby Gas Treatment System No. 1 started on August 1, 1984.

During preparation for Shutdown Margin testing, the plant experienced problems starting the reactor recirculation pumps. To date, only "A" recirculation pump can be started. Attempts to start "B" and "D" pumps have not been successful. Plant Engineering and M&C Departments are investigating the problem.

The special containment isolation valve test ("sneak-circuit" test) performed on July 13, 1984 confirmed an inter-connection/circuit wiring problem. Upon lifting of specified neutral leads in the containment isolation circuit, the valves being tested cycled open. The Technical Functions Division is working to resolve the problem.

Filling of the torus was completed during the report period except for final level adjustment. Presently, the containment spray systems are being filled. After system testing is completed, final torus water level adjustments will be made.

The circulating water system for the main condensers was placed into service during the report period. To date, only No. 2 and No. 3 circulating water pumps have been operated. Only one condenser ("C") is presently in service. Initially, upon operating the pumps one at a time, excessively high motor bearing temperatures were experienced. The load on the pumps was reduced by placing one half of "C" main condenser in the back-wash mode. The temperatures of the motor bearing have decreased below the alarm setpoint. Placing the "A" and "B" main condensers in service is pending maintenance on the circulating water system valves associated with the condensers. Maintenance is currently in progress.

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Diesel fire pump No. 1 was returned to service on July 1, 1984. Fire pump No. 1 was out of service since June 25, 1984 due to a problem that was eventually traced to the pump's auto vent valve. Diesel fire pump No. 2 was inoperable from July 3, 1984 to July 19, 1984 due to a blower coupling problem. Vendor assistance was required to repair both pumps.

During operability testing in preparation for Core Spray Systems I and II restoration to service, adjustments were required on both systems test valves due to failure of the valves to fully close. Failure of the valves to close was attributed to incorrect MOVATS test of limitorque settings. In addition, the fill pumps on both systems had to be replaced. A vibration concern with System I core spray pumps NZ01A/NZ01C was evaluated and resolved by Plant Engineering.

Plant Engineering and M&C are presently troubleshooting a possible logic problem with Core Spray System II blocking valve V-20-18. During MOVATS testing of the limitorque on System II parallel isolation valve V-20-21, the blocking valve (V-20-18) also cycled. The investigation into the problem was in progress at the end of the report period. Core Spray System II relief valve had to be sent back to the vendor (Wyle Labs) due to a premature lifting problem. Presently, System II is out of service due to the V-20-18 problem and a pending pump impeller lock nut inspection. NOTE: On August 4, 1984, valve operability of V-20-18 was confirmed to be in accordance with design logic.

The Reactor Building Closed Cooling Water (RBCCW)/Drywell System was placed back in service to support recirculation pump operation for shutdown margin testing. The system will have to be isolated and drained again to leak rate test the drywell RBCCW isolation valves.

Weld repair of the isolation condenser piping is nearing completion. Repairs to "B" isolation condenser are completed and will be tested in the beginning of the next report period (hydro test, isolation test, valve limitorque test). Weld repairs to "A" isolation condenser are still in progress.

Testing and replacement of the plant's Electrical Distribution System power cables continued during the report period. M&C is having difficulty with the removal of the cables running to "E" reactor recirculation pump motor generator set. Plant Engineering is working with M&C to resolve the problem.

Operations is continuing to perform interference checks and vent/time CRDs as plant conditions permit.

The testing of motor operated valve limitorques (MOVATS-Testing) and leak rate testing (LRT) of valves continued during the report period as system valves became available.

Maintenance continued on the dilution plant No. 2 seal water/cooling pump. The pump is experiencing vibration problems.

At the end of the report period, the plant was making preparations to leak rate test MSIV NS04A which failed earlier in the outage. Leak rate testing of the valve was pending flushing of the valve seat after the main steam line plugs are removed.

The replacement of the 24 volt battery system chargers is in progress. Replacement of the "B" station batteries started on August 1, 1984.

Installation of the RBCCW System annubar flow elements was completed.

The M&C Electrical Department is troubleshooting all the new intake screen drive motors which are meggering unsatisfactorily. Presently, all screens are inoperable.

Maintenance continued on the drywell cooling fans. During the report period, drywell fan 1-5 experienced excessive vibration.

At the end of the report period, the Startup and Test group was testing the DWEDT modification and the Shutdown Cooling System prior to turning the systems over to the plant. Essentially, the DWEDT is in operation during testing.

The following Licensee Event Reports were submitted during July 1984:

Licensee Event Report No. 50-219/84-015

On January 25, 1984, window fogging was found on a number of HFA relays. Fifty-eight (58) relays with date code HW, installed in the Reactor Protection System, began to fog up with an unknown oily vapor within the enclosed portion of the relay when the relays were energized. General Electric was notified of this condition and supplied Oyster Creek with new relays. Two new relays with date code EW were installed and energized on June 9, 1984 and fogging was noticed on June 25, 1984. GE has been notified that further testing is required to determine the cause and solution to the problem. Final resolution is dependent on the results of investigations at GE Labs and GPU Reading Labs.

Licensee Event Report No. 50-219/84-016

On Monday, June 25, 1984, during a review of instrument surveillance procedures, it was discovered that not all sixty (60) installed excess flow check valves had been functionally tested as required by Technical Specifications. The surveillance procedure which dictates the excess flow check valve functional test method cannot be executed on valves which are typically connected to low flow sensing lines. Certain excess flow check valves could not be seated during functional test surveillances due to insufficient flow. The functional test surveillance is being revised to test only those valves which would normally have sufficient flow at hydrostatic test pressure. A modification to the eleven (11) low flow check valve lines is being evaluated and a new surveillance procedure will be developed to satisfy the functional test requirement for check valves in these low flow lines.

Licensee Event Report No. 50-219/84-017

A modification was in progress which involved Plant Computer System tie-ins. During the performance of a procedure which involved the tie-in of computer wiring to existing plant circuitry, a neutral electrical lead was required to be lifted. This action caused fifteen (15) primary containment isolation valves, including all four Main Steam Isolation Valves, to reposition. No violation of the Technical Specifications occurred since primary containment integrity was not required at the time of the incident. The valves repositioned due to an abnormal current flowpath which was established through their solenoids when the neutral lead was lifted. A modification of the neutral wiring configuration and a check of the plant safety circuits for similar problems will be conducted. Also, specific cautions for lifting neutral leads will be incorporated into station administrative procedures.



UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July 1984

DOCKET NO. 50-219  
 UNIT NAME Oyster Creek  
 DATE 8/3/84  
 COMPLETED BY R. Baran  
 TELEPHONE 971-4640

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
31	2-11-83	S	12864	C	1	N/A	ZZ	ZZZZZZ	Start of the 1983 Refueling and Maintenance Outage.

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

REFUELING INFORMATION - July, 1984

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: Presently shutdown for Refueling

Scheduled date for restart following refueling: 9/09/84

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Awaiting receipt of Amendments incorporating Technical Specification changes in support of Cycle 10 operation.

Scheduled date(s) for submitting proposed licensing action and supporting information:

All information to support Cycle 10 operation had already been submitted.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC.
2. Exxon Fuel Assemblies - no major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core = 560  
(b) in the spent fuel storage pool = 980

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800                      Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Full core offload capability will be lost after the 1985 outage. Batch discharge capability will be lost after the 1987 outage. Expanded spent fuel pool rack capacity (2,600) is scheduled for 1984.

OPERATING DATA REPORT  
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: July, 1984
3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe):  $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): N/A
11. REASON FOR RESTRICTION, IF ANY: N/A

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HRS	744.0	5112.0	128040.0
13. HOURS RX CRITICAL	0.0	0.0	84623.9
14. RX RESERVE SHUTDOWN HRS	0.0	0.0	468.2
15. HRS GENERATOR ON-LINE	0.0	0.0	82693.8
16. UT RESERVE SHUTDOWN HRS	0.0	0.0	0.0
17. GROSS THERM ENER (MWH)	0	0	136224729
18. GROSS ELEC ENER (MWH)	0	0	46056905
19. NET ELEC ENER (MWH)	-2663	-14645	44271038
20. UT SERVICE FACTOR	0.0	0.0	64.6
21. UT AVAIL FACTOR	0.0	0.0	64.6
22. UT CAP FACTOR (MDC NET)	0.0	-0.5	55.8
23. UT CAP FACTOR (DER NET)	0.0	-0.4	53.2
24. UT FORCED OUTAGE RATE	0.0	0.0	9.7
25. FORCED OUTAGE HRS	0.0	0.0	8916.8
26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):	N/A		
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME:	9/9/84		

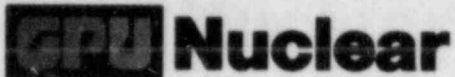
AVERAGE DAILY POWER LEVEL  
NET MWe

DOCKET # . . . . . 50-219  
UNIT . . . . . Oyster Creek #1  
REPORT DATE . . . . . AUGUST 03, 1984  
COMPILED BY . . . . . DONALD V. NOTIGAN  
TELEPHONE # . . . . . 609-971-4695

MONTH JULY, 1984

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	0	16.	0
2.	0	17.	0
3.	0	18.	0
4.	0	19.	0
5.	0	20.	0
6.	0	21.	0
7.	0	22.	0
8.	0	23.	0
9.	0	24.	0
10.	0	25.	0
11.	0	26.	0
12.	0	27.	0
13.	0	28.	0
14.	0	29.	0
15.	0	30.	0
		31.	0





**GPU Nuclear Corporation**  
Post Office Box 388  
Route 9 South  
Forked River, New Jersey 08731-0388  
609 971-4000  
Writer's Direct Dial Number:

August 15, 1984

Director  
Office of Management Information  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Mr. Drew Holland at (609) 971-4643.

Very truly yours,

Peter B. Fiedler  
Vice President and Director  
Oyster Creek

PBF:dsm  
Enclosures

cc: Director (10)  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Dr. Thomas E. Murley, Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

NRC Resident Inspector  
Oyster Creek Nuclear Generating Station  
Forked River, NJ 08731

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