Omaha Public Power District 444 South 16th Street Mall Omaha, Nebraska 68102-2247 402/636-2000

May 20, 1992 LIC-92-137L

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Mail Station P1-137 Washington, DC 20555

Reference: Docket No. 50-321

Gentlemen:

Subject: Licensee Event Report 92-017 for the Fort Calhoun Station

Please find attached Licensee Event Report 92-017 dated May 20, 1992. This report is being submitted pur uant to 10 CFR 50.73(a)(2)(v)(D). If you should have any questions, please contact me.

Sincerely,

and xene

W. G. Gates Division Manager Nuclear Operations

WGG/lah

Attachment

c: R. D. Martin, NRC Regional Administrator, Region IV D. L. Wigginton, NRC Senior Project Manager S. D. Bloom, NRC Project Engineer R. P. Mullikin, NRC Senior Resident Inspector INPO Records Center



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FACILITY NAME (1) FORT Calhoun Station Unit No. 1	EX ESTIMATED BURDEN PER INFORMATIC'N COLLECT COMMENTS REGARDING AND REPORTS MANAGEN PEGULATORY COMMISSI THE PAPERWORK REDUC OF MANAGEMENT AND B DOCKET NUMBER (PIRES: 4/30/32 R. RESPONSE TO COMPLY WITH THIS ON REQUISET 50.0 HRS. FORMAFD BURDEN EXTIMATE TO THE RECORDS RENT BRANCH (P-330), U.S. NUCLEAR ON, WASHINGTON, DC 20556, AND TO
FACILITY NAME (1) Fort Calhoun Station Unit No. 1	DOCKET NUMBER (THON PROJECT (3150-0104), OFFICE
mie (4)		20 PAGE (3)
President of Prop Palls was presented Plants and Plants	0 5 0 0	0 2 8 5 1 OF 0 4
EVENT DATE 06 LEB NUMBER 06 REPORT DATE 01	Control Switches	S and company
MONTH DAY YEAR YEAR SEQUENTIAL REVISION MONTH DAY YEAR FACILITY NAME		DOCKET NUMBER(S)
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	r more of the following) (11)	0 5 0 0 0
MODE (s) 5 20.402(b) 20.405(c) 50.73(a	1(2)(IV)	7ú.71(b)
LEVEL 0 0 20.405(a) (1)(0) 50.73(a) 109 20.405(a) (1)(0) 50.36(c)(1) X 50.73(a) 20.405(a) (1)(0) 50.73(a) (2)(0) 50.73(a) 50.73(a) 20.405(a) (1)(W) 50.73(a) (2)(0) 50.73(a) 50.73(a) 20.405(a) (1)(W) 50.73(a) (2)(0) 50.73(a) 50.73(a) 20.405(a) (1)(W) 50.73(a) (2)(0) 50.73(a) 50.73(a)	(27) (M) (27) (MI) (27) (MII) (27) (MII) (B) (12) (MI)	73.71(c) OTHER (Specify In Abstract prices and in Test, NRC Form 346A)
NAME LICENSEE CONTACT FOR THIS LER (12)		TELEPHON & NUMBER
James R. Geschwender, Station Licensing Engineer	4 0 0 2	5 3 3 - 6 8 5 7
COMPLETE ONE UNE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS I	REPORT (18)	
CAUSE SYSTEM COMPONENT MANUFAC- TURER TO NPRDS CAUSE SYSTEM O	OMPOWENT MANUFAC	TO NPROS
B E [A 5] 2]] G [0 8 0 Y		
YES (If yes, complete EXPECTED SUBMISSION DATE)	EXPEC 1 RUBATE 11	D MONTH DAY YEAR
ABATTACT (Limit to 1400 spaces, i.e., approximately Theor single space typeworther times) (16) On March 14, 1992, with the Fort Calhoun Station in Mode 5 General Electric Type SBM Switch associated with a 4160V by broken contact. An inspection of the switch revealed that broken contact had cracked and rendered the contact unable additional SBM switches associated with 4160V switchgear re- on additional Lexan cam followers. Some of these were miss material. On April 20, 1992 at 1524, with the plant in Mode potential common mode failure mechanism existed. Based upon the ability to manually operate breakers, poten- cracked cam followers was determined to have no immediate splant in refueling shutdown. The root cause of the event is considered to be mechanical followers caused by inadequacies in the initial assembly of manufacturer, with a contributing cause of mechanical stree operation over the life of the plant. SBM switches associated with 4160V switchgear and critical Instrument Panel mounted switches were inspected. Switcher determined to require replacement prior to resuming power of	(Refueling Shut reaker was found a Lexan cam fol to close. Insp evealed indication sing pieces of c de 5, it was det tial switch fail safety implication stress cracking f the switches a ssos induced the Control Board a s which were insoperation, have	tdown), a to have a llower for the pection of ions of cracks cam follower termined that a lure due to ions with the hough the cam it the rough switch and Auxiliary spected and been replaced.

NRC FORM SBEA (8-89)	OPIM SINGA U.B. NUCLEAR REGULATORY COMMISSION			
LICENSEE EVENT REPORT (LER) TEXT CONTINUATION		EXTINATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST BOLD HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-533), D.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20556, AND TO THE FARFRWORK REDUCTION PROLECT (2150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20563.		
FACILITY NAME (1)	DOOKET NUMBER (2)	LER NUMBER (8) PAGE (3)		
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TEXT (If more space is required, use additional NRC Form 3664's)(17)				

General Electric (GE) Type SBM Control Switches are used in 4160V and 480V switchgear, and Control Board and Auxiliary Instrument Panels at the Fort Calhoun Station (FCS). Cam followers in these switches may be composed of Lexan (a transparent polycarbonate) or Delrin (a milky-white acetal resin). Previous industry experience regarding SBM switches manufactured prior to 1976 (including NRC Information Notice 80-13) has indicated a problem involving cracking of polycarbonate cam followers exposed to hydrocarbons during fabrication or maintenance. The Omaha Public Power District review of this issue resulted in a 1984 recommendation to replace safety related SBM switches. Replacement was not pursued for SBM switches located on the 4160V switchgear based on a finding that a sample of eight SBM switches on the 4160V switchgear all had Delrin cam followers. Thirty SBM switches in the Control Room were replaced during the 1985 Refueling Outage. Replacement of additional SBM switches was not recommended at that time based upon results of inspection of the replaced switches which revealed only minor cracking due to stress, not hydrocarbon exposure.

On March 14, 1992, while FCS was in a Refueling Outage (in Mode 5, Refueling Shutdown), it was discovered that a contact was broken in an auxiliary switch (General Electric SBM type) associated with the 4160V breaker for Raw Water Pump AC-10A. The broken contact (contact did not close when switch was operated) was discovered during post-modification testing. The contact was a permissive for auto-closure of the output breaker for Emergency Diesel Generator No. 1 (DG-1). The contact operates one of two redundant circuits which can initiate auto-closure of the DG-1 output breaker. The operability of DG-1 was not in question, since the failure represented a degradation of redundancy of the actuation circuits. The contact function necessary to allow DG-1 to auto-close onto the bus would have been fulfilled by the redundant circuit. A Station Incident Report was initiated to evaluate the failure, and the switch was replaced on April 1, 1992.

An inspection of the switch was conducted on April 17, 1992, in response to the Station Incident Report. This inspection revealed that a plastic cam follower for the broken contact had cracked and allowed the roll pin to fall out of the switch, rendering the contact unable to close. Based on previous industry experience, the cracking of the contact associated with Raw Water Pump AC-10A was initially believed to have resulted from exposure to hydrocarbons. A sample of SBM switches associated with 4160V switchgear was inspected on April 18, 1992. This sample indicated that approximately 20 percent of the switches had Lexan cam followers with varying degrees of cracking, and approximately 80 percent had Delrin cam followers (Delrin does not exhibit cracking). Based on the sample results, it was decided to conduct a systematic inspection of the cam followers in the SBM switches associated with the 4160V switchgear.

Inspection of the SBM switches associated with 4160V switchgear was completed on April 19, 1992. The inspection revealed that 136 of the SBM switches associated with 4160V switchgear were composed of Delrin, and 55 had Lexan cam followers. Of the 55 SBM switches with Lexan cam followers, 40 were found to have indications of cracks on cam followers. Of the 40 SBM switches with indications of cracks, 15 had missing pieces of cam follower material and an additional six had open cracks.

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(6-89)		APPROVED OMB NO. 3100-0104		
LICENSEE EVENT REPORT (LER)		EXTINGE 4/30/02		
TEXT CONTINUATION		ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THE INFORMATION COLLECTION REQUEST: BO.6 HRS, FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS		
		AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO		
		THE PAPERWORK REDUCTION PROJECT (\$150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.		
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		YEAR SEQUENTIAL REVISION NUMBER NUMBER		
Fort Calhoun Station Unit No. 1				
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TEXT (If more space is required, use additional NRC Form 386A sr(17)				
On April 20, 1992 at 1524, with the pl condition existed. Based upon evaluat common mode failure mechanism was iden 20, 1992 at 1632, pursuant to 10 CFR 5 interference with the safety function an accident. This report is submitted the ability to manually operate breake followers was determined to have no im refueling shutdown.	lant in Mode 5, it was tion of the switch fai htified. A four-hour 50.72(b)(2)(iii)(D), b of systems needed to i pursuant to 10 CFR 5 ers, potential switch mmediate safety implic	determined that a reportable ilurc mechanism, a potential notification was made on April based upon the potential for mitigate the consequences of 50.73(a)(2)(v)(D). Based upon failure due to cracked cam cations with the plant in		
Criteria were established for use in d	letermining the need t	o replace SBM switches:		
 All switches with missing cam follower pieces, broken cam followers, or "open cracks" in cam followers, must be replaced. 				
2) Switches with only "tight cracks" in cam followers or a mixture of "tight cracks" and no cracks in cam followers, do not require switch rep ¹ .ment.				
These criteria were based upon industr cam followers. Testing conducted by G followers with "worse than visible hai 100,000 operations. One switch tested without a failure.	y data regarding test E in 1976, using 'cra ir-line cracks"), prod I by CT reached more t	ing of switches with cracked ized" cam followers (i.e., cam luced no failures prior to than 1,450,000 operations		
Following the inspection of the SBM sw which were found to be damaged were fu identified as requiring replacement we in the 4160V switchgear revealed deter applications was pursued.	vitches associated wit wither reviewed. Of t ere replaced. Since t vioration, inspection	th the 4160V switchgear, those these switches, those the inspection of the switches of SBM switches in other		
A sample of 10 readily accessible switt Based on the results of this initial is switches should be inspected. In order inspected, Engineering evaluated a com the critical switches for safety relat Refueling Outage were not considered a recommended for additional Control Roo Engineering Analysis EA-FC-92-032. The switches selected for inspection will basis.	ches in the Control R inspection, it was det inspection, it was det inprehensive list of SB ed functions. (Switch is they have Delrin ca om switches which met e analysis indicated ensure plant safe shu	Room were initially inspected. cermined that additional switches needed to be OM switches at FCS to identify thes replaced during the 1985 im followers.) Inspection was criteria documented in that the subset of all SBM itdown within the FCS design		

NPO FORM SERA (6-89)		U.S. NUCLEAR REGULATORY COMMISSION	APPROVED OMB NO. 3150-0104 EXPIRES: 4/30/92		
LICENSEE EVENT REPORT (LER) TEXT CONTINUATION		(LER)	ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION , CRUEST, 50:0 HRS, FORWARD COMMENTS REGARDING BURDEN SSTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-830), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20055, AND TO THE PAPETWORK REDUCTION PROJECT 3130-0104), OFFICE OF MANAGEMENT AND BURDET WASHINGTON, DC 20053		
ACHLITY NAME (1)		DOCKET NUMBER (2)	LER NUMBER (0) PAGE (0)		
		Level 1 to the second	YEAR SEQUENTIAL REVISION		
Fort Calhoun Station Unit No. 1	0 0 0 0 0 0 0 0 0 0 0 0 0				
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As a result of the requiring replace inspection, repla documented in Eng	ese inspections, seve ment, based on the pr cement and acceptabil ineering Analysis EA-	eral switches in the (reviously established lity of the current p -FC-92-032.	Control Room were identified as replacement criteria. The lant configuration is		
The event is sign prevent certain a breaker closure, pin to work free (closing) during breakers (if auto cracked cam follo plant in refuelin of switches which mitigate the cons	ificant in that postu utomatic safety funct from occurring on dem from the cam and ever switch operation. Ho matic features had be wers was determined t g shutdown. Subseque were not inspected a equences of an accide	alated failures of car tions, such as load sh mand. Cracked cam fo tually prevent the ar owever, based upon the een disabled), potent to hav no immediate s ent replacement of se and/or replaced has ju	n follower contacts could ned and diesel generator llowers could allow the roll ffected contact from operating e ability to manually operate ial switch failure due to safety implications with the lected switches and evaluation ustified the ability to ation.		
The root cause of followers caused manufacturer, wit operation over the	the event is conside by inadequacies in th h a contributing caus e life of the plant.	ered to be mechanical ne initial assembly of se of mechanical stres	stress cracking of the cam f the switches at the sses induced through switch		
Corrective action	s which have been com	mpleted as a result of	f this event are:		
1) SBM s and A	witches associated w uxiliary Instrument H	ith 4160V switchgear, Panel mourted switche	and critical Control Board s have been inspected.		
2) SBM s ensur	SBM switches which were inspected and found to require replacement (to ensure plant safe shutdown within the FCS design basis), have been replaced.				
Corrective action	s which will be compl	leted as a result of	this event are:		
1) Five Refue switc	additional SBM switch ling Outage (justific hes is provided in E/	hes will be replaced cation for the replac A-FC-92-032).	by the end of the 1993 ement schedule for these		
2) Furth deter balan a pri deter	er review of the fai mination made as to u ce of the switches in ority schedule, or re mination of long-term	f the failure mecrenism will be conducted and a de as to what long-term actions are required to address the witches in the plant (i.e., periodically inspect, replace on ule, or replace on an as-fail basis). This review and long-term action will be completed by December 31, 1992.			
This is the first	event to be reported	involving the failur	re of SBM switches at FCS.		