

NOTICE OF VIOLATION

Wisconsin Electric Power
Company
Point Beach Nuclear Plant -
Units 1 and 2

Docket Nos. 50-266; 50-301
Licenses No. DPR-24; No. DPR-27

During an NRC inspection on April 20 through 24, 1992, two violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (1991), the violations are listed below:

- A. 10 CFR 50.55a.(g)(4)(ii), states that "inservice tests to verify operational readiness of pumps and valves . . . shall comply with the requirements of the latest edition and addenda of the Code . . ." ASME Code, Section XI, IWP-3230, "Corrective Action," Section (b) states that, "If deviations fall within the Required Action Range . . . the pump shall be declared inoperative . . ."

Contrary to the above, inservice test procedure IT-07 was performed on January 3, 1992, and subsequently evaluated on January 6, 1992. This evaluation revealed that service water pumps P32E and P32F were in the "Required Action" range for high differential pressure; however, the pumps were not declared inoperable (266/92008-04(DRS); 301/92008-04(DRS)).

This is a Severity Level IV violation (Supplement I.D.3).

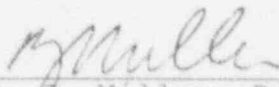
- B. 10 CFR 50, Appendix B, Criterion III, Design Control, requires that, "Design control measures shall be applied to items such as . . . delineation of acceptance criteria for inspection and tests." It continues, "Design changes, including field changes, shall be subject to design control measures commensurate with those applied to the original design and approved by the organization that performed the original design . . ."

Contrary to the above, as of April 24, 1992, the licensee specified inservice testing acceptance criteria for pumps and valves in PBNP 4.12.17, "Point Beach Standing Orders", Attachments E, F, and G; however, changes to the criteria did not require review by the originating organization. Furthermore, changes in the IST acceptance criteria did not require design control measures commensurate with those originally applied (266/92008-03(DRS); 301/92008-03(DRS)).

This is a Severity Level IV violation (Supplement I.D.3).

Pursuant to the provisions of 10 CFR 2.201, Wisconsin Electric Power Company is hereby required to submit a written statement or explanation to the U. S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the U.S. Nuclear Regulatory Commission, Region III, 799 Roosevelt Road, Glen Ellyn, Illinois, 60137, and a copy to the NRC Resident Inspector at the Point Beach Nuclear Power Plant, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violation, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order or a demand for information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Dated at Glen Ellyn, Illinois
this 21st day of May, 1992



H. S. Miller, Director
Division of Reactor Safety