

TENNESSEE VALLEY AUTHORITY
DIVISION OF NUCLEAR POWER
SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT
TO THE
NUCLEAR REGULATORY COMMISSION
JULY 1, 1984 - JULY 31, 1984

UNIT 1

DOCKET NUMBER 50-327

LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328

LICENSE NUMBER DPR-79

Submitted By: P.R. Waller
Plant Manager

8409130180 840731
PDR ADOCK 05000327
R PDR

IE24
1/1

TABLE OF CONTENTS

Operations Summary 1

Significant Operational Events 1-3

PORV's and Safety Valves Summary 3

Reports

 Licensee Events 3-5

 Diesel Generator Failure Reports 5

 Special Reports 5

Offsite Dose Calculation Manual Changes 5

Operating Data

 Unit 1 6-8

 Unit 2 9-11

Plant Maintenance Summary 12-21

Operations Summary

July, 1984

The following summary describes the significant operational activities for the month of July. In support of this summary, a chronological log of significant events is included in this report.

Unit 1

Unit 1 was critical for 744.0 hours, produced 831,710 MWH (gross), resulting in an average hourly gross load of 1,117,890 kW during the month. There are 311.8 full power days estimated remaining until the end of cycle 3 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached August 3, 1985. The capacity factor for the month was 94.5 percent.

There were no reactor scrams, manual shutdowns, or power reductions during the month.

Unit 2

Unit 2 was critical for 744.0 hours, produced 699,640 MWH (gross), resulting in an average hourly gross load of 992,608 kW during the month. There are 25.9 full power days estimated remaining until the end of cycle 2 fuel. The cycle 2 refueling outage is scheduled to begin on September 7, 1984. The capacity factor for the month was 79.5 percent.

There were no reactor scrams, manual shutdowns, or power reductions during the month.

Significant Operational Events

Unit 1

<u>Date</u>	<u>Time</u>	<u>Event</u>
07/01/84	0001	The reactor was in mode 1 at 100% power producing 1120 MWe. An investigation was underway to determine the loss generation.
07/09/84	1438	Began reducing power due to both trains of auxiliary air out of service.
	1449	Held reactor power at 88%.
	1458	Began power ascension.
	1800	Reactor obtained 100% power, producing 1118 MWe.

Significant Operational Events

Unit 1

(Continued)

<u>Date</u>	<u>Time</u>	<u>Event</u>
07/31/84	2359	The reactor was in mode 1 at 100% power producing 1118 MWe. The investigation continues to determine the loss of generation.

Unit 2

<u>Date</u>	<u>Time</u>	<u>Event</u>
07/01/84	0001	The reactor was in mode 2 at 2% reactor power. The outage continued for maintenance to feedwater valve 3-526.
07/02/84	1400	Began power ascension
	1419	Reactor entered mode 1.
	1509	The unit was tied on-line.
	1627	The reactor obtained 30% power producing 176 MWe and holding due to steam generator chemistry.
07/06/84	1404	Began power ascension.
	2206	The reactor obtained 42% power and held due to feedwater chemistry.
07/07/84	0854	No. 3 heater drain tank pump A valve 5-540 had a broken stem.
	1255	5-540 repairs completed, began power ascension.
07/08/84	1158	The reactor obtain 100% power producing 1144 MWe.
07/09/84	1438	Began reducing power because both auxiliary air compressors were inoperable.
	1448	B train auxiliary air declared operable. Began power ascension.

Significant Operational Events

(Continued)

Unit 2

(Continued)

<u>Date</u>	<u>Time</u>	<u>Event</u>
7/09/84	1844	The reactor obtain 100% power producing 1145 MWe.
07/31/84	0839	No. 2 throttle valve closed, the reactor reduced to 98% power producing 1121 MWe.
	0935	Began power ascension.
	1100	The reactor obtained 100% power producing 1136 MWe.
	1605	Began reducing power due to the loss of coolant flow to the EHC.
	1630	Held reactor power at 90% producing 1070 MWe.
	1655	Began power ascension.
	2200	The reactor obtained 100% power producing 1140 MWe.
	2400	The reactor was in mode 1 at 100% power producing 1140 MWe.

PORV'S and Safety Valves Summary

No PORV's or safety valves were challenged during the month.

Licensee Events and Special Reports

The following Licensee Event Reports (LER's) were sent during July 1984, to the Nuclear Regulatory Commission.

<u>LER</u>	<u>DESCRIPTION OF EVENT</u>
1-84036	At 2115C on June 18, 1984, unit 1 experienced a reactor trip. Unit 1 was in mode 1 (2235 psig, 478°F) at 100% reactor power just prior to the event. A turbine trip occurred due to a gasket slipping and shorting the generator bus to ground. The electrical trouble alarm caused the turbine to trip and the P-9 interlock caused the reactor to trip. Unit 1 stabilized at 547°F following the reactor trip.

Licensee Events and Special Reports

(Continued)

<u>LER</u>	<u>DESCRIPTION OF EVENT</u>
1-84040	A technical specification surveillance requirement (SR) for a standby diesel generator was not performed within the required time. The SR was to be performed every 18 months to check the diesel overspeed trip lockout circuit. The part of the procedure that performed this SR had been revised to make this SR a five-year requirement. The procedure was immediately revised and the surveillance was performed. The diesel passed the surveillance, and it was returned to service.
1-84041	The pressurizer pressure indicator in the auxiliary control room (ACR) was discovered inoperable by the performance of a monthly surveillance instruction (SI). Upon investigation, it was discovered that a wire had been reterminated incorrectly the last time this indicator was calibrated. The wire was lifted, reterminated in the correct position, and the loop was returned to service. Since the ACR was not required to be in use during this time, the indicator was not relied upon by personnel for the operation of the plant.
1-84042	Due to an incomplete modification, radiation monitor 0-RE-90-225 would not provide automatic isolation of direct releases from the neutralization tank. Two batch releases were made from the tank without compliance with action statement 30 of LCO 3.3.3.9.
1-84043	At 1000C on June 23, 1984, a thermal fire detector in the diesel generator corridor was determined to be inoperable during the performance of surveillance instruction (SI) 234.6, "Tech Spec Fire Detectors." A fire watch is required to be established within one hour and each hour thereafter until the detector is repaired. Due to miscommunication between personnel, the shift engineer (SE) was not notified that the fire detector was inoperable; therefore, the fire watch was never established. Maintenance personnel replaced the fire detector at 1502C on June 26, 1984.
2-84009	On June 30, 1984, maintenance was being performed on main feedwater pump (MFP) 'B'. MFP 'A' was in the reset condition. The two motor-driven auxiliary feedwater pumps (MDAFP) were running. MFP 'A' was accidentally tripped and generated an auto start signal (ESF actuation) for the auxiliary feedwater (AFW). The turbine-driven auxiliary feedwater pump (TDAFP) did not start due to flow control valve (FCV) 1-15, main steam supply, failing to close. FCV-1-15 was repaired and returned to service. There was no effect upon public health or safety, and no plant safety margins were exceeded.

Licensee Events and Special Reports

(Continued)

<u>LER</u>	<u>DESCRIPTION OF EVENT</u>
2-84-010	This event occurred while unit 2 was in mode 1 (100% reactor power, 2235 psig, 578°F). Technical specification 4.6.3.4 requires each containment purge isolation valve to be demonstrated operable within 24 hours after each closing of the valve. Valves 2-FCV-30-9, -10, -52, and -53 were closed at 2020 CDT on June 19, 1984 after an upper containment purge and were not tested until 1005 CDT on June 20, 1984. The failure to test these valves was due to Operations failing to notify the Engineering Test Section that an upper containment purge had been completed. Upon discovery of this condition, the Engineering Test Section was immediately notified, and the valves were tested per applicable procedures.

Diesel Generator Failure Reports

There were no diesel generator failure reports transmitted during the month.

Special Reports

There were no special reports transmitted during the month.

Offsite Dose Calculation Manual Changes

There were no changes in the Sequoyah Nuclear Plant ODCM this month.

OPERATING DATA REPORT

DOCKET NO. 50-327
 DATE AUGUST 7, 1984
 COMPLETED BY M. G. EDDINGS
 TELEPHONE (615) 870-6248

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1 NOTES:
 2. REPORT PERIOD: JULY 1984
 3. LICENSED THERMAL POWER(MWT): 3411.0
 4. NAMEPLATE RATING (GROSS MWE): 1220.6
 5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
 8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: _____

 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): _____

 10. REASONS FOR RESTRICTIONS, IF ANY: _____

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.00	5111.00	27048.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	744.00	2676.10	17117.66
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	744.00	2521.80	16634.95
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	2533527.07	7634988.97	53126839.27
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	831710.00	2499950.00	17879086.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	803282.00	2391807.00	17168735.00
19. UNIT SERVICE FACTOR	100.00	49.34	61.50
20. UNIT AVAILABILITY FACTOR	100.00	49.34	61.50
21. UNIT CAPACITY FACTOR (USING MDC NET)	94.05	40.76	55.29
22. UNIT CAPACITY FACTOR (USING DER NET)	94.05	40.76	55.29
23. UNIT FORCED OUTAGE RATE	0.00	32.74	20.60
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):	_____		
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:	_____		

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-327
 UNIT NAME Sequoyah One
 DATE August 9, 1984
 COMPLETED BY M. Eddings
 TELEPHONE (615) 870-6248

REPORT MONTH JULY

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method Of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
									No Shutdowns or power reductions during month.

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Cont. of Existing Outage
 5-Reduction
 9-Other

4
 Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
 Exhibit I-Same Source

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-327
 UNIT Sequoyah One
 DATE August 9, 1984
 COMPLETED BY M. Eddings
 TELEPHONE (615)870-6248

MONTH JULY

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>1086.2</u>	17	<u>1077.5</u>
2	<u>1084.3</u>	18	<u>1100.5</u>
3	<u>1087.8</u>	19	<u>1079.2</u>
4	<u>1090.2</u>	20	<u>1077.0</u>
5	<u>1087.0</u>	21	<u>1080.8</u>
6	<u>1082.7</u>	22	<u>1089.2</u>
7	<u>1082.8</u>	23	<u>1077.8</u>
8	<u>1084.8</u>	24	<u>1077.3</u>
9	<u>1070.2</u>	25	<u>1077.0</u>
10	<u>1082.2</u>	26	<u>1074.0</u>
11	<u>1083.2</u>	27	<u>1074.0</u>
12	<u>1082.8</u>	28	<u>1064.8</u>
13	<u>1084.3</u>	29	<u>1077.0</u>
14	<u>1082.5</u>	30	<u>1075.2</u>
15	<u>1078.7</u>	31	<u>1069.2</u>
16	<u>1079.0</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-328
 DATE AUGUST 7, 1984
 COMPLETED BY D.C. DUPREE
 TELEPHONE (615)870-6248

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2
2. REPORT PERIOD: JULY 1-31, 1984
3. LICENSED THERMAL POWER (MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: _____

NOTES:

Item 23 YR-TO-DTD and Cumulative values are corrected. The outage in June was designated a scheduled outage.

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.00	5111.00	19008.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	744.00	5035.40	13396.47
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	704.85	4959.07	15113.39
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	2105415.32	16351509.85	48769577.66
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	699640.00	5600290.00	16632230.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	673665.00	5397464.00	16015201.60
19. UNIT SERVICE FACTOR	94.74	97.03	79.51
20. UNIT AVAILABILITY FACTOR	94.74	97.03	79.51
21. UNIT CAPACITY FACTOR (USING MDC NET)	78.87	91.99	73.39
22. UNIT CAPACITY FACTOR (USING DER NET)	78.87	91.99	73.39
23. UNIT FORCED OUTAGE RATE	0.00	1.87	6.96
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): <u>Refueling/Modification - Cycle 2 September 7, 1984 57 days.</u>			

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

NOTE THAT THE THE YR.-TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-328
 UNIT NAME Sequoyah Two
 DATE August 7, 1984
 COMPLETED BY D. C. Dupree
 TELEPHONE (615) 870-6248

REPORT MONTH JULY

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method Of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
5	840701	S	39.15	B	4				Manually shutdown to repair feedwater drain valve 3-526. Reactor at 2%.

-10-

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Cont. of Existing Outage
 5-Reduction
 9-Other

4
 Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
 Exhibit I-Same Source

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-328
 UNIT Sequoyah Two
 DATE August 7, 1984
 COMPLETED BY D. C. Dupree
 TELEPHONE (615) 870-6248

MONTH JULY

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	1103
2	231	18	1105
3	251	19	1106
4	259	20	1106
5	254	21	1106
6	297	22	1106
7	506	23	1108
8	1042	24	1103
9	1102	25	1101
10	1109	26	1097
11	1107	27	1098
12	1107	28	1101
13	1106	29	1101
14	1106	30	1102
15	1105	31	1087
16	1105		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

Plant Maintenance Summary

The following significant maintenance items were completed during the month of July 1984:

Mechanical Maintenance

1. Replaced the bonnet on 2-VLV-3-524 and injected the drain line with furmanite to stop the leakage. Furmanited several valves on both units on systems #1, 3, 5, & 6. These were both CSSC and Non-CSSC valves.
2. Repaired a bent stem on 2-LCV-6-106A.
3. Replaced a broken linkage that engages the turning gear on the unit 2 main turbine.
4. After being cut in half, the "2A" mixed bed demin tank was welded back together and the welds x-rayed. Indications were ground out and rewelded. Piping to the tank were welded back and some welds were x-rayed. All welds subsequently passed the x-raying.
5. The "2D" cooling tower lift pump was pulled, all bearing and one sleeve were replaced, and the pump was reinstalled.
6. Two Allis-Chalmers pumps were installed in the location of the boric acid evaporator "B" concentrate pumps.
7. Replaced a damaged motor on the 2A-A centrifugal charging pump.
8. Repaired the elevation 734 airlock on unit 2 reactor building.
9. "B-B" auxiliary air compressor was reported to be making a "knocking" noise. Gaskets were replaced on this compressor. "A-A" compressor was also disassembled for maintenance and found to have a broken crankshaft. The crankshaft was replaced and the compressor was completely rebuilt.
10. Lubricated the coupling and aligned the 1B-B containment spray pump.
11. Unplugged the sample line from the unit 2 gross fail fuel detector to the hot sample room.
12. Tightened the packing on the 2A-A and 2B-B motor driven auxiliary feedwater pumps.
13. Oil was added to 2-PCV-3-132 which controls the output pressure on the auxiliary feedwater pumps.
14. Replaced the solenoid on 1-FSV-3-175.
15. Performed SI-102 on the 2A-A diesel generator (annual inspection).

DATE.	COMPONENT.....	FAILURE DESCRIPTION.....	ELECTRICAL MAINTENANCE CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	PROM.....
06-13	2-MTRB-030-003 9-B	CONTAINMENT AIR RETURN TIMER ON 2B2-B 480 VOLT SHUT DOWN BOARD FAILED ACCEPTANCE WHILE PERFORMING SI28	BADAGASTAT TIMER	REPLACED TIMER AND SET FOR 10 MINUTES CHECKED TIMER AND RETURNED TO SERVICE	NONE
06-14	1-MTRB-313-JE/ 11D-A	OIL LOW AND FREON LOW SHOWING ON GAGES ON 480 VOLT BOARD ROOM CHILLER 1A	OIL AND FREON LINES LEAKING	REPAIRED LEAKS AND ADDED 1.5 GAL OF 4CS OIL AND FILLED TO PROPER LEVEL WITH R-22 FREON	NONE
06-14	2-MTRB-030-045 2A-A	DIESEL GENERATOR 2A ROOM EXHAUST FAN 2A-A CONTROLS KEEP BLOWING FUSES	DAMPER MOTOR BAD STARTER WINDING SHORTED OUT	REPLACED MOTOR STARTER DAMPER MOTOR AND CRYDON RELAY PER M&AI 12	NONE
06-21	1-DCRP-082-000 18-B	ALARM HORNE ON PANELOF GENERATOR 18-B CONTROL BOARD DOES NOT WORK	HORNE DEFECTIVE	REPLACED HORN AND PERFORMED MI6.20 AND M&AI 12 OPERATIONS PERFORMED SI 7.1	NONE
07-04	2-BCTD-001-001 5-A	MOTOR THERMAL OVERLOADS KICKING OUT ON BOARD ROOM 2A2-A/19A U(2) WEST VALVE ROOM ISOLATION VALVE 2-FCV-1-15	PROBABLE CAUSE WAS SURGE IN POWER	BRIDGED AND MEGGARED OVERLOADS FOR PROPER SIZE PER SI 251.2 EVERY THING CHECKED OK	NONE
07-06	2-BDB-201-DN-A	LIGHT GOES OUT ON 480 VOLT SHUTDOWN BOARD 2A1-A IN AUX BUILDING WHEN GROUND DETECTS ARE PUSHED	C-PHASE WAS SHOWING DEAD GROUND PROBABLE CAUSED FROM IMPROPER WAY OF PUSHING IN ON THE GROUND DETECT	CHECKED GROUND INDICATORS SEVERAL TIMES NO GROUND COULD BE LOCATED CLEANED AND RETURNED TO SERVICE	NONE
07-07	1-GENB-082-000 1A-A	PUSH BUTTON TEST SWITCH DOES NOT WORK EACH TIME IT IS PUSHED ON ANNUNCIATION CONTROL POWER FAILURE BLOWN FUSE	CONTACTS BAD ON POWER RELAY	REPLACED POWER ON RELAY CHECKED ANNUNCIATION LIGHTS AND RELAY DIESEL GENERATOR WAS NOT INOPERABLE AT ANY TIME	NONE
07-11	0-MTRB-311-017 18-B	AIR BUBBLES IN SIGHT GLASS MASE LOW ON FREON	AIR IN REFRIGENENT GAS	CHECKED PRESSURE IN SIGHT GLASS OPERATING PROPERLY AT THIS TIME	NONE
07-11	0-MTRB-311-017	AIR BUBBLES IN SIGHT	AIR IN REFRIGENENT GAS	CHECKED PRESSURE IN SIGHT	NONE

ELECTRICAL MAINTENANCE

Page 2 of 2

DATE.	COMPONENT.....	FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	PROM....
	18-B	GLASS MABE LOW ON FREON		GLASS OPERATING PROPERLY	
07-15	1-ZS-062-0144	OPEN LIMIT ON MAKE UP CHARGING PUMP WILL NOT CLEAR	VALVE ON MAKE UP CHARGING PUMP WILL NOT FULLY CLOSE POSSIBLE FROM CORROSION OR RUST	VALVE OPENED AND CLOSED PROPERLY WITH A FUNCTIONAL TEST STATUS LIGHTS CORRESPONDED TO VALVE POSITION	NONE
07-16	1-BATA-082-UC-B	CELLS 54 AND 25 IN DIESEL GENERATOR BATTERY 18-B ARE READING IN EXCESS OF 150X10/6 OHMS	STRAP CONNECTIONS CORRODED AND DIRTY AND LOOSE	CLEANED AND RETORQUED CONNECTIONS PERFORMED SI 238 READINGS ARE ACCEPTABLE	NONE

11 records listed.

Plant Maintenance Summary

(Continued)

Instrument Maintenance

1. During the month, five containment vent isolations occurred. Four of the five were on unit 2. These occurred during the first part of the month before time delay relays were installed (ECN 6095) in radiation monitoring circuits. The relays will prevent inadvertent actuation from spurious EMI which occurs in the detector signals.
2. The monthly calibration check of the UHI level switches on both units found all switches except 2-LS-87-22 within Tech Spec tolerance (reference PRO 2-84-107). DCR P-2111 is being processed to replace the Barton Model 288A switches with a more reliable instrument.
3. Unit 1 Post Accident Monitoring wide range reactor coolant system pressure channel P-68-69 was changed from 0-600 psi to 0-3000 psi by TACF 1-84-83-68. The 0-3000 psi indication for this channel was replaced during the refueling outage leaving only the 0-600 psi indicator as the PAM 2 channel (reference PRO 1-84-276). An FCR to ECN 6055 is being prepared to add another wide range indicator so the narrow range indicator can be returned to its original range.
4. Unit 2 P-250 computer was out of service most of the day on July 23, 1984. Problems were encountered with the random access disk memory (RAD) controller. Four circuit cards were replaced and a disk head was spared out. Additional diagnostic testing is planned during the upcoming refueling outage.
5. Unit 2 throttle valve no. 2 drifted close on July 31, 1984. During a transient on the EHC oil system, an EHC oil pump failed. The high pressure filter, an unloader valve, and both oil pumps were replaced. The no. 2 throttle valve remains closed and replacement of the Moog Servo on the dump valve is planned during the upcoming refueling outage.
6. Other maintenance work is shown on the attached list.

INSTRUMENT MAINTENANCE

INSTRUMENT MAINTENANCE MONTHLY SUMMARY 08-08-84

PAGE 1

COMP

MR.	COMP	U	FUNC	SYS	ADDRESS.	DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A121280	2	FIS	074	24		07/13/84	2-FIS-074-24, GAUGE FLUCTUATES (BOUNCES) FROM 0-900 GPM WITH PUMP RUNNING. CHECK CALIBRATION AND PROPER OPERATION OF GAUGE.	THE FLOW INDICATOR SWITCH WAS FOUND OUT OF CALIBRATION. RECALIBRATED THE SWITCH AND RETURNED TO SERVICE.
A233371	2	LT	063	176		07/06/84	2-LT-063-176, CHECK CALIBRATION	A CALIBRATION CHECK WAS REQUESTED DUE TO THE TEST EQUIPMENT USED BEING OUT OF CAL. DATE. VERIFIED CALIBRATION AND RETURNED TO SERVICE.
A238678	0	PS	032	88		07/09/84	0-PS-032-88, RECALIBRATE INSTRUMENT TO 80 PSIG DECR	A REQUEST WAS MADE TO LOWER THE INSTRUMENT FROM 84 PSIG TO 80 PSIG DUE TO THE PRESSURE OPERATED RELIEF VALVES BEING SET AT 80 PSIG. RECALIBRATED THE SETPOINT AND RETURNED TO SERVICE TACF NO-84-79-32.
A238679	0	PS	032	62		07/09/84	0-PS-032-62, RECALIBRATE INSTRUMENT TO 80 PSIG DECR	A REQUEST WAS MADE TO LOWER THE INSTRUMENT FROM 84 PSIG TO 80 PSIG DUE TO THE PRESSURE OPERATED RELIEF VALVES BEING SET AT 80 PSIG. RECALIBRATED THE SET POINT AND RETURNED TO SERVICE. TACF NO-84-79-32
A238767	1	TI	068	44E		07/18/84	1-TI-068-44E, HAS FAIL - INVESTIGATE AND CORRECT.	THE TEMPERATURE INDICATOR WAS FAILING DUE TO TM-68-44P, TM-68-44Q BEING OUT OF CALIBRATION. RECALIBRATED TM-68-44P, TM-68-44E, AND TM-68-44Q, VERIFIED OPERATION AND RETURNED TO SERVICE.
A238800	1	LCV	003	171A		07/24/84	1-LCV-003-171A, LCV-3-1A WILL NOT COME FULLY OPEN. INVESTIGATE AND CORRECT CAUSE OF MALFUNCTION.	THE CURRENT TO PRESSURE TRANSDUCER WAS FOUND HAD DUE TO CYCLIC FATIGUE. REPLACED THE TRANSDUCER, CALIBRATED, VERIFIED OPERATION, AND RETURNED TO SERVICE.
A239274	2	LI	068	312C		07/02/84	2-LI-068-312C, INDICATING 58% WITH CR INDICATING AT 64%.	THE LEVEL TRANSMITTER WAS FOUND OUT OF CALIBRATION DUE TO NATURAL CAL. DRIFT. RECALIBRATED THE TRANSMITTER AND RETURNED TO SERVICE.
A239316	2	LI	068	300		07/02/84	2-LI-068-300, MCR INDICATES HIGHER THAN INDICATING IN AUX. CONTROL RM.	ON INVESTIGATION THE LEVEL TRANSMITTER WAS FOUND OUT OF CALIBRATION DUE TO NATURAL DRIFT. RECALIBRATED THE TRANSMITTER BACKFILLED AND BLED THE SENSOR BELLOWS AND RETURNED TO SERVICE.
A239884	1	PS	003			07/27/84	1-PS-003-, VERIFY CAL. OF SWITCHES, MAKE REPAIRS AND ADJUSTMENTS, REPLACE SWITCHES IF NECESSARY.	THE SWITCH WAS FOUND INOPERABLE. EXERCIZED THE SWITCH, RECALIBRATED,

-16-

INSTRUMENT MAINTENANCE

INSTRUMENT MAINTENANCE MONTHLY SUMMARY 08-08-84 PAGE 1

MR. COMP	U	FUNC	SYS	ADDRESS	DATE	DESCRIPTION	CORRECTIVE ACTION
A238785	2	FI	063	76	07/26/84	2-FI-063-76, CHECK CALIBRATION. GAUGE SHOWING NO FLOW. FREED ROTOMETER INDICATOR BEING STUCK. FREED THE INDICATOR, VERIFIED OPERATION, AND RETURNED TO SERVICE.	THE GAUGE WAS SHOWING NO FLOW DUE TO THE ROTOMETER INDICATOR BEING STUCK. FREED THE INDICATOR, VERIFIED OPERATION, AND RETURNED TO SERVICE.
A241496	2	MTRA	062	104B	07/17/84	2-MTRA-062-104B, INSTALL TC LEAD WIRE FROM THE MOTOR JUNCTION BOX BEARING TCS TO AN AREA OUTSIDE OF THE C-ZONE SO THAT BEARING TEMPERATURES MAY BE CHECKED PERIODICALLY	A REQUEST WAS MADE TO INSTALL THERMOCOUPLE LEADS TO THE OUTSIDE OF CONTAINMENT TO VERIFY BEARING TEMPERATURE. INSTALLED THE LEADS AND VERIFIED OPERATION. TACF #2-84-2016-62
A241497	2	MTRA	062	108A	07/17/84	2-MTRA-062-108A, INSTALL TC LEAD WIRE FROM THE MOTOR JUNCTION BOX BEARING TCS TO AN AREA OUTSIDE OF THE C-ZONE SO THAT BEARING TEMPERATURES MAY BE CHECKED PERIODICALLY	A REQUEST WAS MADE TO INSTALL THERMOCOUPLE LEADS TO THE OUTSIDE OF CONTAINMENT TO VERIFY BEARING TEMPERATURE. INSTALLED THE LEADS AND VERIFIED OPERATION TACF #2-84-2016-62
A241498	1	MTRA	062	104B	07/17/84	1-MTRA-062-104B, INSTALL TC LEAD WIRES FROM MOTOR J-BOX BEARINGS TCS TO AN AREA OUTSIDE OF THE C-ZONE SO THAT BEARING TEMPERATURES MAY BE CHECKED PERIODICALLY.	A REQUEST WAS MADE TO INSTALL THERMOCOUPLE LEADS TO THE OUTSIDE OF CONTAINMENT TO VERIFY BEARING TEMPERATURE. INSTALLED THE LEADS AND VERIFIED OPERATION TACF #1-84-81-62
A243861	1	MTRA	062	108A	07/17/84	1-MTRA-062-108A, INSTALL TC LEAD WIRES FROM MOTOR J-BOX BEARING TC'S TO AN AREA OUTSIDE OF THE C-ZONE SO THAT BEARING TEMPERATURES MAY BE CHECKED PERIODICALLY. USE NI	A REQUEST WAS MADE TO INSTALL THERMOCOUPLE LEADS TO THE OUTSIDE OF CONTAINMENT TO VERIFY BEARING TEMPERATURE. INSTALLED THE LEADS AND VERIFIED OPERATION TACF #1-84-81-62

5 records listed.

INSTRUMENT MAINTENANCE

INSTRUMENT MAINTENANCE MONTHLY SUMMARY 08-08-84

PAGE 2

MR.	COMP	U	FUNC	SYS	ADDRESS.	DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A281658	2	LI	063	176		07/03/84	2-LI-063-176, CONTAINMENT SUMP LEVEL INDICATOR READING 4% WITH NO LEVEL	VERIFIED OPERATION, AND RETURNED TO SERVICE. THE SENSOR BELLOUS WERE FOUND BAD DUE TO CYCLIC FATIGUE. REPLACED THE BELLOUS, BACKFILLED THE SYSTEM RECALIBRATED AND TIME RESPONDED AND RETURNED TO SERVICE.
A282145	1	PS	003	165B		07/28/84	1-PS-003-165B, REPLACE BROKEN LUG ON NC2 BLACK AND WHITE WIRE.	THE LUG WAS FOUND BROKEN TO THE PRESSURE SWITCH RESULTING IN A DEFECTIVE CONNECTION. REPLACED THE LUG VERIFIED SWITCH OPERATION, AND RETURNED TO SERVICE.
A282978	1	LI	063	179		07/10/84	1-LI-063-179, INDICATOR HAS APPEARED TO HAVE FAILED OFFSCALE LOW. CHECK CALIURATION	THE BELLOUS IN THE LEVEL TRANSMITTER WERE FOUND BAD DUE TO CYCLIC FATIGUE. REPLACED THE BELLOUS, FILLED SYSTEM, RECALIBRATED, AND RETURNED TO SERVICE.
A285345	1	PI	068	66		07/27/84	1-PI-068-66, RESCALE 1-PI-68-66 FROM 0 TO 600 PSIG TO 0-3000 PSIG.	THE SYSTEM ENGINEER REQUESTED A PRESSURE INCREASE FROM 600 PSIG TO 3000 PSIG ON THE INDICATOR. INCREASED THE PRESSURE RANGE AND RESCALED THE ASSOCIATED PRESSURE MODIFIER.
A289541	2	LCV	003	156		07/11/84	2-LCV-003-156, POSITIONER NEEDS TO BE ADJUSTED.	THE STEM CONNECTOR CLAMP WAS LOOSE WHICH CAUSED THE LIMIT SWITCH ACTUATER ARM TO BEND CAUSING MECHANICAL DAMAGE. TIGHTENED THE CLAMP, REPAIRED THE LIMIT SWITCH ARM, VERIFIED VALVE STROKE, AND RETURNED TO SERVICE.
A289678	1	LT	063	99		07/30/84	1-LT-063-99, LI HAS FAILED HI	THE TRANSMITTER HAD A HIGH OUTPUT DUE TO CALIBRATION DRIFT. RECALIBRATED THE TRANSMITTER AND RETURNED TO SERVICE.
A292046	1	LCV	003	148		08/03/84	1-LCV-003-148, THE VALVE IS LEAKING THROUGH. *NPRD*	THE RELAY WAS FOUND LEAKING ON THE VALVE POSITIONER. REPLACED THE RELAY AND VERIFIED THE STROKE AND RETURNED TO SERVICE.
A293095	0	RM	090	206		07/25/84	0-RM-090-206, RAD MONITOR IS SHOWING 0 COUNTS. INVESTIGATE AND REPAIR AS NECESSARY. HAS MALFUNCTION ALARM IN ON 0-M-12	THE MONITOR WAS SHOWING 0 COUNTS DUE TO A LOOSE PLUG. TIGHTENED THE PLUG, VERIFIED OPERATION AND RETURNED TO SERVICE.
A293096	0	RM	090	141		07/25/84	0-RM-090-141, RAD MONITOR FOR ERCW 0-RM-90-141 HAS FAILED LOW WITH NOT ANY POWER INDICATIONS.	THE RADIATION MONITOR HAS FAILED LOW DUE TO A BAD POWER SUPPLY. REPLACED THE POWER SUPPLY WITH A SPARE WHICH WAS REPAIRED ON ANOTHER MR.

Plant Maintenance Summary

(Continued)

Modifications Group

1. ECN 5237- Laundry Facility

The work on the HVAC continued. The duct work was completed except testing and completion of the air plenum around the dry cleaners. Work on the chiller package continues.

2. ECN 5596 - Batch Neutralization System

The access platform was completed. Heat trace functional testing to be performed when pipe insulation is completed.

3. ECN 5645 - Steam Generator Blowdown System - Unit 2

All pre-outage piping fabrication is complete. Instrumentation installation continues. Conduit work and cable pulling continues.

4. ECN 2768 - Reactor Pressure Vessel Level Indication System (RVLIS) (Unit 2)

Writing and approval of four mechanical workplans was completed. Two workplans remain in the approval cycle. Pre-outage work has started on two workplans. HVAC work continued and is 90 percent complete. Miscellaneous tubing work was performed. The fire protection for the room was completed except tie-in hydro.

Installation of conduit outside containment began. The preparation of the work plan for cable pulling and cable termination continued. The procurement for all electrical materials continued. The preparation of the workplan for inside containment is in progress. Rework of existing conduit cables over 2-R-148 are complete awaiting support from instrument maintenance to verify operability of loop 68-398.

5. ECN 6055 - Wide Range Pressure Transmitter to the RVLIS Panel

Mechanical workplan writing is in progress. Conduit work continues. Cable pulling and termination workplans are being prepared. ENDES and instrument maintenance are making changes to the unit 2 control room indicators.

6. ECN 5194 - Iodine Monitoring Building

Mechanical work is complete with exception of the control air tie-ins which are scheduled for the unit 2 cycle 2 refueling outage. Conduit and cable pulling is in progress.

7. ECN 5024 - Lay-up Water Treatment

Pre-outage fabrication of piping was started. Core drilling of the air intake room was completed. Construction continues their prefabrication activities associated with the pumps.

Plant Maintenance Summary

(Continued)

Modifications Group

(Continued)

8. ECN 5009 - ERCW Piping Change-Out

Prefabrication of piping for the installation of stainless steel pipe for the centrifugal charging pump room coolers was started. Installation is planned for the Unit 2 cycle 2 refueling outage.

9. ECN 5202 - Interfacing the Diesel Generator Buildings and the Power House

Workplan preparation was started for the ERCW tie-ins required for the diesel generators.

10. ECN 5842 - Replacement of PCV-3-122 and -132

Workplan preparation and approval was completed for hangers. Work implementation is scheduled for unit 2 cycle 2 refueling outage.

11. ECN 5938 - Feedwater Heater Tube Change-Out

Work was completed on pre-outage work involving platforms and rigging devices. Workplan preparation and approval was completed.

12. NUREG 0588

ECN L5457 Solenoids were changed out. Remaining items are scheduled for the outage.
ECN L6090 Installation of quick exhaust valves was completed.
ECN L5895 Workplan preparation and approval was completed.
ECN L6032 Workplan preparation and approval was completed.
ECN L5883 Workplan preparation and approval was completed. The workplan remains in the approval cycle.
ECN L5881 Workplan approved. Work began on fabricating brackets for limit switches.
ECN L5823 Workplan approved. Awaiting craft support.
ECN L5883 Flow switches and press switches workplans in preparation.
ECN L5824 One workplan for valve operator replacement has been completed and two to 125 valve operators have been replaced.
ECN L5370 Workplan 10617 - 15 of 16 motors replaced. Sixteenth motor is awaiting repair or to be transferred in from Hartsville.

Flow transmitter for L5884 has been reordered and workplan 10777 is in approval cycle.

13. ECN 6182

Workplan preparation was started and contractor scheduled to arrive during August.

Plant Maintenance Summary

(Continued)

Modifications Group

(Continued)

14. ECN 5664 - Wells Fargo AS-24 Cards Relay Change-Out
Field work on the boxes is approximately 65% complete. CAS/SAS terminations is 10% complete. Cable pulling is in progress.
15. ECN 5172 & 5968 - Emergency Lighting-SDFWP, AFP, FW REG VLVS and MS Power Operated Relief VLV Areas
Installation of new conduit, strobe lights, and sirens continued. Estimated completion of WP 10578 is around mid October if worked during upcoming unit 2 outage.
16. ECN 5770 - Installation of New Rad Monitor in the Condenser Exhaust
Fifteen of sixteen complete. Awaiting repair or replacement of the 16th.
17. ECN 5867 - Unit 2 Spend Fuel Transfer System
Westinghouse modifications are complete on the pit side and work has commenced on the reactor side.
18. ECN 5871 - Relocation of the ERCW Rad Monitor
"B" Train ERCW Rad Monitor is in service. "A" Train work is completed. Instrument maintenance is expected to be complete by August 10, 1984.
19. ECN L2780- Post Accident Sampling Facility Unit 2
The conduit work outside containment is approximately 98% complete. Cable pulling is approximately 65% complete. Cable termination began this period. Internal PNL wiring in various PNLs has begun and is approximately 40% complete.
20. ECN L5198 - Technical Support Center (TSC) Unit 2
Modification of the status monitoring system (SMS) cabinets has been completed. The control building conduit installation on elevation 685 continued and should be completed by mid August. DPM 50 and P250 tie work is continuing as well as cable installation.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
P. O. Box 2000
Soddy-Daisy, Tennessee 37379

August 14, 1984

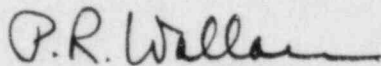
Nuclear Regulatory Commission
Office of Management Information
and Program Control
Washington, DC 20555

Gentlemen:

Enclosed is the July 1984 Monthly Operating Report to the NRC for Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



P. R. Wallace
Plant Manager

Enclosure

cc (Enclosure):

Director, Region II
Nuclear Regulatory Commission
Office of Inspection and Enforcement
101 Marietta Street
Suite 3100
Atlanta, GA 30323 (1 copy)

Director, Office of Inspection
and Enforcement
Nuclear Regulatory Commission
Washington, DC 20555 (10 copies)

Mr. A. Rubio, Director
Electric Power Research Institute
P. O. Box 10412
Palo Alto, CA 94304 (1 copy)

Mr. R. C. Goodspeed
MNC 461
Westinghouse Electric Corporation
P. O. Box 355
Pittsburgh, PA 15230 (1 copy)

Director, Office of Management
Information and Program Control
Nuclear Regulatory Commission
Washington, DC 20555 (2 copies)

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

IE24
/