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May 21, 1992

U.S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

Attention: Document Control Desk

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-29
RWCU Isolation Due to Personnel Error
LER 92-005-00

GNRO-92/00057

Gentlemen:

Attached is Licensee Event Report (LER) 92-005 which is a final report.

Yours truly,

WTC/RSJ/cg
attachment

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NRC Form 386 (9-83)										U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3160-0104 EXPIRES 8-31-88									
LICENSEE EVENT REPORT (LER)																			
FACILITY NAME (1)										DOCKET NUMBER (2)					PAGE (3)				
Grand Gulf Nuclear Station										0 5 0 0 0 4 1 6 1					OF 0 4				
TITLE (4)																			
RWCU Isolation Due To Personnel Error																			
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES					DOCKET NUMBER(S)					
0	4	21	92	005	00	0	5	21	N/A					0 5 0 0 0					
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OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)																	
	5	20.402(b)			20.405(c)			<input checked="" type="checkbox"/> 50.73(a)(2)(iv)			73.71(b)								
POWER LEVEL (10)		20.406(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)								
		20.406(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 386A.)								
		20.406(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(vii)(A)											
		20.406(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(vii)(B)											
		20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12)																			
NAME										TELEPHONE NUMBER									
Ramon S. Johnson / Engineering Assistant I										6 0 1 4 3 7 -2 3 7 5									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS										
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)									
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO									
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																			
<p>On April 21, 1992 at 2352 hours, an inadvertent reactor water cleanup (RWCU) system isolation occurred during performance of a scheduled reactor water sample valve surveillance. This unplanned isolation was due to personnel error resulting from inattention to detail.</p> <p>The assisting Nuclear Operator A (NOA) was briefed on assigned surveillance step sequences and proceeded to the designated switch panels to place the reactor water sample valve logic test switches to the 'TEST' position. The NOA inadvertently manipulated the RWCU system logic test switches instead.</p> <p>The <u>Conduct of Operations and Control and Use of Operations Section</u> procedures have been changed to provide self-verification guidelines for use during performance of operating and surveillance procedures, valve line-up manipulations, and red tag clearances.</p>																			

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104
EXPIRES 8/31/86

FACILITY NAME (1) Grand Gulf Nuclear Station	DOCKET NUMBER (2) 0506141692	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

A. Reportable Occurrence

On April 21, 1992, during performance of a surveillance which includes testing of reactor water sample valve isolation logic, an inadvertent RWCU system [CE] isolation occurred. This event is classified as an Engineered Safety Feature (ESF) actuation and is being reported pursuant to 10CFR50.73(a)(2)(iv).

B. Initial conditions

The plant was in Mode 5, Refueling during the occurrence.

C. Description of Occurrence

On April 21, 1992, during performance of the reactor water isolation logic surveillance, an inadvertent RWCU isolation occurred at 2352 hours as a result of personnel error.

While performing the surveillance, the utility licensed Nuclear Operator A (NOA) proceeded to activate each reactor water sample isolation logic test switches on two separate control room panels as instructed by procedure. The NOA inadvertently activated the RWCU isolation logic test switches instead, which immediately resulted in closure of inboard RWCU containment isolation valves. The RWCU system was restored seven minutes later at 2359 hours.

D. Apparent Cause

The primary cause of the inadvertent RWCU isolation was personnel error resulting from inattention to detail. Just prior to assisting with the reactor water sample valve surveillance, the NOA was involved with RWCU maintenance activities which included removing a RWCU system filter from service.

The test switches associated with both activities are arranged on each end of both panel sectors as shown below. The RWCU System Logic Test switch is located at the left end and the Reactor Water Sample Logic Test switch is located at the right end.

RWCU SYSTEM LOGIC TEST

MSIV LOGIC LOGIC TEST

ISOLATION LOGIC TEST

RHR LOGIC TEST

REACTOR WATER SAMPLE LOGIC TEST

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 305A's) (17)

The NOA, upon completion of the previous RWCU maintenance activity, was assigned to assist with the reactor water sample valve isolation logic testing. The previous activity obscured the NOA's train of thought in that he proceeded to manipulate RWCU switches instead of the reactor water sample switches as specified by the surveillance procedure.

Similar events also due to personnel error occurred on September 28, 1988 (LER 88-017) and September 29, 1989 (LER 89-014). The 1988 event occurred during a monthly surveillance when an operator was to place the RWCU bypass switch to 'BYPASS' but manipulated the Residual Heat Removal (RHR) [BO] bypass switch instead. The operator had been assisting with RHR isolation logic testing earlier.

The 1989 event occurred during an annual surveillance when an operator was to place the 'A' Reactor Core Isolation Cooling (RCIC) [BN] bypass switch to 'BYPASS' but placed the RWCU bypass switch to 'NORMAL'. As a result of the 1989 event, a Management Standard was issued January 1989 which established a programmatic method of performing self-verifications during work tasks. The Management Standard was not proceduralized at this time, therefore this deficiency was a contributing factor to the April 1992 event.

E. Supplemental Corrective Actions

As a result of this event, the Conduct of Operations procedure, 01-S-06-2 and the Control and Use of Operations Section procedure, 02-S-01-2 were modified to implement the Management Standard of self-verification to provide self-verification guidelines for use during performance of operating and surveillance procedures, valve line manipulations, and red tag clearances.

The Training Department is initiating program changes to instill better work habits on self-verification for initial licensed and non-licensed operator training.

Other recent personnel error occurrences were given management attention through site meetings and distribution of newsletters to reiterate the importance of self-verification during performance of work tasks. The NOA was counselled on his inadequate practice of self-verification.

F. Safety Assessment

The actuation of the RWCU isolation system did not compromise the safe operation of GGNS. All safety related equipment operated as designed. Safety and health of the general public were not affected by this event.

NRC Form 388A
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

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TEXT (If more space is required, use additional NRC Form 388A (2) (17))

G. Additional Information

Energy Industry Identification System (EIIS) codes are identified in the text within brackets [].

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