



# Public Service®

16805 WCR 19 1/2, Platteville, CO 80651

May 19, 1992  
Fort St. Vrain  
Unit No. 1  
P-92196

Public Service  
Company of Colorado  
P.O. BOX 840  
DENVER CO 80201-0840

A. Clegg Crawford  
Vice President  
Electric Production

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D. C. 20355

ATTN: Dr. Seymour H. Weiss, Director  
Non-Power Reactor, Decommissioning and  
Environmental Project Directorate

Docket No. 50-267

**SUBJECT: Request for Early Approval of One Provision of the  
Decommissioning Technical Specifications**

**REFERENCE: PSC Letter, Crawford to Weiss, dated March 19, 1992  
(P-92115)**

Dear Dr. Weiss:

This letter supplements Public Service Company of Colorado's (PSC's) previous request for approval of the Fort St. Vrain (FSV) Decommissioning Technical Specifications (DTS), submitted via the referenced letter. PSC specifically requests early consideration of the DTS provision that deletes current limitations on temperatures of the Prestressed Concrete Reactor Vessel (PCR) liner cooling water system.

PCR liner cooling water system temperature limits are currently provided in LCO 4.2.15 in the FSV Technical Specifications. The DTS do not include requirements for PCR liner cooling water system temperatures. PSC requests that the NRC take early action to approve this provision of the DTS by approving the deletion of LCO 4.2.15 from the current FSV Technical Specifications.

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PSC requests early consideration of the deletion of PCRV temperature limits to reduce the heat being added to the Reactor Building, and thereby allow a cooler work environment for workers in that area. LCO 4.2.15 currently includes an 85 degree F minimum temperature limit on average PCRV liner cooling water system inlet and outlet temperatures. This requires the addition of heat from steam coils, which results in uncomfortably hot conditions for workers on the refueling floor of the Reactor Building. These hot conditions are further degraded by warm summer weather and are not conducive to peak personnel performance during the remaining fuel removal and plant closure activities.

The temperature limits in LCO 4.2.15 minimized temperature variations within the PCRV concrete during the entire range of operating and shutdown conditions, thereby ensuring its structural integrity. PSC has completed the removal of all nuclear fuel from the PCRV, so there is no longer a need for the PCRV to contain the fission products that could be released from an active core. The PCRV's current functions are to contain the remaining activity and to provide shielding for workers. These functions are identical to the functions that the PCRV will perform during decommissioning. The ability of the PCRV to perform these functions would not be compromised if its temperature was allowed to cool and reach equilibrium with the ambient temperature in the Reactor Building.

As discussed above, PSC requests that the requirements of LCO 4.2.15 be deleted, consistent with the DTS. Accordingly, the following attachments are provided:

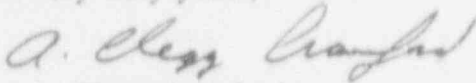
Attachment 1	Summary of Proposed Change
Attachment 2	Proposed Change
Attachment 3	Supporting Safety Analysis

The attached safety analysis provides supporting justification for the early action requested. A No Significant Hazards Consideration Analysis in accordance with the standards in 10 CFR 50.92, was provided for the DTS submittal in the referenced letter. PSC's decommissioning authorization and the accompanying revision to the Technical Specifications were noticed in the Federal Register (57 FR 8940) and no public comments were received. In the absence of public comments, PSC requests that this early consideration be approved as soon as reasonably practical, to improve the work environment inside the FSV Reactor Building.

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If you have any questions regarding this matter, please contact Mr. M. H. Holmes at (303) 620-1701.

Very truly yours,



A. Clegg Crawford  
Vice President  
Electric Production

ACC/SWC/lmg

cc: Regional Administrator, Region IV

Mr. J. B. Baird  
Senior Resident Inspector  
Fort St. Vrain

Mr. Robert M. Quillin, Director  
Radiation Control Division  
Colorado Department of Health

BEFORE THE  
UNITED STATES NUCLEAR REGULATORY COMMISSION

In the Matter of the Facility Operating License)  
of )  
PUBLIC SERVICE COMPANY OF COLORADO )

Application for Amendment to  
the Facility Operating License  
License No. DPR-34

OF THE  
PUBLIC SERVICE COMPANY OF COLORADO  
FOR THE  
FORT ST. VRAIN NUCLEAR GENERATING STATION

This application for Amendment to the  
Facility Operating License, License No. DPR-34,  
is submitted for NRC review and approval.

Respectfully submitted,

PUBLIC SERVICE COMPANY OF COLORADO

By A. Clegg Crawford  
Vice President  
Electric Production

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