

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-387

UNIT One

DATE August 9, 1984

COMPLETED BY L.A. Kuczynski

TELEPHONE (717)542-3759

MONTH July, 1984

AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
962	17	0
1035	18	21
607	19	142
0	20	662
0	21	814
0	22	1019
0	23	1027
0	24	1020
167	25	1025
601	26	944
724	27	1023
964	28	1021
1030	29	966
1030	30	1022
385	31	1021
0		

# INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

8409130229 840731 PDR ADDCK 05000387 R PDR

IEZH 1/1



# OPERATING DATA REPORT

DOCKET NO. 50-387

DATE August 9, 1984

COMPLETED BY L.A. Kuczynski
TELEPHONE (717)542-3759

To Which Restricted, If Any (Net	1068 1032	Notes  nce Last Report, Give R	leasons:									
rical Rating (Net MWe): 1152 rical Rating (Net MWe): 1065 rependable Capacity (Gross MWe): rependable Capacity (Net MWe): rependable Capacity (Net MWe): rependable Capacity (Net MWe): rependable Capacity Ratings (Items Nutricular Items Nutricul	1032	ice Last Report, Give R	teasons:									
tating (Gross MWe): 1152 rical Rating (Net MWe): 1065 ependable Capacity (Gross MWe): ependable Capacity (Net MWe): occur in Capacity Ratings (Items Nu	1032	ice Last Report, Give R	teasons:									
rical Rating (Net MWe): 1065 ependable Capacity (Gross MWe): ependable Capacity (Net MWe): ccur in Capacity Ratings (Items Nu	1032	ice Last Report, Give R	leasons:									
ependable Capacity (Gross MWe): ependable Capacity (Net MWe): occur in Capacity Ratings (Items Nu To Which Restricted, If Any (Net	1032	ice Last Report, Give R	leasons:									
ependable Capacity (Gross MWe): ependable Capacity (Net MWe): occur in Capacity Ratings (Items Nu To Which Restricted, If Any (Net	1032	ce Last Report, Give R	Reasons:									
To Which Restricted, If Any (Net		ice Last Report, Give R	teasons:									
To Which Restricted, If Any (Net	umber 3 Through 7) Sin	nce Last Report, Give R	teasons:									
			8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: None									
Restrictions, If Any: None	MWe): None											
	This Month	Yrto-Date	Cumulative									
porting Period	744	5,111	10,080									
	563.4	3,151.4	6,996.7									
	249.1	405.8										
	523.9	3,026.7	6,795									
	0	0	0									
al Energy Generated (MWH)	1,491,021	9,074,830	20,336,491									
	481,250	2,965,470	6,632,020									
	461,555	2,853,780	6,390,153									
Factor	White the state of the second state of the sec	59.2	67.4									
ility Factor	70.4	59.2	67.4									
y Factor (Using MDC Net)	60.1	54.1	61.4									
y Factor (Using DER Net)	58.2	52.4	59.5									
Outage Rate	29.6	21.2	16.3									
Scheduled Over Next 6 Months (Ty	pe, Date, and Duration	of Each):										
֡֡֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜֜	porting Period Hours Reactor Was Critical erve Shutdown Hours rator On-Line Shutdown Hours ral Energy Generated (MWH) ral Energy Generated (MWH) Factor rility Factor y Factor (Using MDC Net) y Factor (Using DER Net) Outage Rate Scheduled Over Next 6 Months (Ty	porting Period  Hours Reactor Was Critical  erve Shutdown Hours  ator On-Line  Shutdown Hours  all Energy Generated (MWH)  all Energy Generated (MWH)  all Energy Generated (MWH)  All Energy Generated (MWH)  Factor  ility Factor  y Factor (Using MDC Net)  y Factor (Using DER Net)  Outage Rate  744  753.4  220.1  1,491,021  481,250  461,555  70.4  60.1  70.4  60.1	porting Period 744 5,111  Hours Reactor Was Critical 563.4 3,151.4  erve Shutdown Hours 220.1 249.1  ator On-Line 523.9 3,026.7  Shutdown Hours 0 0  all Energy Generated (MWH) 1,491,021 9,074,830  ical Energy Generated (MWH) 481,250 2,965,470  all Energy Generated (MWH) 461,555 2,853,780  Factor 70.4 59.2  illity Factor (Using MDC Net) 58.2 52.4  y Factor (Using DER Net) 58.2 52.4									



### UNIT SHUTDOWNS AND POWER REDUCTIONS

50-387 DOCKET NO. One UNIT NAME DATE August 9, 1984 L.A. Kuczynski COMPLETED BY TELEPHONE (717)542-3759

REPORT MONTH July, 1984

No.	Date	Type1	Duration (Hours)	Reason 2	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code4	Component Code5	Cause & Corrective Action to Prevent Recurrence
6	840703	F	134.6	Н	3	84-029	CI	INSTRU	The unit scrammed on low vessel level caused by a momentary loss of signal to feedwater and reactor recirculation circuitry following a lightning strike to a transmission line which supplies the station. Extensive post scram testing of the feedwater system could not reproduce the response that occurred (see attached page)
7	840715	F	30.4	Н	3	84-034	EA	ELECON	The unit scrammed on turbine control valve fast closure as a result of a phase-to-phase fault on the 230KV transmission line. The tree which had grown (see attached page)
8	840716	F	26.4	А	3	84-033	HC	VALVEX	The unit scrammed on turbine control valve fast closure by loss of condenser (see attached page)
9	840718	F	28.7	G	3	84-035	НС	VALVEX	The unit scrammed due to loss of con- denser vacuum. The loss of vacuum was (see attached page

F: Forced

S: Scheduled

Reason:

A-Equipment Failure (Explain) B-Maintenance of Test

C-Refueling

**D-Regulatory Restriction** 

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain) H-Other (Explain)

Mathod: 1-Manual

2-Manual Scram.

3-Automatic Scram.

4-Other (Explain)

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS (continued):

# NO. 6:

following the momentary loss of signal. A modification was completed to prevent the reactor recirculation system from receiving a false runback signal if the same circums aces were to occur.

## NO. 7:

tall enough to cause the phase-to-phase arc has been trimmed.

# NO. 8:

vacuum due to a operator inadvertently opening the LP condenser vacuum breaker.

# NO. 9:

caused by an incorrect valve lineup established due to an unclear valve number on a piping diagram. The discrepancy was resolved.

#### UNIT 1

### SUSQUEHANNA STEAM ELECTRIC STATION

Docket Number 50-387 Date 8/9/84 Completed by L.A. Kuczynski Telephone (717)542-2181

# Challenges to Main Steam Safety Relief Valves

During the scram of July 3, 1984, safety relief valve 'B' was manually actuated for RPV pressure control. It was closed after 7 minutes, 20 seconds, during which time reactor pressure went from approximately 1075 psig to approximately 880 pounds.

During the scram of July 15, 1984, safety relief valves 'B', 'E', and 'H' actuated. SRV 'H' closed after two seconds, 'E' after 10 seconds, and 'B' after 11 seconds. Reactor pressure went from 1081 psig to ~975 psig during the 12 seconds.

Changes to the Offsite Dose Calculation Manual

None.

Major Changes to Radioactive Waste Treatment Systems

None.



# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-388

UNIT Two

DATE August 9, 1984

COMPLETED BY L.A. Kuczynski

TELEPHONE (717)542-3759

MONTH July, 1984

AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
0	17	0
0	18	275
83	19	314
114	20	326
0	21	340
0	22	337
0	23	358
16	24	344
111	25	349
279	26	1
310	27	0
336	28	0
337	29	0
296	30	0
113	31	0
0		

## INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.



# **OPERATING DATA REPORT**

DATE August 9, 1984
COMPLETED BY L.A. Kuczynski
TELEPHONE (717)542-3759

Unit 2  1. Unit Name: Susquehanna Steam Elec 2. Reporting Period: July, 1984	tric Station	Notes Initial generator sync.	at
3. Licensed Thermal Power (MWt): 3293		0230 on 7/3/84.	
4. Nameplate Rating (Gross MWe): 1152		Initial electrical prod	luc-
5. Design Electrical Rating (Net MWe): 1065		tion at 1300 on 7/3/84.	
6. Maximum Dependable Capacity (Gross MWe):	*	*To be determined.	
<ol> <li>Maximum Dependable Capacity (Net MWe):</li> <li>If Changes Occur in Capacity Ratings (Items Numbers)</li> </ol>	umber 3 Through 7) Sin	ce Last Report, Give Reasons:	
None		Parker of the property of the same of	
None			
None  Power Level To Which Restricted, If Any (Net )	sued Confirmator	Order, dated July 27, 19	984 t
None  Power Level To Which Restricted, If Any (Net 1)  Reasons For Restrictions, If Any: NRC iss	sued Confirmatory	Order, dated July 27, 19	984 t
9. Power Level To Which Restricted, If Any (Net 1) 0. Reasons For Restrictions, If Any: NRC iss prevent restart of the unit pend	sued Confirmatory	Order, dated July 27, 19	

	This Month	Yrto-Date	Cumulative
11. Hours In Reporting Period	683	683	683
12. Number Of Hours Reactor Was Critical	458.7	458.7	458.7
13. Reactor Reserve Shutdown Hours	180.3	180.3	180.3
14. Hours Generator On-Line	392.9	392.9	392.9
5. Unit Reserve Shutdown Hours	142.4	142.4	142.4
6. Gross Thermal Energy Generated (MWH)	474,034	474,034	474.034
7. Gross Electrical Energy Generated (MWH)	121,330	121,330	121,330
8. Net Electrical Energy Generated (MWH)	110,281	110,281	110,281
9. Unit Service Factor	NA	NA	NA
0. Unit Availability Factor	NA	NA	NA
1. Unit Capacity Factor (Using MDC Net)	NA	NA	NA
2. Unit Capacity Factor (Using DER Net)	NA	NA	NA
3. Unit Forced Outage Rate	NA	NA	NA

24. Shutdowns Schedulea Over Next 6 Months (Type, Date, and Duration of Each):
Maintenance Outage 10/27/84, 7 weeks

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _	8/1/84			
26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved		
INITIAL CRITICALITY	05/09/84	05/08/84		
INITIAL ELECTRICITY	06/28/84	07/03/84		
COMMERCIAL OPERATION	12/31/84			



#### UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-388 UNIT NAME TWO DATE August 9, 1984 COMPLETED BY L.A. Kuczynski TELEPHONE (717)542-3759

REPORT MONTH July, 1984

					Method of Shutting Down Reactor	Report #	System Code4	Component Code5	Prevent Recurrence
1	840705	S	85.6	В	2	NA	ZZ	ZZZZZZ	Reactor scram for scheduled startup testing.
2	840715	F	60.2	А	3	84-034 (Unit #1 LER Number)	нс	RECOMB	Reactor scram due to turbine trip caused by low condenser vacuum.
3	840726	S	142.4	В	3	84-013	ZZ	ZZZZZZ	Reactor scram for scheduled startup testing. Due to an incorrectly completed electrical lineup, the diesel generators were prevented from starting and an Unusual Event was declared. The startup test was subsequently repeated and completed successfully.

F: Forced S: Scheduled

Reason:

A-Equipment Failure (Explain) B-Maintenance of Test

C-Refueling

**D-Regulatory Restriction** 

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

Method:

1-Manual 2-Manual Scram.

3-Automatic Scram.

4-Other (Explain)

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

Exhibit I - Same Source

### Unit 2

# SUSQUEHANNA STEAM ELECTRIC STATION

Docket Number 50-388 Date 8/9/84 Completed By L.A. Kuczynski Telephone (717)542-3759

# Challenges to Main Steam Safety Relief Valves

Following the scram of July 26, 1984, SRV 'J' lifted eight times at approximately five munute intervals to limit the reactor vessel pressure increase caused by decay heat. The relief pressure setpoint of the 'J' SRV was checked and found to be low. The setpoint was adjusted to its proper value. All other SRV relief setpoints were checked. Three additional setpoints were found cut of tolerance and recalibrated.

Changes to the Offsite Dose Calculation Manual

None.

Major Changes to Radioactive Waste Treatment Systems

None.

PPal Pennsylvania Power & Light Company
Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

Bruce D. Kenyon Vice President-Nuclear Operations 215/770-7502

# AUG 14 1984

Director, Data Automation & Management Information Division Attention: Mr. M. R. Beebe Management Information Branch Office of Resource Management U.S. Nuclear Regulatory Commission Washington, D.C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION MONTHLY OPERATING REPORTS ER 100450 FILE 841 PLA-2273

Docket Nos. 50-387/NPF-14 50-388/NPF-22

Dear Mr. Beebe:

The July 1984 monthly operating reports for Susquehanna SES Units 1 and 2 are attached.

Very truly yours,

for B. D. Kenyon

Vice President-Nuclear Operations

#### Attachment

cc: Dr. Thomas E. Murley
Regional Administrator-Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555
Attn: Document Control Desk (12 copies)

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Mr. R. H. Jacobs - NRC Mr. R. L. Perch - NRC

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